

CALCULATION ANALYSIS
Nuclear Engineering Department

HEP&T

SECTION

SHEET 1 OF 3

I.D. NO. <u>HXP900904JEW</u>	PLANT <u>Cook Nuclear</u> UNIT <u>12</u>
SAFETY RELATED YES <u>✓</u> NO <u> </u> SYSTEM <u>Residual Heat Removal</u> (RHR)	COMPANY <u>Indiana Michigan Power</u>
TITLE <u>NPSH CALCULATION</u>	CALCULATED BY: <u>John E. Wagner</u> 9/11/90 DATE
FILE LOCATION <u>41.2</u>	CHECKED BY: <u>John J. Rikala</u> 10/30/90 DATE
MICROFILM NO. <u> </u>	APPROVED BY: <u>J. J. Rikala</u> 10/30/90 DATE

PROBLEM DESCRIPTION: Determine if there is adequate
NPSH WITH the RHR Pumps taking suction from the
Reactor Coolant System (RES) hot leg at saturated conditions.

DESIGN BASIS OR REFERENCES: Westinghouse Owners Group
(WOG) Letter OG-90-44 dated August 2, 1990.

METHOD OF VERIFICATION: Reviewed input & method

REVISIONS

NO.	REASON FOR CHANGE	PREP'D BY	DATE	CKD. BY	DATE	APVD. BY	DATE

METHOD OF VERIFICATION:

NEF64A.DOC
C:\FORM,D-E,FF-363

9808100104 980817
PDR ADDOCK 05000315
H PDR

COOK NUCLEAR PLANT	
NED RECORD-NED COPY	
SECTION	<u>HEP&T</u>
ENGINEER	<u>J. J. Rikala</u>
DATE	<u>10-31-90</u>
<input checked="" type="checkbox"/> PLANT LIFETIME	DATE TO PLANT <u>N/A</u>
<input type="checkbox"/> NON PERMANENT	MINIMUM RETENTION <u> </u> YRS.

44



RHR PUMP NPSH AT SATURATED CONDITIONS

Purpose

Westinghouse Owners Group (WOG) letter OG-90-44 dated August 2, 1990, identified a concern with regard to the operation of the residual heat removal (RHR) pumps while the plant is at half loop.

The letter states that, typically, a maximum reactor coolant system (RCS) temperature of 160°F is used to calculate the available NPSH for the RHR pump. While reviewing Generic Letters 87-12, "Loss of RHR while the RCS is partially filled," and 88-17, "Loss of decay heat removal," it became clear to the WOG that under these conditions, the RHR pump could be operating in the RHR mode taking suction from the RCS hot leg at saturated rather than subcooled conditions.

This calculation will determine if adequate NPSH is available to operate the RHR pump under saturated conditions, which could occur as a result of a loss of RHR event at mid-loop.

Conclusion

As shown on the attached graph (Attachment 1), there is adequate NPSH available for the RHR pumps to operate when they are taking suction from the RCS at saturated conditions (212°F when vented to atmospheric pressure).

Inputs

1. Westinghouse Owners Group (WOG) Letter OG-90-44 dated August 2, 1990, (Attachment 4)
2. RCS level at half-loop, 614"-0" per procedure 2-OHP-4021.002.005
3. RHR pump suction elevation 575'-0" per Drawing No. 1-5415-16
4. Hydraulic friction loss calculation program HFLC5 available in the HEP&T Section file 13.22.2.1
5. Ingersoll-Rand curve N-315 typical for the RHR pumps at Cook Nuclear Plant (Attachment 5)
6. The computer print-out attached is output from the HFLC5 program (Attachment 2). The input for the program (flow, pipe length, fittings and valves) is tabulated on the attached pipe friction data sheet (Attachment 3). The data sheet was generated from the drawings listed below:

1-5421	1-RH-28
1-5415	1-SI-4
1-RH-29	1-SI-2

7. Net positive suction head (NPSH) formula taken from Cameron Hydraulic Data, 16th edition Pg 1-10

For positive (flooded) suction

$$\text{NPSH} = h_a - h_{vpa} + h_{st} - h_{fs}$$

where: h_a = absolute pressure (in feet) on surface of the liquid supply level

h_{vpa} = vapor pressure of the liquid at the temperature being pumped (in feet)

h_{st} = static height (in feet) that liquid supply level is above or below the pump centerline

h_{fs} = all suction line losses (in feet)

