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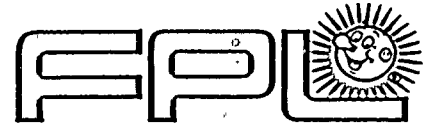
ACCESSION NBR: 8308220311 DOC. DATE: 83/08/18 NOTARIZED: NO DOCKET #
 FACIL: 50-250 Turkey Point Plant, Unit 3, Florida Power and Light Co. 05000250
 50-251 Turkey Point Plant, Unit 4, Florida Power and Light Co. 05000251
 AUTH. NAME: UHRIG, R. E. AUTHOR AFFILIATION: Florida Power & Light Co.
 RECIP. NAME: EISENHUT, D. G. RECIPIENT AFFILIATION: Division of Licensing

SUBJECT: Application to amend Licenses DPR-31 & DPR-41, revising Tech Specs re dose equivalent method of calculating radioiodine activity in primary coolant.

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FLORIDA POWER & LIGHT COMPANY
August 18, 1983

L-83-451

Office of Nuclear Reactor Regulation
Attention: Mr. Darrell G. Eisenhut, Director
Division of Licensing
U. S. Nuclear Regulatory Commission
Washington, D.C. 20555

Dear Mr. Eisenhut:

Re: Turkey Point Units 3 & 4
Docket No. 50-250 & 251
Proposed License Amendment
Maximum Reactor Coolant Activity

In accordance with 10 CFR 50.90, Florida Power & Light Company submits herewith three signed originals and forty copies of a request to amend Appendix A of Facility Operating Licenses DPR-31 and 41.

This amendment is submitted to replace the existing technical specification requirement with that of the current Standard Technical Specification, as modified by the existing Turkey Point Technical Specification format.

The dose equivalent method of calculating radioiodine activity in the primary coolant as used in the Standard Technical Specification would significantly reduce the calculated activity compared to the method currently in the Turkey Point Unit 3 and 4 Technical Specifications. The level of primary activity in Turkey Point Unit 3 is rising steadily and can be extrapolated to reach the current Technical Specification limit by the first week of September 1983, thereby forcing a plant shutdown four weeks prior to the scheduled refueling outage. We therefore, request approval of the amendment by September 1, 1983, on an Emergency basis, per 10 CFR 50.91 (a) (5).

Turkey Point Unit 3 has noted relatively high coolant activity since the beginning of Cycle 8. However, the activity levels have significantly increased since June 1983. We were informed by our fuel vendor, Westinghouse, on August 12, 1983, that the problem of fuel failure has been occurring elsewhere throughout the country. Westinghouse's presentation of fuel failure mechanisms, and the unpredictability of such failures, convinced us that relief from the Technical Specifications was required.

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This relief is required for the following reasons:

- 1) The overall trend in primary coolant activity is that of a sharp increase. This increase could be extrapolated to exceed the Technical Specification limit prior to planned unit shutdown.
- 2) The failure mechanism is not well defined, therefore unpredictable.
- 3) Any reduction in power will raise the primary coolant activity level. The next scheduled power reduction is the 30-day test of the Turbine Stop and Control Valves and Reheater Stop and Intercept Valves during the second week of September. This date includes the grace period allowed by Technical Specifications.
- 4) The current Technical Specification is much more conservative than that now approved for use by the NRC.
- 5) The current Technical Specification may force the plant to shut down four weeks earlier than scheduled.

The proposed amendment is described below and shown on the accompanying Technical Specification pages.

Page i & vi

Definitions and a new Figure are added to the tables.

Page 1-5 to 1-7

Definitions are added

Page 3.1-5, 3.1-5a & 3.1-5b

The existing requirements for maximum reactor coolant activity is replaced by those of the Standard Technical Specifications.

Figure 3.1-1

A figure of allowable dose-equivalent vs. power level is included.

