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U. S. Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, D. C. 20555

Gentlemen:

Re: Turkey Point Unit 4
Docket No. 50-251
Startup Report

The attached Startup Report is being submitted in accordance with Technical Specification 6.9.1.a. The Unit 4 Cycle XII Startup Report documents the first use of fuel assemblies with Integral Fuel Burnable Absorbers (IFBA), extended burnup modifications, reconstitutable top nozzles, standardized fuel pellets, reduced fuel rod backfill pressures, 4g fuel rod plenum springs, and 304L stainless grid sleeve material.

Very truly yours,

R. J. Acosta
for

E. O. Woody
Acting Senior Vice President - Nuclear

COW/GRM/cm

Attachment

cc: Stewart D. Ebnetter, Regional Administrator, Region II, USNRC
Senior Resident Inspector, USNRC, Turkey Point Plant

8908250320 890821
PDR ADCK 05000251
P PDC

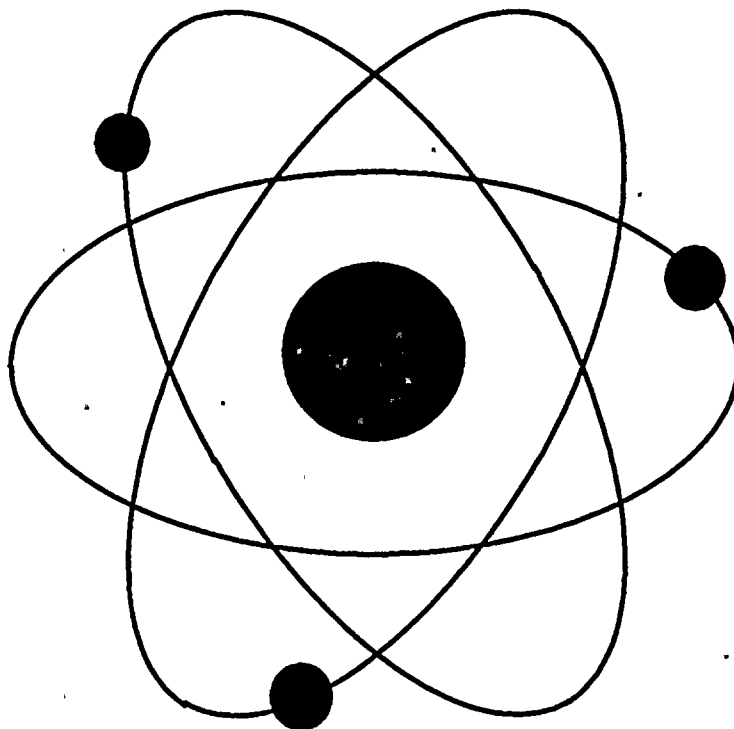
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FLORIDA POWER AND LIGHT COMPANY

TURKEY POINT PLANT

UNIT 4 CYCLE XII

STARTUP REPORT



INTRODUCTION

This report contains the official summary of the Startup Physics Tests performed on Turkey Point Unit 4 at the beginning of Cycle XII. The testing program was conducted in accordance with Operating Procedure 0204.3, Initial Criticality After Refueling, and Operating Procedure 0204.5, Nuclear Design Check Tests During Startup Sequence After Refueling, and meets the minimum requirements of ANSI/ANS 19.6.1, Revision 0 (12-13-85), Startup Physics Tests for Pressurized Water Reactors. Testing commenced on May 19, 1989, at 0148 and was completed on May 21, 1989 at 0637.

The Westinghouse Nuclear Design Report for Unit 4, Cycle XII, (WCAP-12010) is the design data from which deviations were measured for the purpose of verifying that acceptance criteria were met. The acceptance criteria stated are the more conservative of ANSI/ANS 19.6.1., Revision 0 or Operating Procedure 0204.5.

All of the tests included in this report meet their acceptance criteria.

The contents of this report provide the documentation required by Technical Specification 6.9.1.a.



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5

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1.0 UNIT 4 CYCLE XII CORE

This section presents the as-loaded core configuration (Figure 1); the Control and Shutdown Rod pattern (Figure 2); and the Rod Drop Times for all rods as measured in Procedure 4-PMI-028.3 RPI Hot Calibration, CRDM Stepping Test, and Rod Drop Test (Figure 3).