Calculation

Net Positive Suction Head available

$$\text{NPSH}_a = h_a - h_{vpa} + h_{st} - h_{fs}$$

where: fluid = water at 212°F

h_a = 14.696 psi (33.96 feet) - 1 atmosphere

h_{vpa} = 14.696 psi (33.96 feet)

(Standard Handbook for Mechanical Engineers 7th edition - Marks, Table 28, page 4-38)

h_{st} = 614 - 575 = 39 feet

half loop elevation = 614 feet

pump centerline = 575 feet

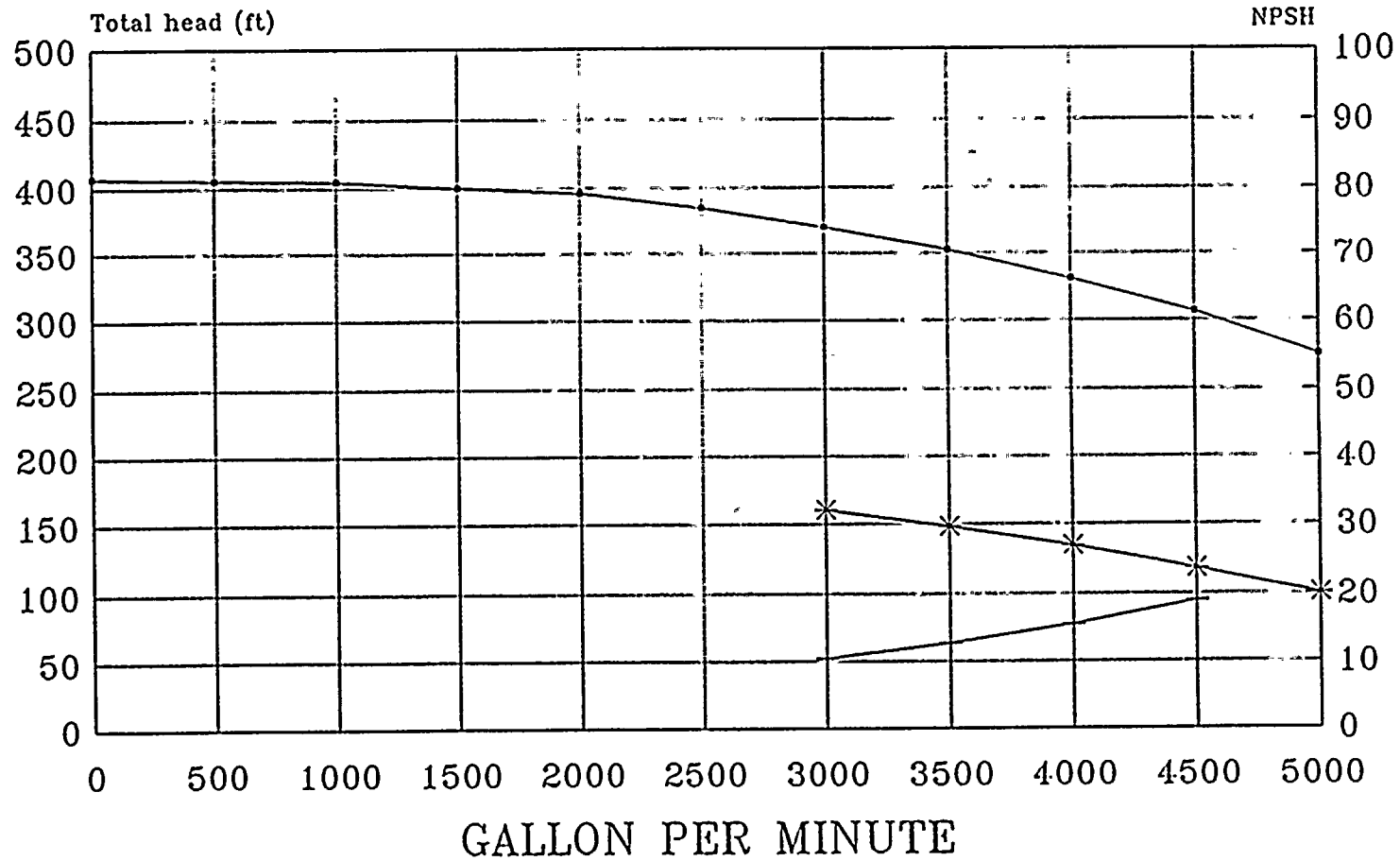
h_{fs} = 6.93 feet with fluid flow of 3000 gpm (ref. Attachment 2)

$$\begin{aligned} \text{NPSH}_a &= 33.96 - 33.96 + 39 - 6.93 \\ &= 32.07 \text{ feet} \end{aligned}$$

