

American Electric Power
Cook Nuclear Plant
One Cook Place
Bridgman, MI 49106
616 465 5901



August 14, 1998

United States Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Operating License DPR-74
Docket No. 50-316

Document Control Manager:

In accordance with the criteria established by 10 CFR 50.73 entitled Licensee Event Report System (LER), the following report is being submitted:

98-005-00

Sincerely,

A handwritten signature in dark ink, appearing to read 'J. R. Sampson', is written over a horizontal line.

J. R. Sampson
Site Vice President

/mbd

Attachment

c: J. L. Caldwell (Acting), Region III
B. P. Powers
P. A. Barrett
J. B. Kingseed
R. Whale
D. Hahn
Records Center, INPO
NRC Resident Inspector

9808170100 JPP
TDR

Enclosure 3

LICENSEE EVENT REPORT (LER)

(See reverse for required number of
digits/characters for each block)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS MANDATORY INFORMATION COLLECTION REQUEST: 50.0 HRS. REPORTED LESSONS LEARNED ARE INCORPORATED INTO THE LICENSING PROCESS AND FED BACK TO INDUSTRY. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (T-6 F33), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503

FACILITY NAME (1)

Cook Nuclear Plant Unit 2

DOCKET NUMBER (2)

50-316

PAGE (3)

1 of 1

TITLE (4)

Interim LER - Potential for High Energy Line Break to Degrade Component Cooling Water System

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
07	15	98	98	005	00	08	14	98	Donald C. Cook - Unit 1	50-315
OPERATING MODE (9)			THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more) (11)							
5			20.2201 (b)			20.2203(a)(2)(v)			50.73(a)(2)(i)	
POWER LEVEL (10)			20.2203(a)(1)			20.2203(a)(3)(i)			X 50.73(a)(2)(ii)	
0			20.2203(a)(2)(i)			20.2203(a)(3)(ii)			50.73(a)(2)(iii)	
			20.2203(a)(2)(ii)			20.2203(a)(4)			50.73(a)(2)(iv)	
			20.2203(a)(2)(iii)			50.36(c)(1)			50.73(a)(2)(v)	
			20.2203(a)(2)(iv)			50.36(c)(2)			50.73(a)(2)(vii)	
									OTHER	
									Specify in Abstract below or in NRC Form 366A	

LICENSEE CONTACT FOR THIS LER (12)

NAME

Mr. Pat Mangan - Mechanical Design Manager

TELEPHONE NUMBER (Include Area Code)

616/697-5198

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS

SUPPLEMENTAL REPORT EXPECTED (14)

X	YES (If Yes, complete EXPECTED SUBMISSION DATE).	NO
---	---	----

EXPECTED
SUBMISSION
DATE (15)

MONTH	DAY	YEAR
10	30	98

Abstract (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16)

On July 15, 1998, with Unit 1 and Unit 2 in Cold Shutdown, it was determined that the potential existed for a postulated critical crack in a Unit 2 main steam line to degrade the ability of adjacent Component Cooling Water (CCW) pumps to perform their design function. The CCW pumps for both units are adjacent to one another in a semi-enclosed area in the Auxiliary Building. Adjacent to the pumps is a pipe chase enclosing two Unit 2 main steam lines and a main feedwater line, which can be accessed through any of 3 doors. Although the pipe chase walls provide a qualified High Energy Line Break (HELB) barrier, no calculation could be found which shows that the doors would withstand the energy released from a postulated critical crack. As the adjacent CCW pump motors and other equipment are not qualified for a high temperature/high humidity environment, this was determined to constitute an unanalyzed condition. An ENS notification was made on July 15 at 1521 hours EDT in accordance with 10CFR50.72(b)(2)(i). This interim LER is therefore submitted under 10CFR50.73(a)(2)(ii)(B) for an unanalyzed condition.

The investigation and analysis of the significance of the event is continuing. Based on the results of the investigation and analysis, a method may be developed to protect the CCW pump area from the effects of the postulated crack. It is anticipated that an update to this LER will be submitted by October 30, 1998.

R.P. Powers

- 2 -

September 27, 1999

Please contact me at 301-415-1345 if you have any questions regarding this request. This request is covered by the existing OMB clearance number (3150-0104) for NRC staff followup review of events documented in LERs. Your response to this request is voluntary and does not constitute a licensing requirement.

Sincerely,

Original signed by:

John F. Stang, Sr. Project Manager, Section 1
Project Directorate III
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket Nos. 50-315 and 50-316

Enclosures: As stated

cc w/encls: See next page

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September 27, 1999

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Docket Nos. 50-315 and 50-316

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