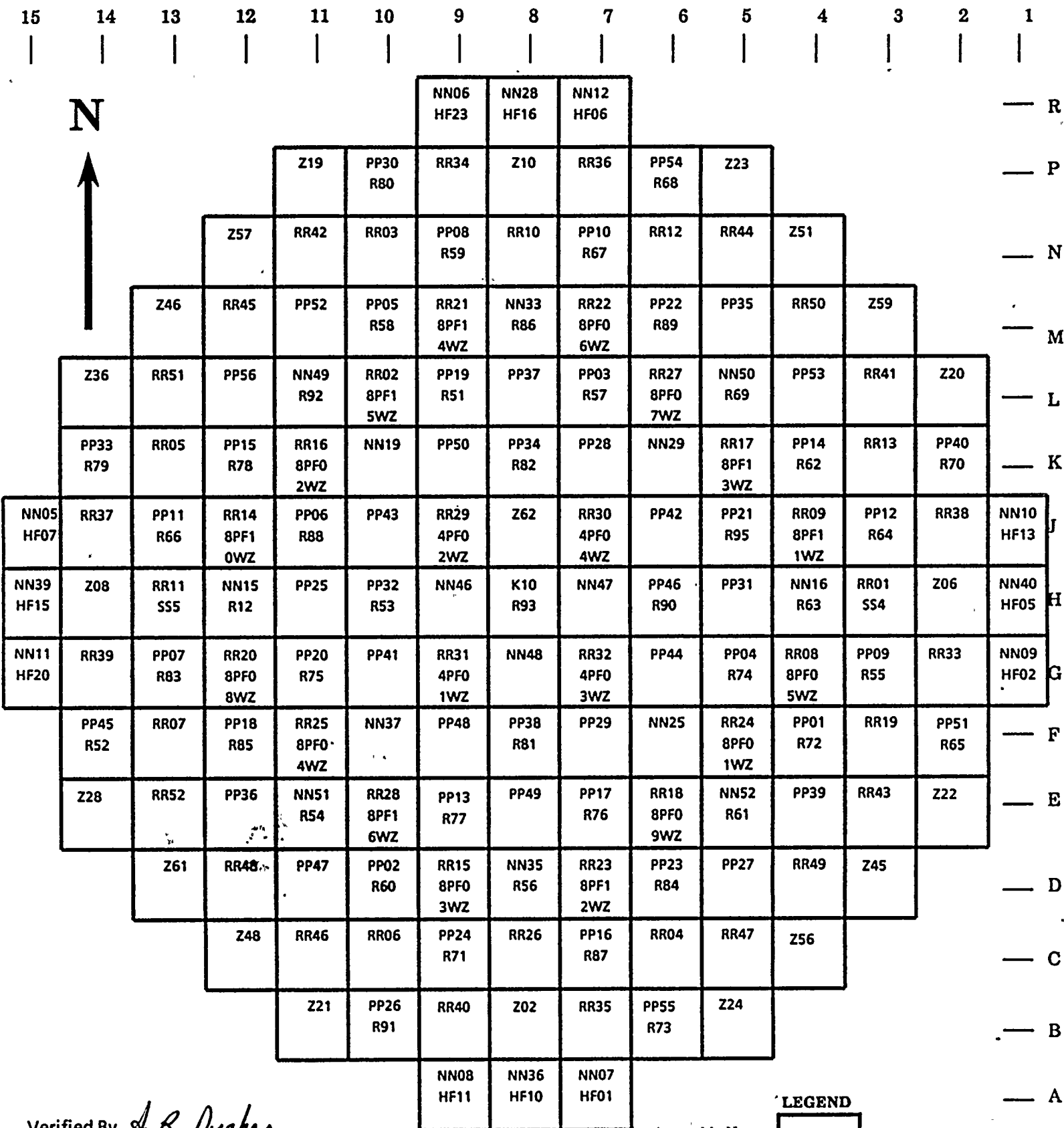
All rods met the drop time limit of 2.4 seconds as per Technical Specification 3.2.3.



