



March 28, 1980
L-80-108

Office of Nuclear Reactor Regulation
Attention: Mr. Darrell G. Eisenhut
Acting Director
Division of Operating Reactors
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Dear Mr. Eisenhut:

Re: Turkey Point Units 3 and 4
Docket Nos. 50-250 and 50-251
Steam Generator Repair Report

Attached herewith is Revision 7 to Florida Power & Light Company's Steam Generator Repair Report (SGRR) which was forwarded to you on September 20, 1977 (L-77-296).

The major changes included in Revision 7 are summarized below.

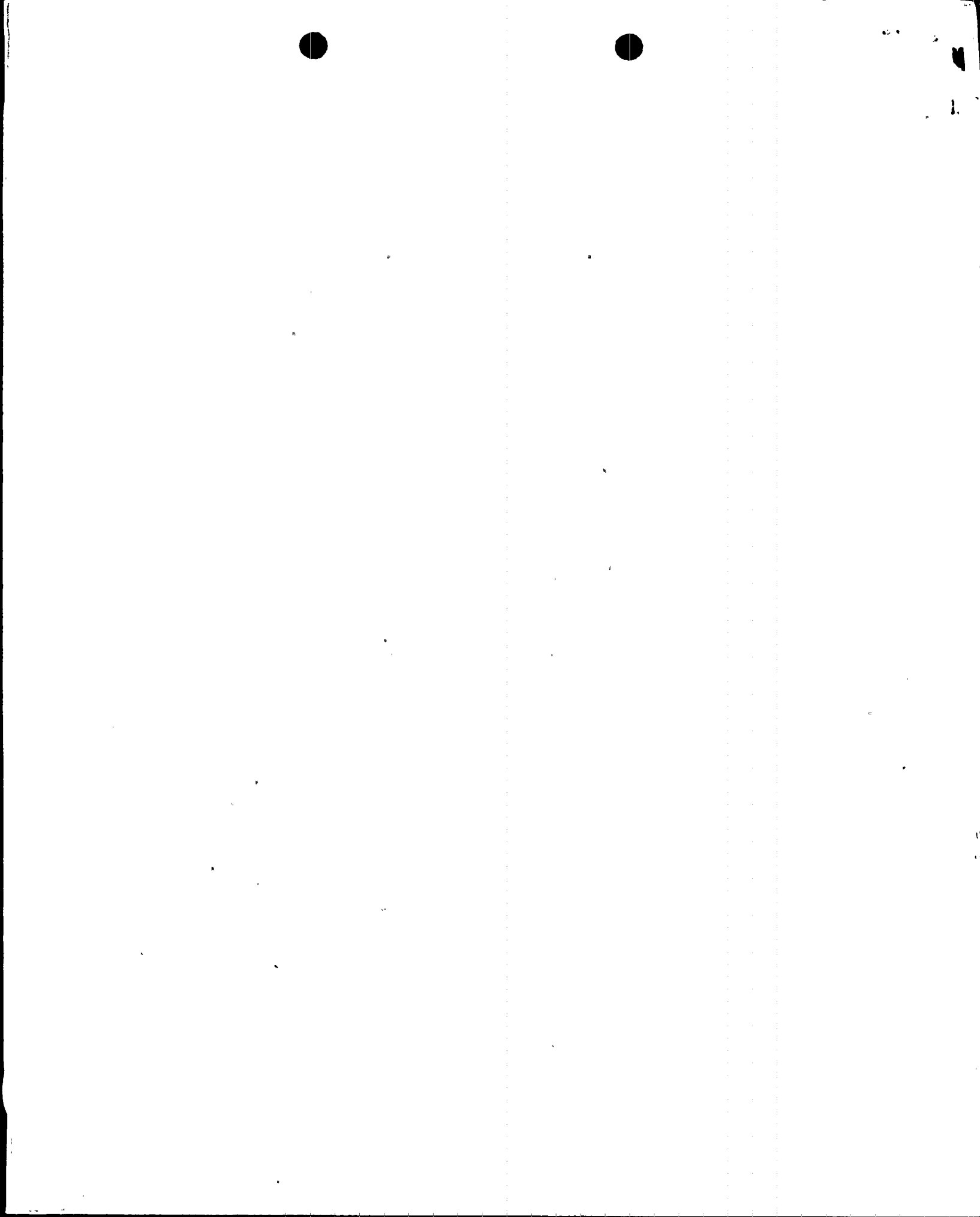
(1) Lower Assembly Removal

The SGRR, through Revision 6, has described the repair as involving removal of the upper assembly by cutting the steam generator in the transition cone area and removing the lower assembly by cutting the reactor coolant piping. This method is referred to as the pipe cut approach. However, due to recent Surry experience, we have re-evaluated the methodology and man-rem exposures associated with the pipe cut approach and have determined that an alternate approach would be more desirable. This revision revises the SGRR to reflect this new approach. It consists of removing the upper assembly, as before, by cutting in the transition cone area. Removal of the lower assembly would be accomplished by cutting the lower assembly near the junction of the steam generator channel head to the tubesheet. This alternative is referred to as the channel cut approach. This approach is expected to result in a significant savings of man-rem over the pipe cut approach.

(2) Dose Calculations

As part of our re-evaluations, in addition to revising man-rem dose estimates, we have also revised off-site

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dose calculations to reflect the non-uniform deposition of corrosion products on the various internal surfaces of the steam generators. Previous release calculations has assumed uniform deposition. The relative concentration factors used in these revised calculations are shown in Table 6.2-1.

