

LICENSEE EVENT REPORT (LER)

(See reverse for required number of
digits/characters for each block)ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS MANDATORY
INFORMATION COLLECTION REQUEST: 50.0 HRS. REPORTED LESSONS LEARNED ARE
INCORPORATED INTO THE LICENSING PROCESS AND FED BACK TO INDUSTRY.
FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND
RECORDS MANAGEMENT BRANCH (T-6 F33), U.S. NUCLEAR REGULATORY
COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION
PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC
20503

FACILITY NAME (1)

Cook Nuclear Plant Unit 1

DOCKET NUMBER (2)

05000-315

PAGE (3)

1 of 4

TITLE (4)

Turbine Driven Auxiliary Feedwater Pump Speed Controller Failure Mode Not Considered

EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER	
11	28	1998	1998	--	052	--	00	12	28	1998	DC Cook Unit 2 05000-316
OPERATING MODE (9)		5	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more) (11)								
POWER LEVEL (10)		000	20.2201 (b)		20.2203(a)(2)(v)		50.73(a)(2)(i)		50.73(a)(2)(viii)		
			20.2203(a)(1)		20.2203(a)(3)(i)		X 50.73(a)(2)(ii)		50.73(a)(2)(x)		
			20.2203(a)(2)(i)		20.2203(a)(3)(ii)		50.73(a)(2)(iii)		73.71		
			20.2203(a)(2)(ii)		20.2203(a)(4)		50.73(a)(2)(iv)		OTHER		
			20.2203(a)(2)(iii)		50.36(c)(1)		50.73(a)(2)(v)		Specify in Abstract below or on NRC Form 366A		
			20.2203(a)(2)(iv)		50.36(c)(2)		50.73(a)(2)(vii)				

LICENSEE CONTACT FOR THIS LER (12)

NAME

Mr. Joel Gebbie, Safety Related Mechanical Engineering Supervisor

TELEPHONE NUMBER (Include Area Code)

616/465-5901 x1543

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO EPIX	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO EPIX

SUPPLEMENTAL REPORT EXPECTED (14)

X	YES (If Yes, complete EXPECTED SUBMISSION DATE).		NO	EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
					02	17	1999

Abstract (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16)

During a Safety System Functional Inspection (SSFI) self-assessment of the Auxiliary Feedwater (AFW) System, no analysis for a failure of the non-safety related remote speed controller manual loader for the Turbine Driven Auxiliary Feedwater Pumps (TDAFP) could be found. Engineering analysis of the condition resulted in the conclusion that a failure of the non-safety related manual loader output could result in the TDAFP operating at a minimum speed of 1900 rpm with only one Motor Driven Auxiliary Feedwater Pump (MDAFP) in operation when Auxiliary Feedwater (AFW) is required to mitigate the consequences of an accident. On November 28, 1998, this was determined to be reportable. Therefore, this LER is being submitted in accordance with 10 CFR 50.73 (a)(2)(ii)(B) as a condition outside the design basis of the plant.

To prevent recurrence of this issue in the future, a training course will be provided to the Plant Engineering staff to ensure that the design requirements for non-safety related systems that interface with safety related systems are met. Potential modifications to the Auxiliary Feedwater System are being evaluated as an additional corrective action. When the course of action is decided, this LER will be updated to reflect that information.

The safety significance of this condition was evaluated considering that the AFW system would be required to satisfy the accident analyses assumptions with the TDAFP operating at 1900 rpm in addition to a single active failure of a MDAFP. Under these conditions, the AFW system would not be able to produce AFW flow rates assumed in the accident analyses. However, the capability exists to cross-tie the AFW systems between units and provide an alternate source of AFW to the affected unit. Therefore, although this condition would result in the AFW system being outside the design bases, it would not pose a threat to the health and safety of the public.

**LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION**

FACILITY NAME (1)	DOCKET NUMBER(2)	LER NUMBER (6)				PAGE (3)
		YEAR	SEQUENTIAL NUMBER		REVISION NUMBER	
		1998	--	052	-- 00	

TEXT (If more space is required, use additional copies of NRC Form (366A) (17))

Conditions Prior to Event

Unit 1 was in Mode 5, Cold Shutdown

Unit 2 was in Mode 5, Cold Shutdown

Description of Event

On September 21, 1998, a Safety System Function Inspection (SSFI) self-assessment of the Auxiliary Feedwater (AFW) system began, using SSFI techniques in accordance with NRC Inspection Procedure 93801, "Safety System Functional Inspection". The inspection team utilized a vertical slice review in the functional areas of engineering design and configuration control, operations, maintenance, surveillance and testing, and quality assurance and corrective actions. The self-assessment concluded on October 23, 1998.

During the SSFI, it was identified that no analysis for a failure of the non-safety related remote speed controller manual loader for the Turbine Driven Auxiliary Feedwater Pumps (TDAFP) existed. Engineering analysis of the condition resulted in the conclusion that a failure of the non-safety related manual loader output could result in the TDAFP operating at a minimum speed of 1900 rpm with only one Motor Driven Auxiliary Feedwater Pump (MDAFP) in operation when Auxiliary Feedwater (AFW) is required. On November 28, 1998, this was determined to be reportable in accordance with 10 CFR 50.73 (a)(2)(ii)(B) as a condition outside the design basis of the plant.

