TABLE 1.1

OPERATIONAL MODES***

Mode	Reactivity Condition, K_{eff}	% Rated Thermal Power*	Average Coolant Temperature
1. Power Operation	$\geq$0.99	> 5%	$\geq$350°F
2. Start-up	$\geq$0.99	$\leq$ 5%	$\geq$350°F
3. Hot Standby	< 0.99	0	$\geq$350°F
4. Hot Shutdown	< 0.99	0	350°F > T_{avg} > 200°F
5. Cold Shutdown	< 0.99	0	$\leq$200°F
6. Refueling**	$\leq$0.95	0	$\leq$140°F

* Excluding decay heat.

** Fuel in the reactor vessel with the vessel head closure bolts less than fully tensioned or with the head removed.

*** **This table shall only be applicable to those specifications that have been modified to reflect Operational Modes in the Applicability section of the LCOs, except as specified in Section 3.0.1 (Note).**

a) A k_{eff} of 0.95 or less, or

b) A boron concentration of greater than or equal to 1950 ppm.**

APPLICABILITY: MODE 6*

ACTION:

With the requirements of the above specification not satisfied, immediately suspend all operations involving CORE ALTERATIONS or positive reactivity changes and initiate and continue boration at greater than or equal to 45 gpm of a solution containing greater than or equal to 1950 ppm boron or its equivalent until k_{eff} is reduced to less than or equal to 0.95 or the boron concentration is restored to greater than or equal to 1950 ppm, whichever is the more restrictive.

*The reactor shall be maintained in MODE 6 whenever fuel is in the reactor vessel with the vessel head closure bolts less than fully tensioned or with the head removed.

**The boron concentration of the Reactor Coolant System and the refueling canal shall be determined by chemical analysis at least once per 72 hours.

3.10.9 CRANE TRAVEL-SPENT FUEL STORAGE AREAS

HEAVY LOADS shall be prohibited from travel over fuel assemblies in the storage pool.*

*Exception may be taken for the temporary construction crane to be used for the re-rack operation which may be carried over irradiated fuel to facilitate installation of the crane. Lift rigs which meet the design and operational requirements of NUREG 0612 "Control of Heavy Loads at Nuclear Power Plants" will be used while performing this installation.

APPLICABILITY: With fuel assemblies in the storage pool.

ACTION:

a) With the requirements of the above specification not satisfied, place the crane load in a safe condition.

b) The provisions of Specification 3.0.1 and 3.0.4 are not applicable.

TABLE 4.18-1

MINIMUM FREQUENCIES FOR SAFETY RELATED SYSTEMS FLOWPATH VERIFICATIONS

	SYSTEM DESCRIPTION (NOTE 1)	FREQUENCY	APPLICABILITY MODE
1.	High Head Safety Injection	M,P	1,2,3
2.	Low Head Safety Injection	M,P	1,2,3
3.	Auxiliary Feedwater	M,P	1,2,3 (Note 2)
4.	Containment Spray	M,P	1,2,3,4
5.	Emergency Diesel Generators	M	1,2,3,4 (Note 2)
6.	Component Cooling Water	M,P	1,2,3,4
7.	Intake Cooling Water	M,P	1,2,3,4
8.	Boric Acid Flowpath to the Core	M	1,2,3,4,5,6
9.	Post-accident Containment Ventilation	M,P	1,2,3 (Note 2)
10.	In-plant AC Electrical Distribution	M,P	1,2,3,4
11.	Post-accident Hydrogen Monitoring	M	1,2,3,4,5,6 (Note 2)
12.	Post-accident Sampling	M	1,2,3,4,5,6 (Note 2)
13.	Fire Suppression Water System	M	1,2,3,4,5,6 (Note 2)

Frequency:

M - Monthly
P - Within one surveillance interval prior to entering applicable MODE.

NOTES:

1. Refer to Bases T.S. B4.18 for definitions of systems required flowpaths.
2. These are shared systems. For this reason, with either reactor being within the applicable modes of operation, the flowpath verification shall be performed for that unit at the designated frequency.

Amendment Nos. ___ and ___

B3.10.7 RESIDUAL HEAT REMOVAL AND COOLANT CIRCULATION

The requirement that at least one residual heat removal (RHR) loop be in operation ensures that: (1) sufficient cooling capacity is available to remove decay heat and maintain the water in the reactor vessel below 140°F as required during the REFUELING MODE, and (2) sufficient coolant circulation is maintained through the core to minimize the effect of a boron dilution incident and prevent boron stratification.

The requirement to have two RHR loops operable when there is less than 23 feet of water above the reactor vessel flange ensures that a single failure of the operating RHR loop will not result in a complete loss of residual heat removal capability. With the reactor vessel head removed and at least 23 feet of water above the reactor pressure vessel flange, a large heat sink is available for core cooling. Thus, in the event of a failure of the operating RHR loop, adequate time is provided to initiate emergency procedures to cool the core.

B3.10.8 BORON CONCENTRATION

The limitations on reactivity conditions during REFUELING ensure that: (1) the reactor will remain subcritical during CORE ALTERATIONS, and (2) a boron concentration is maintained for reactivity control in the water volume having direct access to the reactor vessel. These limitations are consistent with the initial conditions assumed for the boron dilution incident in the safety analyses.

B3.10.9 CRANE TRAVEL - SPENT FUEL STORAGE AREAS

The restriction on movement of HEAVY LOADS over other fuel assemblies* in the storage pool ensures that in the event this load is dropped: (1) the activity release will be limited to that contained in a single fuel assembly, and (2) any possible distortion of fuel in the storage racks will not result in a critical array. This assumption is consistent with the activity release assumed in the safety analyses.

*Exception may be taken for the temporary construction crane to be used for the re-rack operation which may be carried over irradiated fuel to facilitate installation of the crane. Lift rigs which meet the design and operational requirements of NUREG 0612 "Control of Heavy Loads at Nuclear Power Plants" will be used while performing this installation.