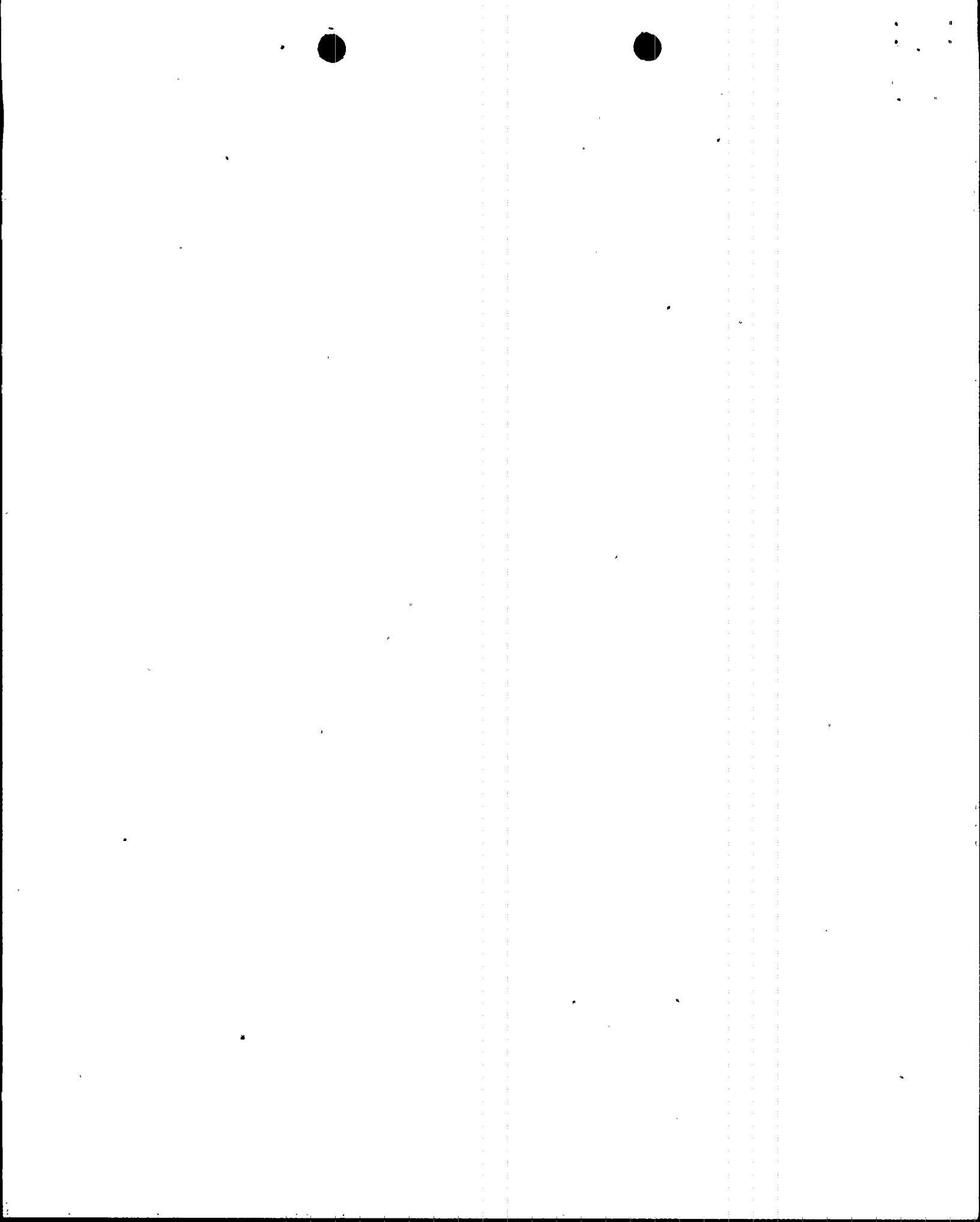
Table 4.1-2 (Sheet 1 of 3) and (Sheet 2 of 3)

Item 1-h, containing special sampling requirements for Isotopic Analyses has been added.

B3.1-5 & 3.1-6

The Bases for the maximum reactor coolant activity has been replaced with those from the Standard Technical Specifications. The entire Reactor Coolant System Bases has been resubmitted.

A Safety Evaluation and Significant Hazards Evaluation as required by 10 CFR 50.91 and 92 is in Attachment A.

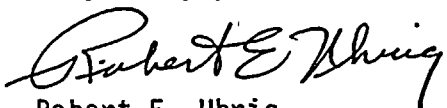


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The proposed amendment has been reviewed by the Turkey Point Plant Nuclear Safety Committee and the Florida Power & Light Company Nuclear Review Board.

In accordance with 10 CFR 170.22, it has been determined that this represents a Class I and Class III amendment and a check for payment will follow under separate cover.

Very truly yours,



Robert E. Uhrig
Vice President
Advanced Systems & Technology

REU/JEM/lmg

Attachment

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