

CATEGORY 1

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 FACIL: 50-315 Donald C. Cook Nuclear Power Plant, Unit 1, Indiana M 05000315
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 RECIP. NAME RECIPIENT AFFILIATION

SUBJECT: LER 98-014-01: on 980310, unanalyzed condition was noted.
 Caused by procedure not being consistent w/analysis record.
 Containment pressure analysis will be performed to support
 future operation. W/980831 ltr.

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 TITLE: 50.73/50.9 Licensee Event Report (LER), Incident Rpt, etc.

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Indiana Michigan
Power Company
Cook Nuclear Plant
One Cook Plant
Bridgman, MI 49106
616 465 5501



August 31, 1998

United States Nuclear Regulatory Commission
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Operating License DPR-58
Docket No. 50-315

Document Control Manager:

In accordance with the criteria established by 10 CFR 50.73 entitled Licensee Event Report System, the following report is being submitted:

98-014-01

Sincerely,

A handwritten signature in cursive script that reads "John R. Sampson".

J. R. Sampson
Site Vice President

/mbd

Attachment

c: J. L. Caldwell (Acting), Region III
R. P. Powers
P. A. Barrett
J. B. Kingseed
R. Whale
D. Hahn
Records Center, INPO
NRC Resident Inspector

Te 22

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PDR ADDCK 05000315
S PDR



LICENSEE EVENT REPORT (LER)

(See reverse for required number of
digits/characters for each block)ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS MANDATORY
INFORMATION COLLECTION REQUEST: 50.0 HRS. REPORTED LESSONS LEARNED
ARE INCORPORATED INTO THE LICENSING PROCESS AND FED BACK TO
INDUSTRY. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE
INFORMATION AND RECORDS MANAGEMENT BRANCH (T-4 F33), U.S. NUCLEAR
REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE
PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND
BUDGET, WASHINGTON, DC 20503

FACILITY NAME (1) Cook Nuclear Plant Unit 1										DOCKET NUMBER (2) 50-315		PAGE (3) 1 of 1			
TITLE (4) Unanalyzed Condition Results from Procedure Not Being Consistent with Analysis of Record															
EVENT DATE (5)			LER NUMBER (6)					REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)				
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME Cook - Unit 2		DOCKET NUMBER 50-316				
03	10	98	98	--	014	--	01	08	31	98	FACILITY NAME DOCKET NUMBER				
OPERATING MODE (9)		5		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more) (11)											
POWER LEVEL (10)		00		20.2201 (b)					20.2203(a)(2)(v)			50.73(a)(2)(i)		50.73(a)(2)(viii)	
				20.2203(a)(1)					20.2203(a)(3)(i)			<input checked="" type="checkbox"/> 50.73(a)(2)(ii)		50.73(a)(2)(x)	
				20.2203(a)(2)(i)					20.2203(a)(3)(ii)			50.73(a)(2)(iii)		73.71	
				20.2203(a)(2)(ii)					20.2203(a)(4)			50.73(a)(2)(iv)		OTHER	
				20.2203(a)(2)(iii)					50.36(c)(1)			50.73(a)(2)(v)		Specify in Abstract below or in NRC Form 366A	
				20.2203(a)(2)(iv)					50.36(c)(2)			50.73(a)(2)(vii)			
LICENSEE CONTACT FOR THIS LER (12)															
NAME Mr. Jeb Kingseed, Nuclear Safety and analysis Manager										TELEPHONE NUMBER (Include Area Code) 616/697-5106					
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)															
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS					
SUPPLEMENTAL REPORT EXPECTED (14)										EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR	
<input checked="" type="checkbox"/>	YES (If Yes, complete EXPECTED SUBMISSION DATE).					<input type="checkbox"/>	NO					11	16	98	
Abstract (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16) On March 10, 1998, with Units 1 and 2 in Mode 5, it was determined that both units had operated in an unanalyzed condition due to Functional Restoration Procedure FRZ-1, "Response to High-High Containment Pressure", not being consistent with the containment integrity analysis of record. In accordance with 10CFR50.72(b)(2)(i), "[a]ny event found while the reactor is shutdown, that, had it been found while the reactor was in operation, would have resulted in the nuclear plant, including it [principle] safety barriers, being in an unanalyzed condition that significantly compromises plant safety", an ENS notification was made. RHR spray is used to supplement containment spray to control containment pressure. The procedure directs that actuation of RHR spray be manually initiated at a containment pressure of 8 psig. Procedure development assumed that operator implementation of this step could be performed without an appreciable increase in containment pressure. Review of the procedure against design basis accident assumptions indicated that this is not the case. As a result, current procedures may be non-conservative with respect to this analysis. Had the procedure been implemented in its current form, a potential may have existed for the post-LOCA containment pressure to exceed its design basis limit of 12 psig. Therefore, this issue is being reported under 10CFR50.73(a)(2)(ii)(A) as an unanalyzed condition. Preliminary evaluation indicates that peak containment pressure would be maintained below the design basis limit of 12 psig when operator and equipment response times are included. A containment pressure analysis will be performed to support future operation. The results of that analysis will assist in determining corrective and preventive actions for this condition. An updated LER will be issued by November 16, 1998.															