REACTOR FUEL LOCATION TURKEY POINT PLANT UNIT NO. 2 CYCLE NO. XII

DATE 2-10-89

FIGURE 1



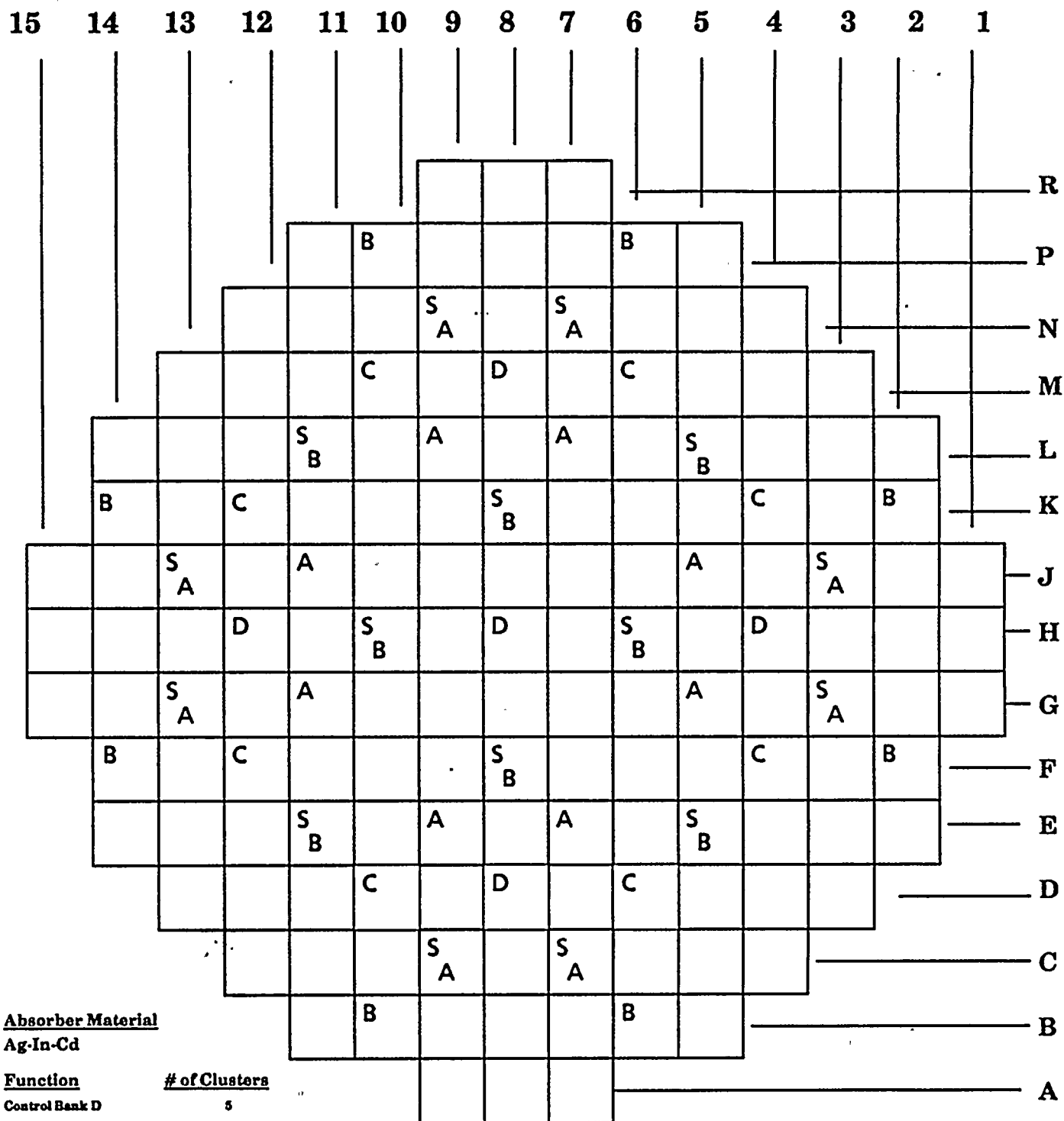
Verified By A. R. Dycker

Assembly No.

Insert No.

