

NRR-DMPSPeM Resource

From: Humphreys, Jerry <Jerry.Humphreys@dep.nj.gov>
Sent: Tuesday, December 12, 2017 1:05 PM
To: Regner, Lisa
Cc: Gubbi, Veena; Rosenfeld, Elliot; Timberman, Tanya J
Subject: [External_Sender] Hope Creek LAR H17-02: Relocate P-T Curves to a PTLR

On March 27, 2017, PSEG Nuclear LLC submitted License Amendment Request (LAR) H17-02 to the NRC. This LAR proposes to amend the Hope Creek (HC) Technical Specifications (TS) to revise and relocate the Pressure-Temperature (P-T) Limit Curves to a Pressure and Temperature Limits Report (PTLR).

According to the proposed PSEG LAR H17-02: The PTLR contains updates to the P-T limit curves for the Hope Creek reactor pressure vessel (RPV). The P-T curves were prepared using the methods documented in the Boiling Water Reactor Owner's Group (BWROG) Licensing Topical Report (LTR) (BWROG-TP-11-022-A, SIR-05-044), "Pressure Temperature Limits Report Methodology for Boiling Water Reactors". This BWROG LTR satisfies the requirements of 10 CFR 50 Appendix G and the American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel (B&PV) Code, Section XI, Non-mandatory Appendix G. The guidance of NRC Generic Letter (GL) 96-03, "Relocation of the Pressure Temperature Limit Curves and Low Temperature Overpressure Protection System Limits," was applied during P-T curve development. Also, Technical Specification Task Force (TSTF) Traveler TSTF-419-A, "Revise PTLR Definition and References in ISTS 5.6.6, RCS PTLR," which has received NRC approval, was followed in development of the proposed TS changes.

The NRC has approved similar license amendments to relocate the P-T limit curves to a PTLR. Recent examples for BWR plants include the Hatch, Cooper, Duane Arnold and Nine Mile Point Stations.

The New Jersey Bureau of Nuclear Engineering (NJ BNE) has reviewed LAR H17-02 and has no comments as a result of the following:

1. The P-T Curves are based on a BWROG Licensing Topical Report that meets the requirements of 10 CFR 50, Appendix G and ASME Code, Section XI, Non-mandatory Appendix G. These documents are used as part of the licensing/design bases for Hope Creek. Therefore, the licensing/design bases are being maintained.
2. The proposed TS were developed under the guidance of NRC GL 96-03 and in accordance with the NRC approved TSTF-419-A.
3. Other operating BWRs have been approved by the NRC to relocate the P-T curves to a PTLR.

If you have any questions, please feel free to contact me via the info below.

Jerry Humphreys

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From: Regner, Lisa

Sent: Monday, December 4, 2017 9:43 AM

To: Humphreys, Jerry

Cc: Regner, Lisa

Subject: Notification of Impending License Amendment Issuance for Hope Creek Nuclear Generating Station and Request for Comments

Good Morning, Jerry.

I hope this finds you well and enjoying the season.

I expect to issue a license amendment to Hope Creek by the end of the year that involves allowing it to move its Pressure-Temperature (PT) Limit Curves to a licensee-controlled document: the PT Limits Report (PTLR).

This is not a new or unusual amendment, but if you have any questions, feel free to give me a call. More details are below.

Thank you, and have a great day.

Lisa Regner

Lisa M. Regner

Senior Project Manager

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☐ OWFN 9D14

By letter dated March 27, 2017 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML17086A364), as supplemented by letters dated April 28, 2017, and September 5, 2017 (ADAMS Accession Nos. ML17118A092 and ML17248A127, respectively), PSEG Nuclear, LLC (PSEG, the licensee) submitted a license amendment request (LAR) for Hope Creek Generating Station (Hope Creek). The amendment requested changes to the Hope Creek Technical Specifications (TSs) to relocate the reactor coolant system (RCS) pressure-temperature (P-T) limits from the TSs to a new licensee-controlled document called the Pressure and Temperature Limits Report (PTLR). The LAR also proposed revisions to the 32 effective full power years (EFPY) P-T limit curves and proposed new P-T curves applicable through 44 EFPY and 56 EFPY through the end of the license renewal term. The revisions to the curves were required due to the results of a recently pulled and tested reactor pressure vessel (RPV) surveillance capsule. The proposed Hope Creek P-T limit curves to be incorporated in the PTLR were developed based on the U.S. Nuclear Regulatory Commission (NRC or the Commission)-approved methodologies described in the Boiling Water Reactor Owners' Group (BWROG) Topical Report BWROG-TP-11-022-A (SIR-05-044), Revision 1, "Pressure-Temperature Limits Report Methodology for Boiling Water Reactors," August 2013 (ADAMS Accession No. ML13277A557). This guidance will be referred to as BWROG-TP-11-022-A, Revision 1, or the BWROG PTLR methodology in this safety evaluation (SE).

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Tracking Status: None
"Regner, Lisa" <Lisa.Regner@nrc.gov>
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