

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

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 50-251 Turkey Point Plant, Unit 4, Florida Power and Light C 05000251
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 VARGA,S.A. Operating Reactors Branch 1

SUBJECT: Forwards revised Tech Spec Table 3.5-2, "ESF Actuation" &
 3.5-4, "ESF Setpoints," per 820806,830103 & 0325 submittals
 re adequacy of station electric distribution voltages.

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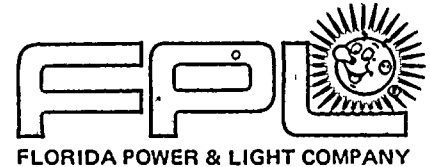
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1. The first step in the process of identifying a problem is to define the problem. This involves identifying the symptoms of the problem and determining the scope of the problem. Once the problem has been defined, the next step is to identify the causes of the problem. This involves identifying the factors that are contributing to the problem and determining the underlying causes. Once the causes have been identified, the next step is to develop a plan of action. This involves identifying the steps that need to be taken to solve the problem and determining the resources that will be needed to implement the plan. Once a plan of action has been developed, the next step is to implement the plan. This involves carrying out the steps that have been identified in the plan and monitoring the progress of the implementation. Finally, the last step in the process is to evaluate the results of the implementation. This involves determining whether the problem has been solved and whether the resources have been used effectively.

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January 31, 1984
L-84-25

Office of Nuclear Reactor Regulation
Attention: Mr. Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Dear Mr. Varga:

Re: Turkey Point Units 3 & 4
Docket Nos. 50-250 and 50-251
Distribution System Voltages

Our letters L-82-334 dated August 6, 1982, L-83-1 dated January 3, 1983, and L-83-179 dated March 25, 1982, provided technical specification changes associated with our review of the adequacy of station electric distribution voltages at Turkey Point Units 3 and 4. The purpose of this letter is to correct our proposed revisions of Table 3.5-2 (Engineered Safety Features Actuation) and Table 3.5-4 (Engineered Safety Feature Setpoints).

- 1) The proposed specification for minimum channels operable in our previous letters could severely restrict operation in that it would not allow maintenance of the undervoltage relays during operation and that cold shutdown would be required if only one of the devices became inoperable. It is our opinion that this was not the intention of the original June 2, 1977 NRC letter which requested our review of the subject.

In addition to changing the minimum channels operable from 2 to 1, a feature has been added to the relays to place an inoperable channel in trip, and allowing operation in that condition. Because of the AND logic of the circuit, this note will ensure that the safety function of the circuit will be available while the relay is being maintained or repaired.

- 2) The proposed specification for Engineered Safety Feature Setpoints in our previous letters included relay tag numbers within the Load Centers and their corresponding setpoints.

We have reconsidered the listing of the 480V relays and have determined that listing tag numbers only adds confusion to the understanding of the Technical Specifications while supplying little or no technical content.

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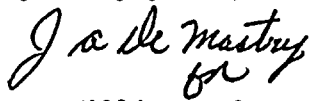
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For this reason, we are [revising Table 3.5-4 to only reflect Load Centers and specific instantaneous and delay setpoints.] It is our opinion that this revision still meets the intent of the original June 2, 1977 NRC letter which requested our review of the subject.

Corrected pages incorporating the changes discussed above are attached. Should you or your staff have any questions on this information, please contact us.

Very truly yours,



J. W. Williams, Jr.
Vice President
Nuclear Energy Department

JWW/PLP/js

Attachment

cc: Mr. James P. O'Reilly, Region II
Harold F. Reis, Esquire
PNS-LI-83-719