



FPL

JUN 27 1990

L-90-216

U. S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, D. C. 20555


Gentlemen:

Re: Turkey Point Units 3 and 4  
Docket Nos. 50-250 and 50-251  
Request for Information on the Status of Licensee  
Implementation of Generic Safety Issues Resolved with  
Imposition of Requirements or Corrective Actions  
(Generic Letter 90-04)

By Generic Letter 90-04, dated April 25, 1990, the NRC staff requested that licensees review the status of, and provide documentation concerning the implementation status of the generic safety issues identified in Generic Letter 90-04. The requested information for Turkey Point Units 3 and 4 is attached.

If further discussion is required on this topic, please contact us.

Very truly yours,

  
K. N. Harris  
Vice President  
Turkey Point Plant

KNH/TCG/gp

Attachment

cc: Stewart D. Ebnetter, Regional Administrator, Region II, USNRC  
Senior Resident Inspector, USNRC, Turkey Point Plant

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FDR ADOCK 05000250  
P FDC

FACILITY NAME: Turkey Point Unit 4  
DOCKET NO.: 50-251  
LICENSEE: Florida Power and Light Co.

STATUS OF LICENSEE IMPLEMENTATION OF GENERIC SAFETY ISSUES  
RESOLVED WITH IMPOSITION OF REQUIREMENTS OR CORRECTIVE ACTIONS

<u>GSI/(MPA No.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS*</u>	<u>COMMENTS</u>
40 (B065)	Safety Concerns Associated With Pipe Breaks In The BWR Scram System	All BWRs		NA
41 (B058)	BWR Scram Discharge Volume Systems	All BWRs		NA
43 (B107)	Reliability Of Air Systems	All Plants	I	Note 1
51 (L913)	Improving the Reliability of Open-Cycle Service Water Systems	All Plants	I	L-90-29. dated Jan. 30, 1990
67.3.3 (A017)	Improved Accident Monitoring	All Plants	I (12/90)	Note 2
75** (B076)	Item 1.1 - Post-Trip Review (Program Description and Procedure)	All Plants	C	Note 3
75 (B085)	Item 1.2 - Post-Trip Review - Data and Information Capability	All Plants	C	Note 3

\*Please follow attached guidance for completing this column.

\*\*The 16 items listed for GSI 75 all relate to actions derived from the generic implications of Salem ATWS events.  
Item numbers correspond to Generic Letter 83-28 action item numbers.

<u>GSI/(MPA No.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS*</u>	<u>COMMENTS</u>
75 (B077)	Item 2.1 - Equipment Classification and Vendor Interface (Reactor Trip System Components)	All Plants	C	Note 3
75 (B086)	Item 2.2.1 - Equipment Classification for Safety-Related Components	All Plants	I	Notes 3, 5
75 (L003)	Item 2.2.2 - Vendor Interface for Safety-Related Components	All Plants	I	Notes 3, 5
75 (B078)	Items 3.1.1 & 3.1.2 - Post - Maintenance Testing (Reactor Trip System Components)	All Plants	C	Note 3
75 (B079)	Item 3.1.3 - Post-Maintenance Testing-Changes to Test Requirements (Reactor Trip System Components)	All Plants	C	Note 3
75 (B087)	Items 3.2.1 & 3.2.2 - Post-Maintenance Testing (All Other Safety-Related Components)	All Plants	I (9/1/90)	Note 3
75 (B088)	Item 3.2.3 - Post-Maintenance Testing-Changes to Test Requirements (All Other Safety-Related Components)	All Plants	C	Note 3
75 (B080)	Item 4.1 - Reactor Trip System Reliability (Vendor-Related Modifications)	All Plants	C	Note 3