Similarly NPSH_a was calculated for the following flows:

<u>Flow (gpm)</u>	<u>NPSHa (feet)</u>
3000	32.07
3500	29.62
4000	26.83
4500	23.65
5000	20.1

Residual Heat Removal Pump



—●— PUMP CURVE —+— NPSHr —*— NPSHa(212 F)

jen 10/26/90

Attachment 1
HXPA009045EW

. PIPE FRICTION CALC - INPUT FILE IS-a:rhrrnpsha

INPUT DATA FOR THE HFLC5 SYS. RES. CALC.
CONSISTS OF THE FOLLOWING DATA:

T - TEMPERATURE DEG F
E - PIPE ABSOLUTE ROUGHNESS (FT.)
N - FIRST PIPE SEGMENT NUMBER
N1 - LAST PIPE SEGMENT NUMBER
QDES - DESIGN FLOW THRU PIPE SEGMENT (GPM)
QMIN - MINIMUM FLOW THRU PIPE SEGMENT (GPM)
QMAX - MAXIMUM FLOW THRU PIPE SEGMENT (GPM)
QDELT - FLOW INCREMENT THRU PIPE SEGMENT (GPM)
D - PIPE SEGMENT INTERNAL DIA. (IN.)
L - PIPE SEGMENT LENGTH (FT.)
K - PIPE SEGMENT K FACTORS
L/D - PIPE SEGMENT L/D FACTORS

FOLLOWING IS YOUR INPUT DATA

T	E	N	N1
212.00	.00015	1	1

QDES	QMIN	QMAX	QDELT	D	L	K	L/D
3000.00	3000.00	5000.00	500.00	13.124	170.35	.50	465.00

FOLLOWING IS HFLC5 RESULTS

WATER TEMP.(F)	= 212.00
DENSITY(LBM/CUFT)	= 59.74
WATER VISCOSITY(LBM/SEC/FT)	= .190201E-03
PIPE ABS ROUGHNESS(FT)	= .150000E-03

PIPE SEG NO	1	PIPE DIA(ID-IN) = 13.124				TOT	HD(FT)
FLOW-GPM	VEL(FPS)	LHD(FT)	KHD(FT)	LDHD(FT)			
3000.0	7.12	1.64	.39	4.90		6.93	
3500.0	8.30	2.22	.54	6.63		9.38	
4000.0	9.49	2.88	.70	8.59		12.17	
4500.0	10.67	3.63	.89	10.84		15.35	
5000.0	11.86	4.47	1.09	13.34		18.90	

REYNOLDS NUMBER FRICTION FACTOR TABLE

PIPE SEG	DES. FLOW	RE.NO.	F-FACTOR	HEAD LOSS
1	3000.0	2444029.0	.0134	6.93

PIPE FRICTION CALCULATION

DATA SHEET

RHR System

SHEET 1 OF 1
PLANT Cook Nuclear
BY JEW DATE
HXP900904JEW

SYSTEM From 2.9" Line (Half Loop) to RHR Pump Suction

REF. 1-5421, 1-5415, 1-RH-29, 1-RH-28, 1-SI-4, 1-SI-2

FLUID TEMP. (°F) 212 PIPE ABS. ROUGHNESS (FT) .00015 PIPE SEGMENT NUMBER 1

FLUID FLOW (GPM) 3000 PIPE I.D. (IN) 13.124 PIPE EL. 614'-0" TO EL. 575'-0"