LEGEND

CONTROL ROD BANK LOCATION
TRIPLE POINT PLANT UNIT No. 1
CYCLE No. XII
FIGURE 2

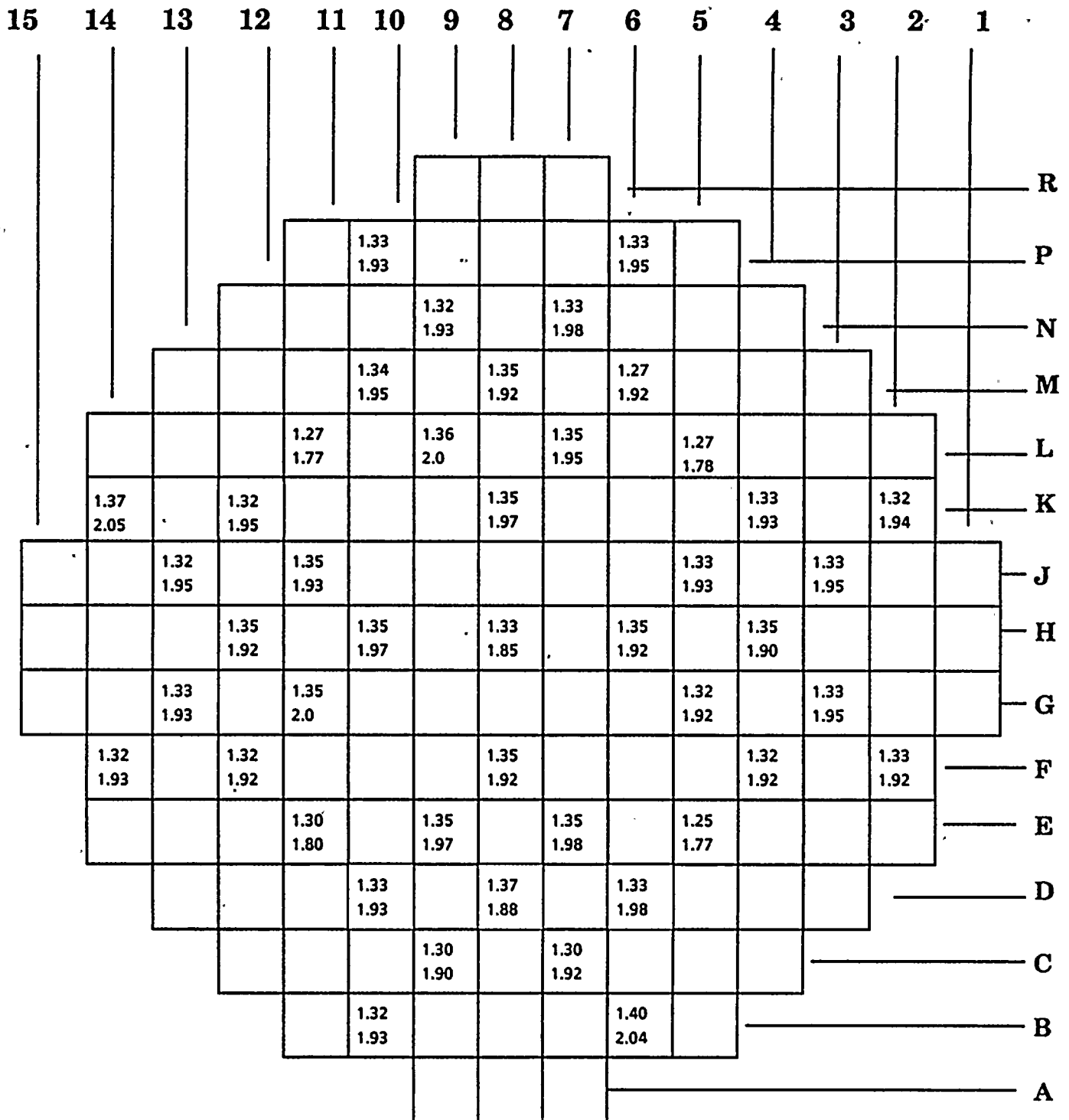


Absorber Material
Ag-In-Cd

<u>Function</u>	<u># of Clusters</u>
Control Bank D	5
Control Bank C	8
Control Bank B	8
Control Bank A	8
Control Bank SB	8
Control Bank SA	8



ROD DROP TIMES
TURKEY POINT PLANT UNIT No. 4
CYCLE No. XII
FIGURE 3



LEGEND



TIME TO DASHPOT
 TIME TO BOTTOM

2.0 INITIAL CRITICALITY

The approach to criticality began May 19, 1989, @ 0158 hours in accordance with Operating Procedure 0204.3, Initial Criticality After Refueling. Criticality was achieved May 19, 1989, @ 1550 hours by withdrawing control rods to 180 steps on Bank D and diluting the RCS with 19,400 gallons of water.

Upon attaining criticality the flux level was increased to 1×10^{-8} amps on the intermediate range to obtain critical data.

Tavg	=	547°F
Control Bank D	=	210 Steps
Boron	=	1530 ppm
Flux	=	1×10^{-8} amps

TABLE 2.1

FLUX

<u>Picoammeter</u>	<u>N-35</u>	<u>N-36</u>
1.05×10^{-8} amps	1.5×10^{-8} amps	1.6×10^{-8} amps

The following graph (Figure 4) is a plot of the ICRR during the approach to criticality.



R3:2

N31 = •

N35 = x

100

0.10 CENTIMETER
STANDARD

FIGURE 4

46151

ICRR VS. DILUTION H₂O
OP 0204.3

Unit 4 Cycle XII

Date: 5-17-89

ICRR

1.0

.9

.8

.7

.6

.5

.4

.3

.2

.1

1

2

3

4

5

6

7

8

9

10

11

12

H₂O (Gallons) X 1000



1. The first step is to identify the problem or question that needs to be answered. This involves understanding the context and the specific requirements of the task.

