REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR:8511040241 DOC.DATE: 85/10/28 NOTARIZED: NO DOCKET #
FACIL:50-250 Turkey Point Plant, Unit 3, Florida Power and Light C 05000250
50-251 Turkey Point Plant, Unit 4, Florida Power and Light C 05000251
AUTH.NAME AUTHOR AFFILIATION
WILLIAMS,J.W. Florida Power & Light Co.
RECIP.NAME RECIPIENT AFFILIATION
VARGAS,S.A. Operating Reactors Branch 1

SUBJECT: Forwards revised pages to inservice test program for pumps & valves,in response to NRC 850520 request for rev.Revs reflect test requirements for MOV-750 & MOV-751.

DISTRIBUTION CODE: A047D COPIES RECEIVED:LTR 1 ENCL 1 SIZE: 5
TITLE: OR Submittal: Inservice Inspection/Testing

NOTES: 05000250
OL:07/19/72
05000251
OL:04/14/73

	RECIPIENT ID CODE/NAME		COPIES LTTR	COPIES ENCL	RECIPIENT ID CODE/NAME		COPIES LTTR	COPIES ENCL
	NRR ORB1 BC	01	7	7				
INTERNAL:	ACRS	16	10	10	ADM/LFMB		1	0
	ELD/HDS4		1	0	NRR/DE/MEB	15	1	1
	NRR/DE/MTEB	14	1	1	NRR/DL/TAPMG		1	1
	REG FILE	04	1	1	RGN2		1	1
EXTERNAL:	24X		1	1	LPDR	03	1	1
	NRC PDR	02	1	1	NSIC	05	1	1

TOTAL NUMBER OF COPIES REQUIRED: LTTR 28 ENCL 26

P. O. BOX 14000, JUNO BEACH, FL 33408

FLORIDA POWER & LIGHT COMPANY

OCT. 28 1985

L-85-350

Office of Nuclear Reactor Regulation
Attention: Mr. S. A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Dear Mr. Varga:

Re: Turkey Point Units 3 & 4
Docket Nos. 50-250 and 50-251
Revision to the Inservice Test
Program for Pumps and Valves

In a May 20, 1985 letter (L-85-204) to you requesting a revision to the Inservice Test Program for Pumps and Valves for Turkey Point Units 3 and 4, we stated that we would submit revised program pages reflecting the test requirements for MOV-750 and MOV-751 found acceptable in your letter and safety evaluation dated May 21, 1985.

Attached are the revised program pages. They include the relief requests for deletion of Appendix J testing, a revised Table 1A (List of Containment Isolation Valves Test to Appendix J 10 CFR 50 Requirements) and a revised Table II (Reactor Coolant System Pressure Isolation Valves). These pages replace the same numbered pages in FPL's March 30, 1984 submittal.

If you have any questions, please call us.

Very truly yours,

J. W. Williams, Jr.
Group Vice President
Nuclear Energy

JWW/TCG/gp

Attachments

cc: Harold F. Reis, Esq.
Dr. J. Nelson Grace, NRC Region II

A047
1/1

TCG/009/1

RELIEF REQUEST BASIS

SYSTEM: Auxiliary Coolant, Residual Heat Removal

3. Valve: MOV-*-751 and MOV-*-750
Category: A
Class: 1

Function: Provides a flow path from the Reactor Coolant System to the Residual Heat Removal System for removal of decay heat from the reactor core.

Test Requirement: IWV-3410

Basis for Relief: These valves are provided with interlocks to prevent opening these valves when:
(1) RCS pressure is greater than 465 psig
(2) Either MOV-*-862A, MOV-*-862B, MOV-*-863A or MOV-*-863B of the associated unit is open.

Alternate Testing: These valves will be tested during cold shutdowns.

Test Requirement: IWV-3420

Basis for Relief: These valves are not required to be tested to 10 CFR Part 50, Appendix J, Type C tests. These valves will be tested as pressure isolation valves.

Alternate Testing: These valves will be tested as Reactor Coolant System pressure isolation valves using methods, procedures, and acceptance criteria in Table II and Plant Technical Specification 4.17.

GENERIC RELIEF REQUEST BASIS

SYSTEM: Various

2. Valve:

CV-*-519B	CV-*-522A	CV-*-522B	CV-*-522C
HV-*-1	HV-*-3	CV-*-4659A	POV-*-2601
POV-*-2603	CV-*-4668A	MOV-*-860A	*-10-582
MOV-*-860B	MOV-*-872		

Category: A
Class: 2

Function: These valves provide for containment isolation

Test Requirement: IWV-3423 and IWV-3424

Basis for Relief: The containment isolation valves identified above are tested by pressurizing the piping or ducting between two or more valves installed in the associated containment penetration. This will result in performing the CODE Category A valve seat leakage test in a reverse direction from that specified in in IWV-3423 on one or more of the valves installed in the associated containment penetration.

Alternate Testing: Continue to perform the CODE Category A valve seat leakage test by pressurizing the piping or ducting between two or more valves installed in the associated containment penetration.

Nothing in Section XI of the ASME Boiler and Pressure Vessel Code shall be construed as superseding the requirements of Appendix J, 10CFR50 or the Plant Technical Specifications.

TABLE IA - LIST OF CONTAINMENT ISOLATION VALVES TESTED TO APPENDIX J; 10CFR50 REQUIREMENTS

Containment Penetration Number	Service	Valve Number(s)	Leakage Rate CC/Min
5	Gas Analyzer Sample PRT	CV-*-516 SV-*-6385	1 000 1 000
6	Nitrogen Supply to PRT	*-518 *-519	2 500 2 500
7	PW Supply to PRT and RCP Standpipes	CV-*-519A CV-*-519B CV-*-522A CV-*-522B CV-*-522C	2 000 2 000 2 000 2 000 2 000
8	Sample PRZ Steam Space	CV-*-951 CV-*-956A	1 000 1 000
9	Sample PRZ Liquid Space	CV-*-953 CV-*-956B	1 000 1 000
10	Vent and N_2 Supply for RCDT	PCV-*-1014 CV-*-4658B CV-*-4658A	2 000 2 000 2 000
11	Alt. Low Head Safety Injection	MOV-*-872	8 000
14	Letdown from Reactor Coolant System	CV-*-200A CV-*-200B CV-*-200C CV-*-204	3 000 3 000 3 000 3 000
15	Charging to Reactor Coolant System	HCV-*-121 *-333 *-312C	2 000 2 000 4 000

TABLE II

REACTOR COOLANT SYSTEM PRESSURE ISOLATION VALVES

SYSTEM	VALVE NO. Unit 3	VALVE NO. Unit 4	Maximum[a] [b] Allowable Leagage-gpm
High-Head Safety Injection			
Loop A, hot leg	3-874A	4-874A	5.0
cold leg	3-875A	4-875A	5.0
cold leg	3-873A	4-873A	5.0
Loop B, hot leg	3-874B	4-874B	5.0
cold leg	3-875B	4-875B	5.0
cold leg	3-873B	4-873B	5.0
Loop C, cold leg	3-875C	4-875C	5.0
cold leg	3-873C	4-873C	5.0
Residual Heat Removal			
Loop A, cold leg	3-876A	4-876A	5.0
		4-876E	5.0
Loop B, cold leg	3-876B	4-876B	5.0
	3-876D	4-876D	5.0
Loop C, cold leg	3-876C	4-876C	5.0
	3-876E		5.0
Loop A, hot leg		MOV-4-750	5.0
		MOV-4-751	5.0
Loop C, hot leg	MOV-3-750		5.0
	MOV-3-751		5.0