<u>GSI/(MPA No.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS*</u>	<u>COMMENTS</u>
75 (B081)	Items 4.2.1 & 4.2.2 - Reactor Trip System Reliability-Maintenance and Testing (Preventative Maintenance and Surveillance Program for Reactor Trip Breakers)	All PWRs	C	Note 3
75 (B082)	Item 4.3 - Reactor Trip System Reliability - Design Modifications (Automatic Actuation of Shunt Trip Attachment for Westinghouse and B&W Plants)	All W and B&W Plants	C	Note 3
75 (B090)	Item 4.3 - Reactor Trip System Reliability - Tech Spec Changes (Automatic Actuation of Shunt Trip Attachment For Westinghouse and BW Plants)	All W & B&W Plants	C	Notes 3, 5
75 (B091)	Item 4.4 - Reactor Trip System Reliability (Improvements in Maintenance and Test Procedures for B&W Plants)	All B&W Plants	NA	

<u>GS1/(MPA No.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS*</u>	<u>COMMENTS</u>
75 (B092)	Item 4.5.1 - Reactor Trip System Reliability-Diverse Trip Features (System Functional Testing)	All Plants	C	Note 3
75 (B093)	Items 4.5.2 & 4.5.3 - Reactor Trip System Reliability - Test Alternatives and Intervals (System Functional Testing)	All Plants	C	Note 3
86 (B084)	Long Range Plan for Dealing with Stress Corrosion Cracking in BWR Piping	All BWRs	NA	
93 (B098)	Steam Binding of Auxiliary Feedwater Pumps	All PWRs	C	L-88-237, dated May 24, 1988
99 (L817)	RCS/RHR Suction Line Valve Interlock on PWRs	All PWRs	I	Notes 4, 5
124	Auxiliary Feedwater System Reliability	ANO-1&2, Rancho Seco, Prairie Island 1&2, Crystal River-3 Ft. Calhoun	NA	
A-13 (B017)	Snubber Operability Assurance - Hydraulic Snubbers	All Plants	NA	See B022 Hydraulic snubbers have been replaced with mechanical snubbers

<u>GSI/(MPA No.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS*</u>	<u>COMMENTS</u>
A-13 (B022)	Snubber Operability Assurance - Mechanical Snubbers	All Plants	C	Amend 90, dated Oct 14, 1983
A-16 (D012)	Steam Effects on BWR Core Spray Distribution	Oyster Creek & NMP-1	NA	
A-35 (B023)	Adequacy of Offsite Power Systems	All Plants	C	Amend 98, dated Aug 14, 1984
B-10	Behavior of BWR Mark III Containments	All BWR Mark III Plants	NA	
B-36	Develop Design, Testing and Maintenance Criteria for Atmosphere Cleanup System Air Filtration and Adsorption Units for Engineered Safety Features Systems and for Normal Ventilation Systems	All Plants with OL Applications After 4/1/80	NA	
B-63 (B045)	Isolation of Low Pressure Systems Connected to the Reactor Coolant System Pressure Boundary	All Plants	C	Order dated April 20, 1981 L-87-264, dated June 23, 1987

#### NOTES

1. In letter L-90-197 dated May 30, 1990, FPL reported that the design and operations verification of the instrument air system, as required by GL 88-14 was complete. Schedules for action items resulting from the required verification are stated in that letter.
2. The Area Radiation Monitoring System upgrade will be completed by December 31, 1990. An NRC review of compliance with Regulatory Guide 1.97 requirements was included in NRC Inspection Report 50-250/90-01 and 50-251/90-01 dated February 2, 1990.
3. A detailed status of implementation of required actions based on the generic implications of the ATWS events at the Salem Nuclear Plant was provided by FPL in letter L-90-95 dated April 5, 1990. FPL is reviewing GL 90-03, Relaxation of Staff Position in Generic Letter 83-28, Item 2.2, Part 2, "Vendor Interface for Safety-Related Components", and will provide a response by September 29, 1990.
4. FPL's response to GL 88-17, "Loss of Decay Heat Removal", was provided in letters L-88-559 dated January 3, 1989 and February 1, 1989. Required action items and schedules for their completion were identified in that letter.
5. The NRC has not yet completed its review of this item.