★ 8:1

K-E

TO THE CENTIMETER 10 x 25 CM
A 15581 CD 10410054

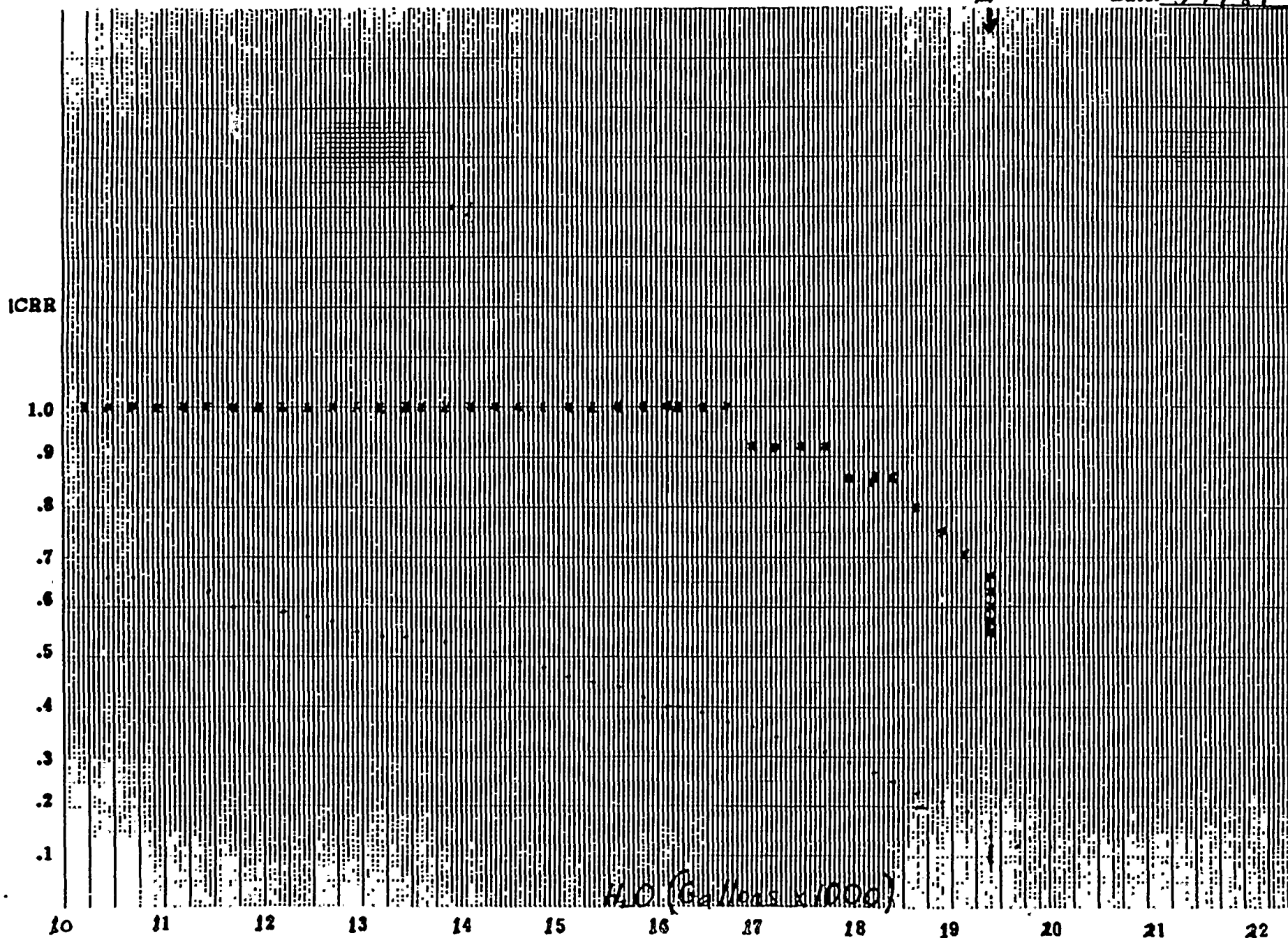
FIGURE 4

46 15.

ICRR VS. DILUTION H₂O continued
OP 0204.3

Dilution
Stopped

Unit 4 Cycle XII
Date: 5-19-89





3.0 Summary of Tests

This section provides a summary of the results of the low power physics tests along with the Westinghouse design data. This report compares design and measured data using difference¹ and percent difference². For each test, the acceptance criteria is listed at the bottom of the table.

¹The difference = predicted - measured.

²For calculating the percent difference, the equation is:

$$\left[\frac{\text{Predicted Value}}{\text{Measured Value}} - 1 \right] \times 100\%$$

3.1 Nuclear Heating

The point of adding Nuclear Heat was determined in accordance with Operating Procedure 0204.3, Initial Criticality After Refueling, Step 8.15 and Appendix A. This is performed by establishing a small positive startup rate and measuring the point (flux level) at which T_{avg} departs from its established, steady value.

Nuclear Heating was measured to first occur at:

TABLE 3.1.1
FLUX LEVEL (AMPS)

<u>Picoammeter</u>	<u>N-35</u>	<u>N-36</u>
3.17×10^{-7}	4.5×10^{-7}	4.56×10^{-7}

All physics tests were conducted at or below 1.0×10^{-7} amps on the picoammeter connected to N-44 to assure Nuclear Heating did not occur.

