



December 13, 2017

NRC 2017-0054
10 CFR 50.73

U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, DC 20555

Point Beach Nuclear Plant, Unit 1
Docket 50-266
Renewed License Nos. DPR-24

Licensee Event Report 266/2017-003-00

Enclosed is Licensee Event Report (LER) 266/2017-003-00 for Point Beach Nuclear Plant, Unit 1. NextEra Energy Point Beach, LLC is providing this LER regarding the degraded condition on Unit 1.

This letter contains no new regulatory commitments.

If you have any questions please contact Mr. Eric Schultz, Licensing Manager, at 920/755-7854.

Sincerely,

NextEra Energy Point Beach, LLC

A handwritten signature in black ink, appearing to read "Bob Coffey".

Bob Coffey
Site Vice President

Enclosure

cc: Administrator, Region III, USNRC
Project Manager, Point Beach Nuclear Plant, USNRC
Resident Inspector, Point Beach Nuclear Plant, USNRC
PSCW



LICENSEE EVENT REPORT (LER)

(See Page 2 for required number of digits/characters for each block)

(See NUREG-1022, R.3 for instruction and guidance for completing this form
<http://www.nrc.gov/reading-rm/doc-collections/nuregs/staff/sr1022/r3/>)

Estimated burden per response to comply with this mandatory collection request: 80 hours. Reported lessons learned are incorporated into the licensing process and fed back to industry. Send comments regarding burden estimate to the Information Services Branch (T-2 F43), U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, or by e-mail to Infocollects.Resource@nrc.gov, and to the Desk Officer, Office of Information and Regulatory Affairs, NEOB-10202, (3150-0104), Office of Management and Budget, Washington, DC 20503. If a means used to impose an information collection does not display a currently valid OMB control number, the NRC may not conduct or sponsor, and a person is not required to respond to, the information collection.

1. FACILITY NAME
Point Beach Nuclear Plant Unit 1

2. DOCKET NUMBER
05000266

3. PAGE
1 of 2

4. TITLE
Degraded Condition

5. EVENT DATE			6. LER NUMBER			7. REPORT DATE			8. OTHER FACILITIES INVOLVED	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REV NO.	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
10	30	2017	2017	003	00	12	13	2017	NA	NA
									NA	NA

9. OPERATING MODE MODE 3	11. THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check all that apply)			
	<input type="checkbox"/> 20.2201(b)	<input type="checkbox"/> 20.2203(a)(3)(i)	<input checked="" type="checkbox"/> 50.73(a)(2)(ii)(A)	<input type="checkbox"/> 50.73(a)(2)(viii)(A)
	<input type="checkbox"/> 20.2201(d)	<input type="checkbox"/> 20.2203(a)(3)(ii)	<input type="checkbox"/> 50.73(a)(2)(ii)(B)	<input type="checkbox"/> 50.73(a)(2)(viii)(B)
	<input type="checkbox"/> 20.2203(a)(1)	<input type="checkbox"/> 20.2203(a)(4)	<input type="checkbox"/> 50.73(a)(2)(iii)	<input type="checkbox"/> 50.73(a)(2)(ix)(A)
	<input type="checkbox"/> 20.2203(a)(2)(i)	<input type="checkbox"/> 50.36(c)(1)(i)(A)	<input type="checkbox"/> 50.73(a)(2)(iv)(A)	<input type="checkbox"/> 50.73(a)(2)(x)
10. POWER LEVEL 0%	<input type="checkbox"/> 20.2203(a)(2)(ii)	<input type="checkbox"/> 50.36(c)(1)(ii)(A)	<input type="checkbox"/> 50.73(a)(2)(v)(A)	<input type="checkbox"/> 73.71(a)(4)
	<input type="checkbox"/> 20.2203(a)(2)(iii)	<input type="checkbox"/> 50.36(c)(2)	<input type="checkbox"/> 50.73(a)(2)(v)(B)	<input type="checkbox"/> 73.71(a)(5)
	<input type="checkbox"/> 20.2203(a)(2)(iv)	<input type="checkbox"/> 50.46(a)(3)(ii)	<input type="checkbox"/> 50.73(a)(2)(v)(C)	<input type="checkbox"/> 73.77(a)(1)
	<input type="checkbox"/> 20.2203(a)(2)(v)	<input type="checkbox"/> 50.73(a)(2)(i)(A)	<input type="checkbox"/> 50.73(a)(2)(v)(D)	<input type="checkbox"/> 73.77(a)(2)(i)
	<input type="checkbox"/> 20.2203(a)(2)(vi)	<input type="checkbox"/> 50.73(a)(2)(i)(B)	<input type="checkbox"/> 50.73(a)(2)(vii)	<input type="checkbox"/> 73.77(a)(2)(ii)
		<input type="checkbox"/> 50.73(a)(2)(i)(C)	<input type="checkbox"/> OTHER	Specify in Abstract below or in NRC Form 366A