FACILITY NAME: Turkey Point Unit 4  
 DOCKET NO.: 50-251  
 LICENSEE: Florida Power & Light Company

GSI Status Summary

<u>GSI/MPA No.</u>	<u>Title</u>	<u>Licensee Status Determination</u>	<u>Staff Status Determination</u>	<u>Remarks</u>
40 (B065)	Safety Concerns Associated With Pipe Breaks In The BWR Scram System	N/A	N/A	
41 (B058)	BWR Scram Discharge Volume Systems	N/A	N/A	
43 (B107)	Reliability Of Air Systems	I	I	Licensee met requirements of GL 88-14 by letter dated 5/30/90. Licensee still implementing identified improvements. Expect to complete by 12/07/90.
51 (L913)	Improving the Reliability of Open-Cycle Service Water Systems	I	I	Licensee response dated 1/30/90 established the required 5 programs. All additional actions such as reviews or inspections to carry out the programs, are scheduled and documented in 1/30/90 letter, all to be complete by restart from dual unit outage ending in 12/91.
67.3.3 (A017)	Improved Accident Monitoring	I/12-90	I/12-90	RG 1.97 implementation inspected in IR 90-01 and found acceptable. Area Rad. Monitoring System upgrade to be complete 12/31/90. Exception for ARMS had been re- quested to permit a range of $10^{-3}$ to $10^2$ R/hr instead of $10^{-1}$ to $10^4$ . Licensee will withdraw request for exception and meet RG 1.97 by 12/31/90.



<u>GSI/MPA No.</u>	<u>Title</u>	<u>Licensee Status Determination</u>	<u>Staff Status Determination</u>	<u>Remarks</u>
75** (B076)	Item 1.1 - Post-Trip Review (Program Description and Procedure)	C	C 5/31/85	
75 (B085)	Item 1.2 - Post-Trip Review - Data and Information Capability	C	C 8/15/85	
75 (B077)	Item 2.1 - Equipment Classification and Vendor Interface (Reactor Trip System Components)	C	C 4/17/87	
75 (B086)	Item 2.2.1 - Equipment Classification for Safety-Related Components	I	C 4/17/87	Licensee implementation complete. Staff issued SER 8/09/90.
75 (L003)	Item 2.2.2 - Vendor Interface for Safety-Related Components	I	E/09-29-90	GL 90-03 supercedes 2.2.2.
75 (B078)	Items 3.1.1 & 3.1.2 - Post - Maintenance Testing (Reactor Trip System Components)	C	C 7/31/86	

\*\*The dates shown for the Salem ATWS items are contained in a 4/5/90 letter from the licensee.

<u>GSI/MPA No.</u>	<u>Title</u>	<u>Licensee Status Determination</u>	<u>Staff Status Determination</u>	<u>Remarks</u>
75 (B079)	Item 3.1.3 - Post-Maintenance Testing - Changes to Test Requirements (Reactor Trip System Components)	C	C 7/31/86	
75 (B087)	Items 3.2.1 & 3.2.2 - Post-Maintenance Testing (All Other Safety-Related Components)	I/9-1-90	I/10/15/90	Licensee has indicated verbally item was completed about 9/1/90. Letter to NRC documenting completion expected by 10/15/90.
75 (B088)	Item 3.2.3 - Post-Maintenance Testing-Changes to Test Requirements (All Other Safety-Related Components)	C	C 7/31/86	SER dated 11/14/85.
75 (B080)	Item 4.1 - Reactor Trip System Reliability (Vendor-Related Modifications)	C	C 11/8/83	