3.2 Reactivity Vs. Period

Reactivity Computer checkout was done in accordance with Operating Procedure 0204.3, Initial Criticality After Refueling, Step 8.17 and Appendix B. This checkout is performed by inserting small positive and negative reactivities using rod motion, measuring the period generated and the indicated worth, and then comparing design worths to measured worths for the given period.

TABLE 3.2.1

<u>Period (sec)</u>	<u>Reactivity (pcm)</u>	<u>Reactivity (design)</u>	<u>Diff (%)</u>
+123.8	+45.5	+45.8	.66
-191.5	-44.5	-47	5.62
+155.2	+37.5	+38	1.33
-266.6	-30.0	-31.2	4.0
+52.2	+86.0	+87.8	2.09

Acceptance Criteria is +/- 10.0%.

3.3 Boron Endpoints (PPM)

The Boron Endpoints noted below are determined as per Operating Procedure 0204.5, Appendix A. A just-critical condition is established as near as practicable to the required rod configuration (i.e., ARO and control Bank C in). The RCS boron concentration was determined and then adjusted analytically for the ppm worth of the reactivity (measured in pcm) by which the actual critical state deviated from the design condition. Appendix A was performed for the ARO boron endpoint and latter for the CBC in boron endpoint.

TABLE 3.3.1
BORON ENDPOINTS (PPM)

	<u>Measured</u>	<u>Westinghouse</u>	<u>Difference</u>
ARO	1538	1572	34 PPM
CBC	1399	1424	25 PPM

Acceptance Criteria is +/- 50 ppm



3.4 ROD WORTH

Rod worths were measured as per Operating Procedure 0204.5, Appendices D and F. The Reference Bank (highest predicted worth) was diluted into the core. The boron concentration prior to and subsequent to this insertion was determined and the difference in the two boron concentrations is defined as the boron (Rod) worth of the Bank (Table 3.4). The differential and integral worth of control bank C was measured and plotted (Figure 5).

TABLE 3.4

ROD WORTH (PPM)

	<u>Measured</u>	<u>Westinghouse</u>
CBC	139	148

3.5 ROD WORTH (PCM)

The remaining rod bank worths were measured using the rod swap technique, "swapping" negative reactivity insertions on the bank being measured with positive reactivity insertions from the Reference Bank.

TABLE 3.5.1

ROD WORTH (PCM)

	<u>Measured</u>	<u>Westinghouse</u>	<u>Diff (PCM)</u>	<u>% Diff</u>
CBD	691	712	21	+3.04
CBC ¹	1314	1272	-42	-3.20
CBB	375	364	-11	-2.93
CBA	1177	1182	5	+0.42
SBB	1180	1224	44	+3.73
SBA	1000	959	-41	-4.1
Total	5737	5713	-24	-.42

The acceptance criteria for rod worth measurements are:

- (1) Reference bank within +/- 10% of design, and
- (2) Individual banks within +/- 15% or +/- 100 pcm of design whichever is greater, and
- (3) Sum of all measured banks within +/- 10% of design.



3.6 TEMPERATURE COEFFICIENT

The isothermal and moderator temperature coefficients were determined using Appendix B in Operating Procedure 0204.5, Nuclear Design Check Tests During Startup After Refueling.

The values determined for this testing sequence (in pcm/°F) are:

TABLE 3.6.1
ISOTHERMAL TEMPERATURE COEFFICIENT (PCM/°F)

<u>Rods</u>	<u>Measured</u> ¹	<u>Design</u> <u>Westinghouse</u>	<u>Diff</u>
D 214/213	-.88	-1.7	-.82

Acceptance Criteria is +/- 2 pcm/°F of design.

TABLE 3.6.2
MODERATOR TEMPERATURE COEFFICIENT (PCM/°F)

<u>Rods</u>	<u>Measured</u> ¹	<u>Design</u> ² <u>Westinghouse</u>	<u>Diff</u>
D 214/213	+.92	+.1	-.82

Acceptance Criteria is $\leq + 5$ pcm/°F.

¹This is the average of one heat up and one cool down measurement.

²This value has been adjusted for boron and temperature sensitivity.

3.7. HZP DIFFERENTIAL BORON WORTH

The Hot Zero Power (HZP) Differential Boron worth was measured using Control Bank C, which had a bank worth of 1314 pcm. The value obtained for this test was:

TABLE 3.7.1

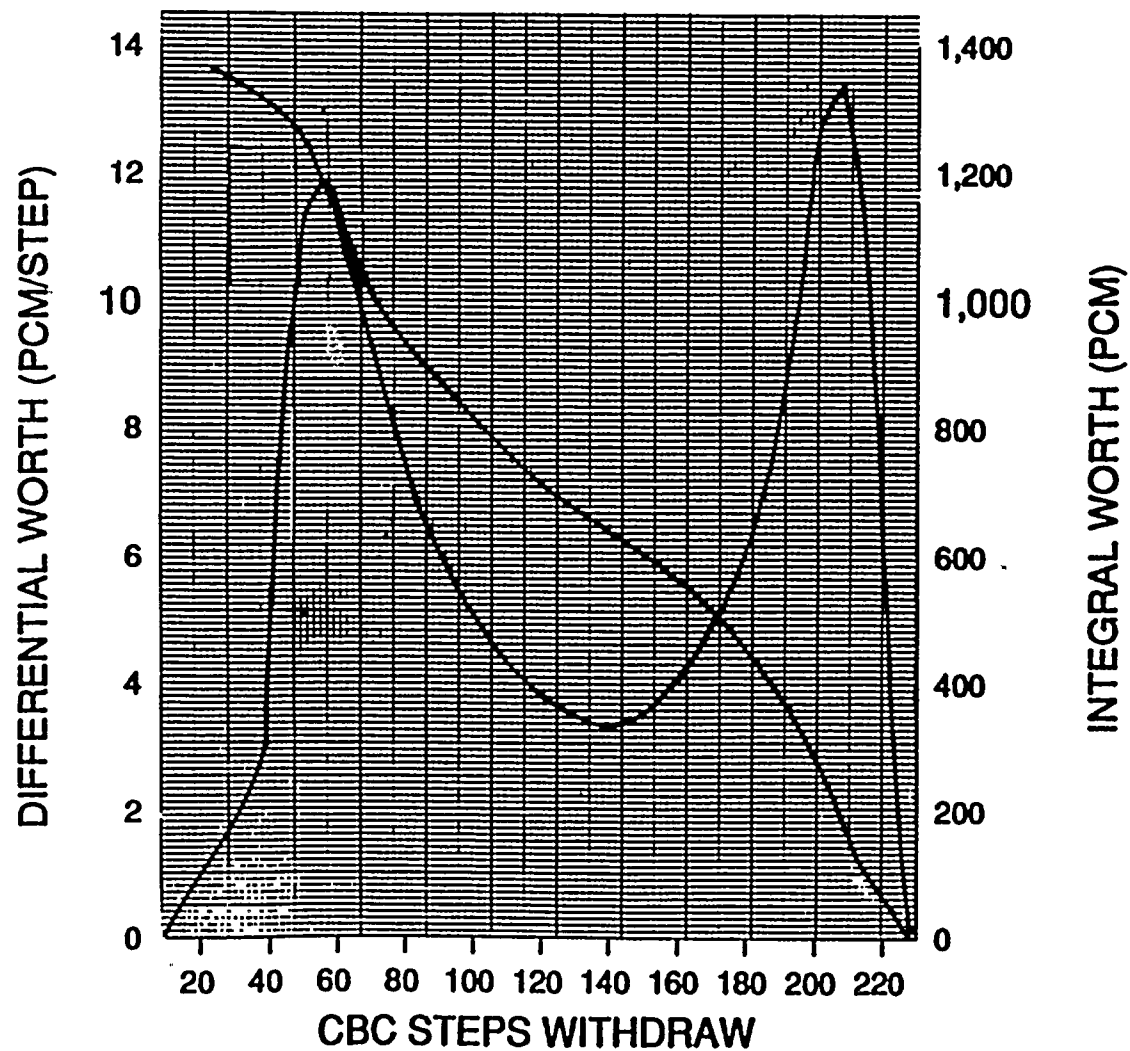
HZP DIFFERENTIAL BORON WORTH (PCM/PPM)

<u>Measured</u>	<u>Westinghouse</u>	<u>% Diff</u>
9.45	8.58	-9.2

Acceptance criteria is $\leq \pm 15\%$.

**HOT ZERO POWER
DIFFERENTIAL AND INTEGRAL CONTROL BANK C WORTH
VS
BANK POSITION**

FIGURE 5



UNIT 4
CYCLE XII
EXPOSURE 0.0 MWD/MTU
BANK CBC

BANK POSITIONS

	OUT	IN	MOVING
SBA	<input checked="" type="checkbox"/>	<input type="checkbox"/>	<input type="checkbox"/>
SBB	<input checked="" type="checkbox"/>	<input type="checkbox"/>	<input type="checkbox"/>
CBA	<input checked="" type="checkbox"/>	<input type="checkbox"/>	<input type="checkbox"/>
CBB	<input checked="" type="checkbox"/>	<input type="checkbox"/>	<input type="checkbox"/>
CBC	<input type="checkbox"/>	<input type="checkbox"/>	<input checked="" type="checkbox"/>
CBD	<input checked="" type="checkbox"/>	<input type="checkbox"/>	<input type="checkbox"/>

