

ATTACHMENT 3 TO AEP:NRC:1295

PROPOSED REVISED TECHNICAL SPECIFICATION PAGES

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3/4 LIMITING CONDITIONS FOR OPERATION AND SURVEILLANCE REQUIREMENTS  
3/4.4 REACTOR COOLANT SYSTEM

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SURVEILLANCE REQUIREMENTS (continued)

Note: In all inspections, previously degraded tubes or sleeves must exhibit significant (greater than or equal to 10%) further wall penetrations to be included in the above percentage calculations.

4.4.5.3 Inspection Frequencies - The above required inservice inspections of steam generator tubes shall be performed at the following frequencies: †

- a. The first inservice inspection shall be performed after 6 Effective Full Power Months but within 24 calendar months of initial criticality. Subsequent inservice inspections shall be performed at intervals of not less than 12 nor more than 24 calendar months after the previous inspection. If two consecutive inspections following service under AVT conditions, not including the preservice inspection, result in all inspection results falling into the C-1 category or if two consecutive inspections demonstrate that previously observed degradation has not continued and no additional degradation has occurred, the inspection interval may be extended to a maximum of once per 40 months.
- b. If the results of inservice inspection of a steam generator conducted in accordance with Table 4.4-2 at 40 month intervals fall in Category C-3, the inspection frequency shall be increased to at least once per 20 months. The increase in inspection frequency shall apply until the subsequent inspections satisfy the criteria of Specification 4.4.5.3.a; the interval may then be extended to a maximum of once per 40 months.
- c. Additional, unscheduled inservice inspections shall be performed on each steam generator in accordance with the first sample inspection specified in Table 4.4-2 during the shutdown subsequent to any of the following conditions:
  1. Primary-to-secondary tubes leaks (not including leaks originating from tube-to-tube sheet welds) in excess of the limits of Specification 3.4.6.2.
  2. A seismic occurrence greater than the Operating Basis Earthquake.
  3. A loss-of-coolant accident requiring actuation of the engineered safeguards.
  4. A main steam line or feedwater line break.

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† Amendment \_\_\_\_\_ granted an extension to this surveillance until the start of cycle 17. In the event the steam generators are replaced at that time, the retired steam generators are exempted from further surveillance under this specification.



ATTACHMENT 4 TO AEP:NRC:1295

EXISTING TECHNICAL SPECIFICATION PAGES  
MARKED TO INDICATE PROPOSED CHANGES



3/4 LIMITING CONDITIONS FOR OPERATION AND SURVEILLANCE REQUIREMENTS  
3/4.4 REACTOR COOLANT SYSTEM

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SURVEILLANCE REQUIREMENTS (continued)

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  3. A loss-of-coolant accident requiring actuation of the engineered safeguards.
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