<u>GSI/MPA No.</u>	<u>Title</u>	<u>Licensee Status Determination</u>	<u>Staff Status Determination</u>	<u>Remarks</u>
75 (B081)	Items 4.2.1 & 4.2.2 - Reactor Trip System Reliability-Maintenance and Testing (Preventative Maintenance and Surveillance Program for Reactor Trip Breakers)	C	C 1/18/85	
75 (B082)	Item 4.3 - Reactor Trip System Reliability - Design Modifications (Automatic Actuation of Shunt Trip Attachment for Westinghouse and B&W Plants)	C	C 6/30/86	
75 (B090)	Item 4.3 - Reactor Trip System Reliability - Tech Spec Changes (Automatic Actuation of Shunt Trip Attachment For Westinghouse and B&W Plants)	C	C 12/31/87	Licensee's design accepted by NRC. TS are part of the TS upgrade amendment issued 8/28/90. Implementation required by 8/28/91. TS have been implemented in 1987 by administrative procedure.
75 (B091)	Item 4.4 - Reactor Trip System Reliability (Improvements in Maintenance and Test Procedures for B&W Plants)	N/A	N/A	

<u>GSI/MPA No.</u>	<u>Title</u>	<u>Licensee Status Determination</u>	<u>Staff Status Determination</u>	<u>Remarks</u>
75 (B092)	Item 4.5.1 - Reactor Trip System Reliability-Diverse Trip Features (System Functional Testing)	C	C 5/1/86	
75 (B093)	Items 4.5.2 & 4.5.3 - Reactor Trip System Reliability - Test Alternatives and Intervals (System Functional Testing)	C	C 5/1/86	
86 (B084)	Long Range Plan for Dealing with Stress Corrosion Cracking in BWR Piping	N/A	N/A	
93 (B098)	Steam Binding of Auxiliary Feedwater Pumps	C	C 5/24/88	NRC letter dated 6/15/88.
99 (L817)	RCS/RHR Suction Line Valve Interlock on PWRs	I	I 12/91	GL 88-17, Licensee actions/schedules provided in letters dated 1/3/89, 2/1/89. Staff review not complete. Implementation expected to be complete 12/91.
124	Auxiliary Feedwater System Reliability	N/A	N/A	
A-13 (B017)	Snubber Operability Assurance - Hydraulic Snubbers	N/A	N/A	

<u>GSI/MPA No.</u>	<u>Title</u>	<u>Licensee Status Determination</u>	<u>Staff Status Determination</u>	<u>Remarks</u>
A-13 (B022)	Snubber Operability Assurance - Mechanical Snubbers	C	C 10/14/83	Amendments 96 and 90 for Units 3 and 4
A-16 (D012)	Steam Effects on BWR Core Spray Distribution	N/A	N/A	
A-35 (B023)	Adequacy of Offsite Power Systems	C	C 5/10/84	Licensee (G. Salamon) indicated verbally that Plant Change Modification #80-44 dated 5/10/84 was completed. See also FPL letter L-85-347 to J. Nelson Grace (Region II) dated 9/3/85, "10 CFR 50.59 Report."
B-10	Behavior of BWR Mark III Containments	N/A	N/A	
B-36	Develop Design, Testing and Maintenance Criteria for Atmosphere Cleanup System Air Filtration and Adsorption Units for Engineered Safety Features Systems and for Normal Ventilation Systems	N/A	N/A	

<u>GSI/MPA No.</u>	<u>Title</u>	<u>Licensee Status Determination</u>	<u>Staff Status Determination</u>	<u>Remarks</u>
B-63 (B045)	Isolation of Low Pressure Systems Connected to the Reactor Coolant System Pressure Boundary	C	C 4/20/81	

Item

99 (L817)  
(cont)

Status

RCS/RHR Suction Line Valve Interlock on PWRs

(7) Provide permanent scale in containment for Tygon tube level indication

Both reactor units will shut down beginning in late 1990, for about a year or more, to install two additional diesel generators. Implementation of the above modifications will proceed during the outage. All implementation on this issue is expected to be complete by 12/91. This is an acceptable schedule.