TEST METHOD

DILUTION	<input checked="" type="checkbox"/>
BORATION	<input type="checkbox"/>

4.0 SHUTDOWN MARGIN

The Shutdown Margin was calculated prior to power escalation to verify adequate shutdown capability. For this calculation the total of the design rod worths (minus the most reactive stuck rod) were reduced by 10%. The results show adequate shutdown margin at BOC and EOC. The following is a summary of the results:

	<u>Cycle XII</u>	
	<u>BOC</u>	<u>EOC</u>
<u>Control Rod Worth (%$\Delta\rho$)</u>		
All Rods Inserted Less Worst Stuck Rod	6.41	6.53
(1) Less 10%	5.77	5.88
<u>Control Rod Requirements (%$\Delta\rho$)</u>		
Reactivity Defects (Doppler, T_{avg} , Void, Redistribution)	1.90	3.27
Rod Insertion Allowance	1.52	.50
(2) Total Requirements	3.42	3.77
Shutdown Margin (1) - (2) % $\Delta\rho$	2.35	2.11
Required Shutdown Margin (% $\Delta\rho$)	1.00	1.77

Source: WCAP 12010

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1
1
1

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.....
10.23910.28310.2391
10.22410.27110.2241
1 5.51 5.51 5.51
.....
10.32810.64511.16110.91311.14610.66110.3461
10.33210.65311.12110.87411.12110.65310.3321
1 -1.21 -1.21 3.41 4.41 2.31 1.31 4.11
.....
10.41711.08111.26911.16611.46111.12411.36111.13910.4331
10.42211.07511.28411.13611.32111.13611.28411.07510.4221
1 -1.21 -1.21 -1.21 2.61 6.11 -1.11 1.31 4.11 2.71
.....
10.41711.14111.27011.18111.25610.98511.25911.19311.30411.17810.4211
10.42111.15211.28511.20111.25410.96611.25411.2011.28511.15210.4211
1 -0.81 -1.01 -1.21 -1.71 0.21 2.01 0.51 -0.61 1.41 2.21 0.11
.....
10.32911.00411.30011.21011.26911.14511.25911.14511.27311.21311.34611.12110.3321
10.33211.09211.28411.22411.30311.17411.25111.17411.30311.22411.28411.09210.3321
1 -0.81 -0.81 1.21 -0.51 -2.61 -2.51 0.71 -0.81 -2.31 -0.91 4.81 2.61 0.01
.....
10.67911.27611.23711.31311.08111.21611.24511.20011.06911.30011.23511.34710.6701
10.65011.28011.19711.29811.07711.23311.26311.23311.07711.29811.19711.28010.6501
1 4.41 1.31 3.31 1.21 -1.51 -1.31 -1.41 -2.71 -2.61 0.21 3.11 5.21 3.01
.....
10.25811.24111.17711.29111.17511.21711.17510.97211.13311.20711.16511.22611.11011.15610.2331
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1 14.41 11.21 4.11 3.31 0.51 -0.91 -0.91 -4.71 -4.51 -1.71 -0.31 -1.81 -1.91 3.61 3.61
.....
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.....
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.....
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10.65011.28011.19711.29811.07711.23311.26311.23311.07711.29811.19711.28010.6501
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.....
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10.33210.65311.12110.87411.12110.65310.3321
1 2.11 2.11 0.41 -0.81 -3.11 -2.61 -1.91
.....
10.22410.24810.2231 MEASURED F DELTA H
10.22410.27110.2241 EXPECTED F DELTA H
1 -0.21 -1.11 -2.41 DIFFERENCE
.....

```

R
P
N
H
L
K
J
H
G
F
E
D
C
B
A

ROD POSITION

Bank	Location in Steps	Classification
SBA	<u>228</u>	Map No. <u>FM4XII1</u>
SBB	<u>228</u>	Power % <u>25</u>
CBA	<u>228</u>	Axial Offset <u>5.22</u>
CBB	<u>228</u>	
CBC	<u>228</u>	Max F N Δ H <u>1.551</u>
CBD	<u>158</u>	Max F N Q <u>2.038</u>

INCORE TILT

N	
1.0084	1.0050
1.0100	0.9766



A

★R1:1-yrz



6.0 CRITICAL BORON CONCENTRATION

Unit 4 Cycle XII

The critical boron concentration was calculated by adjusting a measured boron concentration to the equilibrium hot full power, all rods out condition. For Unit 4 Cycle XII this calculation was performed at 900 MWD/MTU.

The following is a summary of the results in PPM:

<u>MEASURED¹</u>	<u>WESTINGHOUSE</u>	<u>DIFF</u>
1081	1068	-13

Acceptance Criteria is +/- 50 ppm

- 1) 1081 = Actual Boron concentration (adjusted to equilibrium, HFP, ARO conditions) + 34 ppm (Predicted HZP, ARO C_B - Measured HZP, ARO C_B) per ANSI Standard 19.6.1.

