

LICENSEE EVENT REPORT

CONTROL BLOCK: 1 (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

01 F L T P S 4 2 0 0 - 0 0 0 0 - 0 0 3 4 1 1 1 1 4 5

LICENSEE CODE 14 15 LICENSE NUMBER 25 26 LICENSE TYPE 30 31 CAT 36

CON'T

01 X 6 0 5 10 0 0 0 2 5 1 7 1 1 2 6 8 0 3 1 2 1 0 8 0 9

REPORT SOURCE 60 61 DOCKET NUMBER 68 69 EVENT DATE 74 75 REPORT DATE 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES 10

02 Notification was received that the inspection/evaluation in progress in

03 accordance with USNRC I&E Bulletin 79-14 had revealed a deficiency

04 involving portions of accumulator and residual heat removal system piping

05 inside containment. This condition would potentially exist when safe

06 shutdown earthquake loads are superimposed on all other design basis

07 loads.

09

09 S F 11 B 12 A 13 S U P O R T 14 B 15 Z 16

SYSTEM CODE 9 10 CAUSE CODE 11 CAUSE SUBCODE 12 COMPONENT CODE 13 COMP. SUBCODE 19 VALVE SUBCODE 20

17 8 0 0 1 4 0 1 T 0

LER/RO REPORT NUMBER 21 22 EVENT YEAR 23 24 SEQUENTIAL REPORT NO. 25 26 OCCURRENCE CODE 27 28 REPORT TYPE 29 REVISION NO. 32

ACTION TAKEN 33 FUTURE ACTION 34 EFFECT ON PLANT 35 SHUTDOWN METHOD 36 HOURS 37 ATTACHMENT SUBMITTED 40 NPRO-4 FORM SUB. 42 PRIME COMP. SUPPLIER 43 COMPONENT MANUFACTURE 44

F 13 Z 19 Z 20 Z 21 0 0 0 0 Y 23 N 24 A 25 X 9 9

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS 27

10 Evaluation revealed that the calculated maximum stress exceeded the

11 acceptance criteria (established for this review) on the above piping.

12 The unit is currently in refueling shutdown. A plant change/modification

13 will be implemented to support the piping before the unit is returned to

14 service.

15 H 28 0 0 0 29 NA C 31 I&E Bulletin 79-14 Inspection

FACILITY STATUS 3 9 % POWER 10 12 OTHER STATUS 20 METHOD OF DISCOVERY 44 DISCOVERY DESCRIPTION 32

15 Z 33 Z 34 NA NA

ACTIVITY 3 9 CONTENT 10 12 AMOUNT OF ACTIVITY 35 LOCATION OF RELEASE 36

15 Z 33 Z 34 NA NA

PERSONNEL EXPOSURES 10 11 12

17 0 0 0 37 Z 38 NA

NUMBER 11 TYPE 12 DESCRIPTION 39

PERSONNEL INJURIES 10 11 12

13 0 0 0 40 NA

NUMBER 11 DESCRIPTION 41

LOSS OF OR DAMAGE TO FACILITY 10 12

19 Z 42 NA

TYPE 11 DESCRIPTION 43

PUBLICITY 10

20 N 44 NA

ISSUED DESCRIPTION 45

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Additional Event Description and Probable Consequences:

On Wednesday, November 26, 1980, notification was received from our Architect-Engineer that inspections and reanalyses in progress in accordance with USNRC I&E Bulletin 79-14 had revealed a potential problem involving inadequate support of the Safety Injection System piping from the accumulator and interconnected Residual Heat Removal piping to the Reactor Coolant System loops inside containment. Specifically, the results indicated that the calculated maximum stress associated with the piping exceeded the acceptance criteria established for this review. USNRC I&E Region II was notified of the above information both verbally and by facsimile transmission.

Similar occurrences (relating to the inspections pursuant to USNRC I&E Bulletin 79-14) were reported as LER 250-79-26, LER 250-79-40, LER 250-80-08, and LER 250-80-14.

Additional Cause Description and Corrective Actions:

The evaluation confirmed that an overstress condition could potentially exist. Based on these results, a plant change/modification was designed and will be implemented prior to heatup following the current refueling shutdown.

A full accounting of inspection results and repairs will be made in response to USNRC I&E Bulletin 79-14.