NRC FORM 366
(7-77)

U. S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT

CONTROL BLOCK: (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

01 LICENSEE CODE: F L T P S 3 (2) LICENSE NUMBER: 0 0 - 0 0 0 0 0 - 0 0 0 (3) LICENSE TYPE: 4 1 1 1 1 (4) CAT (5)

CON'T 01 REPORT SOURCE: X (6) DOCKET NUMBER: 0 5 0 0 0 2 5 0 (7) EVENT DATE: 1 1 0 1 7 9 (8) REPORT DATE: 1 1 1 5 7 9 (9)

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

02 The NSSS vendor has identified and corrected an input error in each of two
03 LOCA analyses specifically applicable to Turkey Point Units 3 and 4. The
04 LOCA analyses for the 22% and the 25% steam generator tube plugging levels
05 were affected. Correction of the error in the 22% analysis resulted in a
06 reduction of the maximum allowable F_q from 2.10 to 1.90. Both units have
07 less than 22% steam generator tube plugging.
08

09 SYSTEM CODE: Z Z (11) CAUSE CODE: X (12) CAUSE SUBCODE: Z (13) COMPONENT CODE: Z Z Z Z Z Z (14) COMP. SUBCODE: Z (15) VALVE SUBCODE: Z (16)

(17) LER/RO REPORT NUMBER: EVENT YEAR: 7 9 — SEQUENTIAL REPORT NO.: 0 3 3 / OCCURRENCE CODE: 0 1 REPORT TYPE: T — REVISION NO.: 0

ACTION TAKEN: X (18) FUTURE ACTION: X (19) EFFECT ON PLANT: Z (20) SHUTDOWN METHOD: Z (21) HOURS (22): 0 0 0 0 ATTACHMENT SUBMITTED: Y (23) NPRD-4 FORM SUB.: N (24) PRIME COMP. SUPPLIER: Z (25) COMPONENT MANUFACTURER: Z 9 9 9 (26)

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

10 Both units are now operating with a revised F_q limit, and a T.S. change
11 proposal will be formally submitted. Root cause and additional corrective
12 action are discussed in the attachment.
13
14

15 FACILITY STATUS: E (28) % POWER: 1 0 0 (29) OTHER STATUS (30): NA METHOD OF DISCOVERY: D (31) DISCOVERY DESCRIPTION (32): NSSS notification

16 ACTIVITY RELEASED: Z (33) CONTENT OF RELEASE: Z (34) AMOUNT OF ACTIVITY (35): NA LOCATION OF RELEASE (36): NA

17 PERSONNEL EXPOSURES NUMBER: 0 0 0 (37) TYPE: Z (38) DESCRIPTION (39): NA

18 PERSONNEL INJURIES NUMBER: 0 0 0 (40) DESCRIPTION (41): NA

19 LOSS OF OR DAMAGE TO FACILITY (43) TYPE: Z (42) DESCRIPTION: NA

20 PUBLICITY ISSUED: N (44) DESCRIPTION (45): NA

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Additional Cause Description and Corrective Actions:

There are two methods which utilize pressure drops for input to SATAN computer code calculations. One is a standard technique and the other is used for steam generator tube plugging analyses when only the tube plugging input is changed. Since the 22% analysis involved an updated hot assembly flow area input to SATAN (unrelated to tube plugging), the first method should have been used. However, since the 22% analysis was a "tube plugging" analysis, the second method was inadvertently used.

With respect to the 25% analysis, a value for power in the hot assembly that was input to the SATAN calculation was 3% too low.

In order to assure that a similar error will not occur in the future, the following revisions to the NSSS vendor's procedures are being implemented.

a. Automated cross checks of current input data with those of a previous analysis (or analysis for a similar plant) will be required (while it is now only recommended).

b. Safeguards Engineering Standards dealing with SATAN pressure drops will be revised to include a cautionary statement about the impact of data changes on the methods used.

c. A standard for steam generator tube plugging analysis will be written.