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 FACIL:50-250 Turkey Point Plant, Unit 3, Florida Power and Light C 05000250  
 50-251 Turkey Point Plant, Unit 4, Florida Power and Light C 05000251  
 AUTH.NAME AUTHOR AFFILIATION  
 UHRIG,R.E. Florida Power & Light Co.  
 RECIP.NAME RECIPIENT AFFILIATION  
 EISENHUT,D,G. Division of Licensing

SUBJECT: Advises that Westinghouse submittal of WCAP-9748 &  
 WCAP-9749 completes Phase C asymmetric LOCA loads analysis.  
 WCAP-9558 & WCAP-9570 provide addl info requested at 800226  
 meeting.

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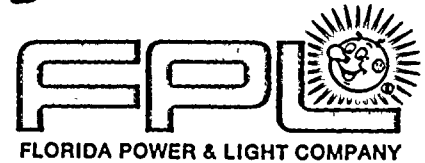
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July 7, 1980  
L-80-213

Office of Nuclear Reactor Regulation  
Attention: Mr. Darrell G. Eisenhut, Director  
Division of Licensing  
U. S. Nuclear Regulatory Commission  
Washington, D.C. 20555

Dear Mr. Eisenhut:

RE: Turkey Point Units 3 & 4  
Docket Nos. 50-250 and 50-251  
Asymmetric LOCA Loads Analysis

In our letter of February 15, 1980 (L-80-58) we provided, as members of the Westinghouse Owners Group, an interim report which contained an initial assessment of the ability of nuclear steam supply system components to accommodate asymmetric LOCA loads.

Westinghouse, on behalf of the Owners Group, has subsequently submitted to the NRC Staff the following technical reports:

- (1) WCAP-9558 Revision 1 (proprietary), "Mechanistic Fracture Evaluation of Reactor Coolant Pipe Containing a Postulated Circumferential Through-Wall Crack", June 1980.
- (2) WCAP-9570 Revision 1 (non-proprietary), "Mechanistic Fracture Evaluation of Reactor Coolant Pipe Containing a Postulated Circumferential Through-Wall Crack", June 1980.
- (3) WCAP-9748 (proprietary), "Westinghouse Owners Group Asymmetric LOCA Loads Evaluation - Phase C", June 1980.
- (4) WCAP-9749 (non-proprietary) "Westinghouse Owners Group Asymmetric LOCA Loads Evaluation - Phase C", June 1980.

These reports are applicable to Turkey Point Units 3 & 4 and should be considered as part of our response to the NRC's January 25, 1978 information request.

The submittal of topical reports WCAP-9748 and WCAP-9749 completes our planned Phase C Asymmetric LOCA Loads analysis for pipe breaks at the reactor nozzles. The revisions to the mechanistic pipe break topical reports, WCAP-9558 and WCAP-9570, address the staff request for additional information discussed at a meeting between the Owners Group, Westinghouse, and the NRC on February

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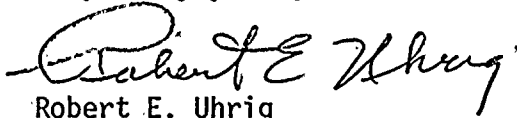
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26, 1980. The single open item, not included herein, concerns the extension of the mechanistic pipe break study to address weld metal. A weld metal test program is currently being performed with results expected in August 1980.

Very truly yours,



Robert E. Uhrig  
Vice President  
Advanced Systems & Technology

REU/MAS/PKG/pa

cc: J. P. O'Reilly, Region II  
Harold F. Reis, Esquire

