

CATEGORY 1

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR: 9812110024 DOC. DATE: 98/12/07 NOTARIZED: NO DOCKET #
 FACIL: 50-316 Donald C. Cook Nuclear Power Plant, Unit 2, Indiana M 05000316
 AUTH. NAME AUTHOR AFFILIATION
 BRASSART, G. Indiana Michigan Power Co.
 SAMPSON, J. R. Indiana Michigan Power Co.
 RECIP. NAME RECIPIENT AFFILIATION

SUBJECT: LER 98-007-00: on 981106, high energy line break effects in auxiliary FW sys was noted. Cause of event is under investigation & will be completed by 990220. Updated LER will be submitted by 990310. With 981207 ltr.

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Indiana Michigan
Power Company
Cook Nuclear Plant
One Cook Plant
Bridgman, MI 49106
616 465 5501



December 7, 1998

United States Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Operating License DPR-74
Docket No. 50-316

Document Control Manager:

In accordance with the criteria established by 10 CFR 50.73 entitled Licensee Event Report System (LER), the following interim report is being submitted:

LER 316/98-007-00, "High Energy Line Break Effects On Auxiliary Feedwater System."

No commitments have been identified in the attached document.

Sincerely,

A handwritten signature in cursive script, appearing to read "John R. Sampson".

J. R. Sampson
Site Vice President

/mbd

Attachment

c: J. L. Caldwell (Acting), Region III
R. P. Powers
P. A. Barrett
J. B. Kingseed
R. Whale
D. Hahn
Records Center, INPO
NRC Resident Inspector

9812110024 981207
PDR ADOCK 05000316
S PDR

NRC Form 366 (6-1998)				U.S. NUCLEAR REGULATORY COMMISSION LICENSEE EVENT REPORT (LER) (See reverse for required number of digits/characters for each block)				APPROVED BY OMB NO. 3150-0104 EXPIRES 06/30/2001 <small>ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS MANDATORY INFORMATION COLLECTION REQUEST: 50.0 HRS. REPORTED LESSONS LEARNED ARE INCORPORATED INTO THE LICENSING PROCESS AND FED BACK TO INDUSTRY. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (T-6 F33), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20535-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503</small>			
FACILITY NAME (1)				DOCKET NUMBER (2)		PAGE (3)					
Cook Nuclear Plant Unit 2				05000-316		1 of 1					
TITLE (4)											
Interim LER - High Energy Line Break Effects On Auxiliary Feedwater System											
EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)		OTHER FACILITIES INVOLVED (8)			
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME		
11	06	1998	1998	-- 007 --	00	12	07	1998	DOCKET NUMBER		
OPERATING MODE (9)			THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more) (11)								
1			20.2201 (b)		20.2203(a)(2)(v)		50.73(a)(2)(i)		50.73(a)(2)(viii)		
POWER LEVEL (10)			20.2203(a)(1)		20.2203(a)(3)(i)		X 50.73(a)(2)(ii)		50.73(a)(2)(x)		
100			20.2203(a)(2)(i)		20.2203(a)(3)(ii)		50.73(a)(2)(iii)		73.71		
			20.2203(a)(2)(ii)		20.2203(a)(4)		50.73(a)(2)(iv)		OTHER		
			20.2203(a)(2)(iii)		50.36(c)(1)		50.73(a)(2)(v)		Specify in Abstract below or on NRC Form 366A		
			20.2203(a)(2)(iv)		50.36(c)(2)		50.73(a)(2)(vii)				
LICENSEE CONTACT FOR THIS LER (12)											
NAME						TELEPHONE NUMBER (Include Area Code)					
Mr. Gary Brassart, Nuclear Safety Analysis Supervisor						616/697-5106					
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)											
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO EPIX		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO EPIX	
SUPPLEMENTAL REPORT EXPECTED (14)						EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR	
X	YES (If Yes, complete EXPECTED SUBMISSION DATE).				NO			03	10	1999	
Abstract (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16) <p>In July 1995, the fire door/High Energy Line Break (HELB) door for the startup blowdown flash tank room was blocked open for approximately 39 hours to allow draining of the Essential Service Water (ESW) header. At the time, the condition was evaluated and it was concluded that the condition was acceptable for up to 72 hours. This conclusion was based upon the assumption that the HELB would be detected and isolated within 10 minutes and that the line break would not result in automatic actions.</p> <p>As part of the recent Auxiliary Feedwater (AFW) System Safety System Functional Inspection, the assumption that the break would be isolated within 10 minutes was revisited and determined not to be valid. A subsequent review of Westinghouse WCAP 10961 revealed that automatic action, a reactor trip, would occur at 12 minutes into the event.</p> <p>Based on the new information, on November 6, 1998 it was determined that a HELB in the startup blowdown flash tank room would potentially expose the motor control centers (MCC) in the area to a steam environment and higher temperatures than they are qualified to withstand. Auxiliary Feedwater would potentially be lost because the valves which supply ESW to the AFW pumps are located on these MCCs. It was determined that this condition represented a condition outside the design bases, reportable under 10 CFR 50.73 (a)(2)(ii)(B).</p> <p>Investigation into this condition and potential effects of the increased temperature and steam environment on required equipment is ongoing, and is expected to be completed by February 20, 1999. An update to this interim LER will be submitted by March 10, 1999.</p>											