12. LICENSEE CONTACT FOR THIS LER

LICENSEE CONTACT

Thomas P. Schneider, Senior Licensing Engineer

TELEPHONE NUMBER (Include Area Code)

920-755-7797

13. COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT

CAUSE	SYSTEM	COMPONENT	MANU-FACTURER	REPORTABLE TO EPIX	CAUSE	SYSTEM	COMPONENT	MANU-FACTURER	REPORTABLE TO EPIX
B	AB	PSF	NA	Y	NA	NA	NA	NA	NA

14. SUPPLEMENTAL REPORT EXPECTED

☐ YES (If yes, complete 15. EXPECTED SUBMISSION DATE)☒ NO

15. EXPECTED SUBMISSION DATE

MONTH	DAY	YEAR
NA	NA	NA

ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines)

On October 30, 2017, with Unit 1 in MODE 3 for refueling activities, a boric acid indication downstream of 1CV-309B, 1P-1B Reactor Coolant Pump (RCP) Labyrinth Seal 1DPT-124 Upper Root Valve was identified as a through-wall flaw. The flaw location on the root valve to differential pressure transmitter (DPT) instrument tubing welded joint was within the reactor coolant system (RCS) pressure boundary.

This event is being reported pursuant to 10 CFR 50.73(a)(2)(ii)(A) for material defects in the primary coolant system that were not acceptable in accordance with ASME Section XI.

**LICENSEE EVENT REPORT (LER)
CONTINUATION SHEET**

(See NUREG-1022, R.3 for instruction and guidance for completing this form
<http://www.nrc.gov/reading-rm/doc-collections/nuregs/staff/sr1022/r3/>)

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1. FACILITY NAME		2. DOCKET NUMBER		3. LER NUMBER		
Point Beach Nuclear Plant Unit 1		05000266		YEAR	SEQUENTIAL NUMBER	REV NO.
				2017	003	00

NARRATIVE**Description of the Event:**

At 0404 CST on October 30, 2017, with Unit 1 in MODE 3 for refueling activities, a boric acid indication downstream of 1CV-309B, 1P-1B RCP Labyrinth Seal 1DPT-124 Upper Root Valve was identified as a through-wall flaw. The flaw location on the root valve [RTV] to DPT [PDT] instrument tubing welded joint [PSF] was within the RCS [AB] pressure boundary.

The root valve to DPT instrument tubing welded joint has been repaired and returned to service.

This 60 day licensee event report is being submitted in accordance with the requirements of 10 CFR 50.73(a)(2)(ii)(A) for any event or condition that results in the condition of the nuclear power plant, including its principal safety barriers, being degraded. The material defect in the primary coolant system could not be found acceptable in accordance with ASME Section XI, IWB-3600, "Analytical Evaluation of Flaws" or ASME Section XI, Table IWB-3410-1, "Acceptance Standards."

Cause of the Event:

The most likely cause of the degraded barrier was crater cracking at a weld stop point in the root valve to instrument tubing welded joint.

Analysis of the Event:

Maintenance activities identified a through-wall flaw downstream of the root valve to the RCP Labyrinth Seal DPT instrument tubing welded joint. The condition was discovered during reactor startup activities after the reactor had been shut down for several weeks for a planned refueling. The labyrinth seal serves as a buffering interface, to limit the exchange of reactor coolant from the seal portion of the RCP. The RCP Labyrinth Seal DPT instrument provides performance information only of the buffer interface. The through-wall flaw was within the RCS pressure boundary. The through-wall flaw in the welded joint was most likely the result of crater cracking at a weld stop point. The flaw has been corrected. Adequate heat removal capability for the reactor was provided by two separate engineered safety feature system trains.

Corrective Actions:

The root valve to DPT instrument tubing welded joint has been repaired and returned to service. RCS pressure boundary systems and components will continue to receive repetitive visual examinations on predetermined frequencies in accordance with existing programs.

Safety Significance:

The event was determined to be of very low safety significance. The flaw location was on small bore piping. There was no loss of any safety systems, structures or components needed to shut down the reactor, maintain safe shutdown conditions, remove residual heat, control the release of radioactive material or mitigate the consequences of an accident. There was no impact on the health and safety of the public as a result of this condition.

Similar Events:

There have not been similar events of this degraded condition in the past three years.

Component Failure Data:

None