Gordon E. Edison, Sr. Project Manager  
Project Directorate II-2  
Division of Reactor Projects I/II

Enclosures: As stated

FACILITY NAME: Turkey Point Unit 4  
 DOCKET NO.: 50-251  
 LICENSEE: Florida Power & Light Company

GSI Status Summary

<u>GSI/MPA No.</u>	<u>Title</u>	<u>Licensee Status Determination</u>	<u>Staff Status Determination</u>	<u>Remarks</u>
40 (B065)	Safety Concerns Associated With Pipe Breaks In The BWR Scram System	N/A	N/A	
41 (B058)	BWR Scram Discharge Volume Systems	N/A	N/A	
43 (B107)	Reliability Of Air Systems	I	I	Licensee met requirements of GL 88-14 by letter dated 5/30/90. Licensee still implementing identified improvements. Expect to complete by 12/07/90.
51 (L913)	Improving the Reliability of Open-Cycle Service Water Systems	I	I	Licensee response dated 1/30/90 established the required 5 programs. All additional actions such as reviews or inspections to carry out the programs, are scheduled and documented in 1/30/90 letter, all to be complete by restart from dual unit outage ending in 12/91.
67.3.3 (A017)	Improved Accident Monitoring	I/12-90	I/12-90	RG 1.97 implementation inspected in IR 90-01 and found acceptable. Area Rad. Monitoring System upgrade to be complete 12/31/90. Exception for ARMS had been requested to permit a range of $10^{-3}$ to $10^2$ R/hr instead of $10^{-1}$ to $10^4$ . Licensee will withdraw request for exception and meet RG 1.97 by 12/31/90.



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<u>GSI/MPA No.</u>	<u>Title</u>	<u>Licensee Status Determination</u>	<u>Staff Status Determination</u>	<u>Remarks</u>
75** (B076)	Item 1.1 - Post-Trip Review (Program Description and Procedure)	C	C 5/31/85	
75 (B085)	Item 1.2 - Post-Trip Review - Data and Information Capability	C	C 8/15/85	
75 (B077)	Item 2.1 - Equipment Classification and Vendor Interface (Reactor Trip System Components)	C	C 4/17/87	
75 (B086)	Item 2.2.1 - Equipment Classification for Safety-Related Components	I	C 4/17/87	Licensee implementation complete. Staff issued SER 8/09/90.
75 (L003)	Item 2.2.2 - Vendor Interface for Safety-Related Components	I	E/09-29-90	GL 90-03 supercedes 2.2.2.
75 (B078)	Items 3.1.1 & 3.1.2 - Post - Maintenance Testing (Reactor Trip System Components)	C	C 7/31/86	

\*\*The dates shown for the Salem ATWS items are contained in a 4/5/90 letter from the licensee.

<u>GSI/MPA No.</u>	<u>Title</u>	<u>Licensee Status Determination</u>	<u>Staff Status Determination</u>	<u>Remarks</u>
75 (B079)	Item 3.1.3 - Post-Maintenance Testing - Changes to Test Requirements (Reactor Trip System Components)	C	C 7/31/86	
75 (B087)	Items 3.2.1 & 3.2.2 - Post-Maintenance Testing (All Other Safety-Related Components)	I/9-1-90	I/10/15/90	Licensee has indicated verbally item was completed about 9/1/90. Letter to NRC documenting completion expected by 10/15/90.
75 (B088)	Item 3.2.3 - Post-Maintenance Testing-Changes to Test Requirements (All Other Safety-Related Components)	C	C 7/31/86	SER dated 11/14/85.
75 (B080)	Item 4.1 - Reactor Trip System Reliability (Vendor-Related Modifications)	C	C 11/8/83	

