

CATEGORY 1

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 FACIL: 50-315 Donald C. Cook Nuclear Power Plant, Unit 1, Indiana M 05000315
 AUTH. NAME: AUTHOR AFFILIATION
 BOESCH, J. Indiana Michigan Power Co.
 SAMPSON, J.R. Indiana Michigan Power Co.
 RECIP. NAME RECIPIENT AFFILIATION

SUBJECT: LER 98-051-00: on 981122, reactor trip signal from manual
 safety injection not verified as required by TS
 surveillance, was discovered. Maintenance currently evaluating
 significance & cause of event.

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Power Company
Cook Nuclear Plant
One Cook Plant
Bridgman, MI 49106
616 465 5901



December 22, 1998

United States Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Operating License DPR-58
Docket No. 50-315

Document Control Manager:

In accordance with the criteria established by 10 CFR 50.73 entitled Licensee Event Report System, the following interim report is being submitted:

LER 315/98-051-00

No commitments were identified in this submittal.

Sincerely,


J. R. Sampson
Site Vice President

/mbd
Attachment

c: J. L. Caldwell (Acting), Region III
R. P. Powers
P. A. Barrett
J. B. Kingseed
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LICENSEE EVENT REPORT (LER)

(See reverse for required number of
digits/characters for each block)ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS MANDATORY
INFORMATION COLLECTION REQUEST: 50.0 HRS. REPORTED LESSONS LEARNED ARE
INCORPORATED INTO THE LICENSING PROCESS AND FED BACK TO INDUSTRY.
FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND
RECORDS MANAGEMENT BRANCH (T-6 F33), U.S. NUCLEAR REGULATORY
COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION
PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC
20503

FACILITY NAME (1) Cook Nuclear Plant Unit 1										DOCKET NUMBER (2) 05000-315		PAGE (3) 1 of 1		
TITLE (4) Reactor Trip Signal from Manual Safety Injection Not Verified as Required by Technical Specification Surveillance														
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)				
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME Cook Unit 2		DOCKET NUMBER 05000-316			
11	22	1998	1998	-- 051 --	00	12	22	1998	FACILITY NAME		DOCKET NUMBER			
OPERATING MODE (9)		5		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more) (11)										
POWER LEVEL (10)		00		20.2201 (b)		20.2203(a)(2)(v)		<input checked="" type="checkbox"/>		50.73(a)(2)(i)		50.73(a)(2)(viii)		
				20.2203(a)(1)		20.2203(a)(3)(i)				50.73(a)(2)(ii)		50.73(a)(2)(x)		
				20.2203(a)(2)(i)		20.2203(a)(3)(ii)				50.73(a)(2)(iii)		73.71		
				20.2203(a)(2)(ii)		20.2203(a)(4)				50.73(a)(2)(iv)		OTHER		
				20.2203(a)(2)(iii)		50.36(c)(1)				50.73(a)(2)(v)		Specify in Abstract below or on NRC Form 366A		
				20.2203(a)(2)(iv)		50.36(c)(2)				50.73(a)(2)(vii)				
LICENSEE CONTACT FOR THIS LER (12)														
NAME Mr. John Boesch, Maintenance Manager										TELEPHONE NUMBER (Include Area Code) (616) 465-5901, x2634				
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)														
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO EPIX		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO EPIX				
SUPPLEMENTAL REPORT EXPECTED (14)										EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR
<input checked="" type="checkbox"/>	YES (If Yes, complete EXPECTED SUBMISSION DATE).					<input type="checkbox"/>	NO				01	12	1999	
Abstract (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16)														
<p>On November 22, 1998, during a review of Technical Specification (TS) surveillance requirements for the engineered safety feature actuation system instrumentation by engineering, it was discovered that in Modes 3 and 4 the reactor trip signal from a manual initiation of a safety injection (SI) had not been verified in accordance with TS 4.3.2.1.1, Table 4.3-2, Item 9.a. The TS requires the SI tests be performed for Modes 1, 2, 3 and 4 on a refueling frequency. Normally during preparations for unit restart from refueling, the reactor trip breakers are required to be open in Modes 3 and 4 for other TS surveillance testing. The tripping of the reactor trip breakers from manual SI initiation cannot be verified with the reactor trip breakers open. Therefore the reactor trip portion of the manual SI test has not normally been performed for Modes 3 and 4. In accordance with 10CFR50.73(a)(2)(i)(B), this interim LER is being reported as a condition prohibited by the plant's TS.</p> <p>Maintenance is currently evaluating the significance and cause of this event. Corrective actions will be developed based on the results of the evaluation. It is anticipated that an update to this LER will be submitted by January 12, 1999.</p>														