The steam supply to the Turbine Driven Auxiliary Feedwater Pump (TDAFP) is regulated by a Woodward governor valve. Depending upon the position of the governor valve, the speed of the TDAFP can vary from 1900 rpm to 4350 rpm. The governor valve is a pneumatically controlled device. The position of the governor valve may be remotely adjusted from the Control Room by using the speed control manual loader. The pneumatic input to the speed control manual loader is supplied from the 20 psig compressed air header. The loader output can vary from 3 psig to 15 psig. Loader output of 3 psig corresponds to maximum pump speed of 4350 rpm, while loader output of 15 psig corresponds to minimum pump speed of 1900 rpm.

Upon a loss of control air pressure to the loader, either due to loss of the compressed air system or a failure of the speed control manual loader, the TDAFP speed will automatically go to its design speed of 4350 rpm. This condition has been previously evaluated.

However, if the speed control manual loader fails such that the TDAFP governor valve receives the compressed air header pressure of 20 psig, the TDAFP speed will automatically go to its minimum speed setting of 1900 rpm. No evaluation of the failure mode where the TDAFP governor receives full compressed air system pressure of 20 psig had been performed.

The TDAFP speed control manual loader is a non-safety related component. As such, failure of the loader cannot be considered as the single active/passive failure of the AFW system. Therefore, the AFW system would have to satisfy the accident analyses assumptions with the TDAFP operating at 1900 rpm in addition to the single active failure of a MDAFP. Under these conditions, the AFW system would not be able to produce AFW flow rates assumed in the accident analyses.

Cause of Event

The root cause of this condition is that not all failure modes were considered when the compressed air system was originally designed. The plant was designed in accordance with General Design Criterion 26, "Protection Systems Fail-Safe Design." The design criterion specifically refers to loss of air as a failure mode; however, it does not specify over-pressurization.

Review of components installed in the plant has revealed that a number of the components which utilize controllers are also designed with a dump solenoid. The dump solenoid ensures that the component (e.g., valve) will fail to its safe position. If a component was credited with an active safety function, it was fitted with a dump solenoid. However, if the

**LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION**

FACILITY NAME (1)	DOCKET NUMBER(2)	LER NUMBER (6)				PAGE (3)
		YEAR	SEQUENTIAL NUMBER		REVISION NUMBER	3 of 4
		1998	--	052	--	

Cook Nuclear Plant Unit 1

05000-315

3 of 4

TEXT (If more space is required, use additional copies of NRC Form (366A) (17))

component did not have an active safety function (such as the TDAFP speed controller manual loader), it was not designed with a dump solenoid.

Analysis of Event

This LER is submitted in accordance with 10 CFR 50.73(a)(2)(ii)(B) as a condition outside the design bases of the plant.

The AFW system provides water to the steam generators when main feedwater is unavailable because of a loss of main feedwater, unit trip, feedwater or steam line break, loss of off-site power, or small break Loss of Coolant Accident (SBLOCA). This water removes core residual heat to prevent the release of primary water through the pressurizer safety or power-operated relief valves and allows the plant to cool down to the point at which Residual Heat Removal (RHR) can be placed in service.

Each unit is equipped with one turbine driven AFW pump and two motor driven AFW pumps. For each unit, the TDAFP serves all four steam generators and each MDAFW pump serves two steam generators. The steam to the AFW pump turbine is supplied from two of the steam generators.

The preferred source of water for the AFW system is the non-safety related CST. Each unit's CST is cross-tied by a normally closed air operated valve to provide condensate to the opposite unit's AFW. If both CSTs are unavailable, water is supplied from Lake Michigan via the safety related ESW system, which is connected upstream of the AFW pump suction strainers. A minimum of 175,000 gallons is required to maintain the unit at hot shutdown for nine hours.

The TDAFP speed control manual loader is a non-safety related component. As such, failure of the loader cannot be considered as the single active/passive failure of the AFW system. Therefore, the AFW system would be required to satisfy the accident analyses assumptions with the TDAFP operating at 1900 rpm in addition to the single active failure of a MDAFP. Under these conditions, the AFW system would not be able to produce AFW flow rates assumed in the accident analyses. However, the capability exists to cross-tie the AFW systems between units. Procedures are in place and operators are trained to perform this evolution. Should this condition occur and since Cook is not designed for simultaneous design basis accidents in both units, supplemental AFW could be provided from the opposite unit. Therefore, although this condition would result in the AFW system being outside the design bases, it would not pose a threat to the health and safety of the public.

Corrective Actions

To ensure that this condition does not affect other components in the plant, a review has been performed to identify those components that utilize a similar controller. Several components with similar controllers have been identified. Those components are the centrifugal charging pump discharge flow control valve (QRV-251) and the steam generator PORVs (MRV-213, 223, 233,243). Components identified during this search will be evaluated for all failure modes and appropriate corrective actions will be implemented as necessary.

This condition occurred because the effect of non-safety related components on the operation of safety related systems was not properly evaluated. To prevent recurrence of this condition, a training course will be provided to the Plant Engineering staff to ensure that the design requirements for non-safety related systems, structures or components (SSCs) that interface with safety related SSCs are met. This training will be completed prior to restart.

The current design change process has been verified to address requirements to design against single failures. The current procedure provides a significant improvement in the design change process over the process that was in place at the time of the original design for DC Cook

**LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION**

FACILITY NAME (1)	DOCKET NUMBER(2)	LER NUMBER (6)				PAGE (3)
		YEAR	SEQUENTIAL NUMBER		REVISION NUMBER	
		1998	—	052	— 00	
Cook Nuclear Plant Unit 1	05000-315					4 of 4

TEXT (If more space is required, use additional copies of NRC Form (366A) (17))

Potential modifications to the Auxiliary Feedwater System are being evaluated as an additional corrective action. When the course of action is decided, this LER will be updated to reflect that information.

Previous Similar Events

315/97-023-00

315/97-026-00