<u>GSI/MPA No.</u>	<u>Title</u>	<u>Licensee Status Determination</u>	<u>Staff Status Determination</u>	<u>Remarks</u>
75 (B081)	Items 4.2.1 & 4.2.2 - Reactor Trip System Reliability-Maintenance and Testing (Preventative Maintenance and Surveillance Program for Reactor Trip Breakers)	C	C 1/18/85	
75 (B082)	Item 4.3 - Reactor Trip System Reliability - Design Modifications (Automatic Actuation of Shunt Trip Attachment for Westinghouse and B&W Plants)	C	C 6/30/86	
75 (B090)	Item 4.3 - Reactor Trip System Reliability - Tech Spec Changes (Automatic Actuation of Shunt Trip Attachment For Westinghouse and B&W Plants)	C	C 12/31/87	Licensee's design accepted by NRC. TS are part of the TS upgrade amendment issued 8/28/90. Implementation required by 8/28/91. TS have been implemented in 1987 by administrative procedure.
75 (B091)	Item 4.4 - Reactor Trip System Reliability (Improvements in Maintenance and Test Procedures for B&W Plants)	N/A	N/A	

<u>GSI/MPA No.</u>	<u>Title</u>	<u>Licensee Status Determination</u>	<u>Staff Status Determination</u>	<u>Remarks</u>
75 (B092)	Item 4.5.1 - Reactor Trip System Reliability-Diverse Trip Features (System Functional Testing)	C	C 5/1/86	
75 (B093)	Items 4.5.2 & 4.5.3 - Reactor Trip System Reliability - Test Alternatives and Intervals (System Functional Testing)	C	C 5/1/86	
86 (B084)	Long Range Plan for Dealing with Stress Corrosion Cracking in BWR Piping	N/A	N/A	
93 (B098)	Steam Binding of Auxiliary Feedwater Pumps	C	C 5/24/88	NRC letter dated 6/15/88.
99 (L817)	RCS/RHR Suction Line Valve Interlock on PWRs	I	I 12/91	GL 88-17, Licensee actions/schedules provided in letters dated 1/3/89, 2/1/89. Staff review not complete. Implementation expected to be complete 12/91.
124	Auxiliary Feedwater System Reliability	N/A	N/A	
A-13 (B017)	Snubber Operability Assurance - Hydraulic Snubbers	N/A	N/A	

<u>GSI/MPA No.</u>	<u>Title</u>	<u>Licensee Status Determination</u>	<u>Staff Status Determination</u>	<u>Remarks</u>
A-13 (B022)	Snubber Operability Assurance - Mechanical Snubbers	C	C 10/14/83	Amendments 96 and 90 for Units 3 and 4
A-16 (D012)	Steam Effects on BWR Core Spray Distribution	N/A	N/A	
A-35 (B023)	Adequacy of Offsite Power Systems	C	C 5/10/84	Licensee (G. Salamon) indicated verbally that Plant Change Modification #80-44 dated 5/10/84 was completed. See also FPL letter L-85-347 to J. Nelson Grace (Region II) dated 9/3/85, "10 CFR 50.59 Report."
B-10	Behavior of BWR Mark III Containments	N/A	N/A	
B-36	Develop Design, Testing and Maintenance Criteria for Atmosphere Cleanup System Air Filtration and Adsorption Units for Engineered Safety Features Systems and for Normal Ventilation Systems	N/A	N/A	

<u>GSI/MPA No.</u>	<u>Title</u>	<u>Licensee Status Determination</u>	<u>Staff Status Determination</u>	<u>Remarks</u>
B-63 (B045)	Isolation of Low Pressure Systems Connected to the Reactor Coolant System Pressure Boundary	C	C 4/20/81	

Item

Status

99 (L817)  
(cont)

RCS/RHR-Suction Line-Valve-Interlock-on-PWRs

(7) Provide permanent scale in containment for Tygon tube level indication

Both reactor units will shut down beginning in late 1990, for about a year or more, to install two additional diesel generators. Implementation of the above modifications will proceed during the outage. All implementation on this issue is expected to be complete by 12/91. This is an acceptable schedule.