We have also updated the SGRR to reflect actual 1978-1979 releases and exposure data. This data is used throughout the report for comparison purposes with the releases and exposures expected from the repair. Previously, 1976 data had been used.

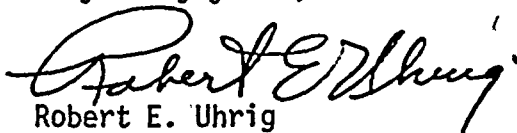
(3) Steam Generator Disposal

As part of our on-going evaluations, we have now determined that disposing of the old steam generators by shipping them by barge to a land burial site may now be a viable alternative. Barge shipment had previously not been considered viable because of the lack of barge-slip facilities at the Barnwell, S. C. disposal site. We now feel that the appropriate facilities can be made available at Barnwell. Chapters 3 and 8 have been revised to reflect this.

(4) Return To Service Testing

Chapter 4 of the SGRR has been revised to include additional details on the return to service testing which will be done to verify the integrity of the reactor coolant system and the containment building. This information is in response to the Atomic Safety and Licensing Board order dated October 11, 1979.

Very truly yours,



Robert E. Uhrig
Vice President
Advanced Systems & Technology

REU/GDW/ah

Attachment

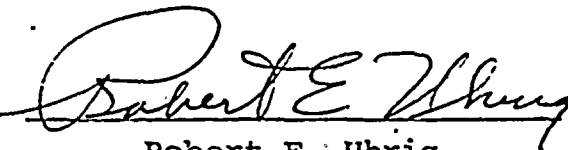
cc: J. P. O'Reilly, Region II
Harold F. Reis, Esquire

STATE OF FLORIDA)
)
COUNTY OF DADE) SS.

Robert E. Uhrig, being first duly sworn, deposes and says:

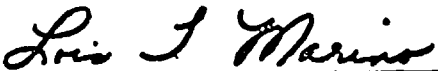
That he is a Vice President of Florida Power & Light Company,
the Licensee herein;

That he has executed the foregoing document; that the state-
ments made in this said document are true and correct to the
best of his knowledge, information, and belief, and that he
is authorized to execute the document on behalf of said
Licensee.


Robert E. Uhrig

Subscribed and sworn to before me this

28th day of March, 1980



NOTARY PUBLIC, in and for the county of Dade,
State of Florida

My commission expires: _____

NOTARY PUBLIC STATE OF FLORIDA at LARGE
MY COMMISSION EXPIRES AUGUST 24, 1981
BONDED THRU MAYNARD BONDING AGENCY