Original signed by

Gordon E. Edison, Sr. Project Manager  
Project Directorate II-2  
Division of Reactor Projects I/II

Enclosures: As stated

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

October 1, 1990

Docket No. 50-251

MEMORANDUM FOR: Docket File

FROM: Gordon E. Edison, Sr. Project Manager  
Project Directorate II-2  
Division of Reactor Projects - I/II

SUBJECT: STATUS OF RESOLVED GENERIC SAFETY ISSUE  
IMPLEMENTATION REQUIREMENTS AT  
TURKEY POINT UNIT 4

The current status of resolved Generic Safety Issue (GSI) Implementation Requirements at the Turkey Point 4 facility is summarized in the enclosures to this memorandum.

Enclosure 1 contains a copy of the information provided by the licensee in its response to Generic Letter 90-04.

Enclosure 2 is a status summary for each GSI applicable to this facility. This status summary is based upon the licensee's response to the Generic Letter, discussions with the licensee and cognizant staff and my review of available NRC records and information.

Those items considered to be incomplete are listed below together with my assessment of the significance of that status and of the licensee's plans for completion.

Item

Status

43 (B107)

Reliability of Air Systems

In a letter dated 7/25/90 the staff notified the licensee that they had met the requirements of GL 88-14 with their response dated 5/30/90. Licensee will complete implementation of identified improvements by December 7, 1990.

51 (L913)

Improving the Reliability of Open-Cycle Service Water Systems

Licensee response dated 1/30/90 established the required 5 programs with schedules to carry out additional actions such as reviews or inspections. All implementation to be complete by restart from dual unit outage (about 12/91).

Item

Status

67.3.3 (A017)

Improved Accident Monitoring

NRC Inspection Report 90-01, dated 2/12/90, reviewed compliance with RG 1.97. The results of the inspection were that the licensee satisfactorily met the requirements and intent of RG 1.97 guidelines. Last instrumentation to be upgraded (Area Radiation Monitoring) will be implemented by 12/90. This is an acceptable schedule. Licensee had requested relief from  $10^{-1}$  to  $10^4$  R/hr range requirement. Licensee subsequently is planning to withdraw request and meet the requirement.

75 (L003)

Salem ATWS Item 2.2.2 - Vendor Interface for Safety Related Components

Recent guidance (GL 90-03) issued 3/20/90 by staff. Licensee response expected by 9/29/90. This is an acceptable schedule.

75 (B087)

Salem ATWS Items 3.2.1 & 3.2.2 - Post Maintenance Testing (All other Safety Related Components)

Licensee has indicated verbally (G. Salamon to G. Edison) on 9/7/90 that these items were completed about 9/1/90, and a letter to NRC is in preparation to document the completion of implementation. The letter is expected by 10/15/90 and this is an acceptable schedule.

99 (L817)

RCS/RHR Suction Line Valve Interlock on PWRs

Licensee actions and schedules responding to GL 88-17 were provided in letters dated 1/3/89 and 2/1/89.

Staff review is not complete. Licensee is in process of modifications, including:

- (1) RCS level system (alarms in C.R., and transducers)
- (2) Correlation for intermediate-to-hot-leg level
- (3) Installation of redundant independent level channel
- (4) RCS temperature indication
- (5) RHR flow indication
- (6) Review Westinghouse Owners Group analyses and incorporate

Item

Status

99 (L817)  
(cont)

RCS/RHR Suction Line Valve Interlock on PWRs

(7) Provide permanent scale in containment for Tygon tube level indication

Both reactor units will shut down beginning in late 1990, for about a year or more, to install two additional diesel generators. Implementation of the above modifications will proceed during the outage. All implementation on this issue is expected to be complete by 12/91. This is an acceptable schedule.

Original signed by

Gordon E. Edison, Sr. Project Manager  
Project Directorate II-2  
Division of Reactor Projects I/II

Enclosures: As stated

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