STRAIGHT PIPE LENGTHS

FITTINGS

NUMBER

*K OR L/D

ΣK

ΣL/D

(Feet)					
2.34	GATE VALVE	111	13		39
1.75	GLOBE VALVE		340		
1.75	BUTTERFLY VALVE		40		
10.61	SWING CHECK		135		
21.38	90° STD. ELBOW		30		
5.33	90° S.R. ELBOW	111	50		150
7.75	90° L.R. ELBOW	THH THH	20		200
4.42	45° STD. ELBOW	⊙	16		16
6.75	45° S.R. ELBOW	⊙	26		
4.92	180° CLOSE RETURN		50		
7.0	STD. TEE RUN	111	20		60
29.25	STD. TEE BRANCH		60		
.5	* MITRE BENDS		1.2(1-COS θ)		
21.75	* LATERAL < OUTLET		1.0		
15.0	* LATERAL < INLET		0.5		
2.583	* STRAIGHT RUN LATERAL		0.15		
10.499	* PIPE ENTR PROJ. INWD.		0.78		
10.67	* " " SHARP EDGE	1	0.50	.50	
2.5	* " " WELL ROUND		0.04		
.92	* PIPE EXIT SHARP EDGED		1.0		
1.83	* ORIFICE (C _D = .61)		2.69 RF/β ⁴		
.85	* SUDDEN CONTRACTION †		.5(1-β ²)		
	* SUDDEN INCREASE †		(1-β ²) ²		
	* VALVE, MISCELLANEOUS		891.4 d ⁴ /C _V ²		
	MISC				
Σ 170.352				Σ 0.50	Σ 465

TOTALS

* ITEMS ARE "K" VALUES ONLY

† BASED ON SMALLER PIPE DIAMETER

β = d/D

RF = RECOVERY FACTOR

FK-9-1-72



RECEIVED
AUG 8 1990
MANAGERIAL DEPARTMENT

Attachment 4
HXP900904JEW

Westinghouse Owners Group

Domestic Utilities

Alabama Power
American Electric Power
Carolina Power & Light
Commonwealth Edison
Consolidated Edison
Duquesne Light
Duke Power

Georgia Power
Florida Power & Light
Houston Lighting & Power
New York Power Authority
Northeast Utilities
Northern States Power
Pacific Gas & Electric

Portland General Electric
Public Service Electric & Gas
Public Service of New Hampshire
Rochester Gas & Electric
South Carolina Electric & Gas
Southern California Edison
Tennessee Valley Authority

Texas Utilities Electric
Union Electric
Virginia Power
Wisconsin Electric Power
Wisconsin Public Service
Wolf Creek Nuclear
Yankee Atomic Electric

Foreign Utilities

Belgian Utilities
ENEL
Kansai Electric Power
Korea Electric
Nukleonna Elektrom
Saarland Utilities
Swedish State Power Board
Taiwan Power

OG-90-44

August 2, 1990

To: Westinghouse Owners Group Primary Representatives
Operations Subcommittee Representatives
Analysis Subcommittee Representatives

Subject: Westinghouse Owners Group
RHR Pump NPSH At Saturated Conditions

This letter is to notify all WOG utilities of a concern associated with operation of the RHR pump at mid-loop conditions that may apply at some plants.

When the plant is operating in the Residual Heat Removal (RHR) mode, the net positive suction head (NPSH) for the RHR pump is calculated based on subcooled conditions in the Reactor Coolant System (RCS). Typically, a maximum RCS temperature of 160°F is used to calculate the available NPSH for the RHR pump. Since the Reactor Coolant Pump (RCP) is assumed to be operating above this temperature, the RCS will be pressurized to approximately 400 psig to ensure proper RCP operating conditions and NPSH for the RHR pump is not a concern.

Over the past several years, a significant amount of effort has been expended by the utilities and Westinghouse to address NRC Generic Letters 87-12, "Loss of Residual Heat Removal (RHR) While the Reactor Coolant System (RCS) is Partially Filled" and 88-17, "Loss of Decay Heat Removal". In responding to the generic letters, it has become clear that under these conditions the RHR pump could be operating in the RHR mode taking suction from the RCS hot leg at saturated rather than subcooled conditions. In addition, the Westinghouse Owners Group has recently issued the Abnormal Response Guideline, ARG-1, "Loss of Residual Heat Removal While Operating at Mid-Loop Conditions", which directs the operator to run the RHR pump at saturated conditions if the RCS has heated up as a result of losing RHR while at mid-loop.



The available NPSH for the RHR pump will be much less at saturated RCS conditions than subcooled conditions. A preliminary investigation of this situation indicates that for some plants adequate NPSH may not be available at saturated conditions in the RCS, especially at higher RHR pump flowrates. A typical range of RHR pump flowrates is 500 gpm to 3,000 gpm. Based upon the results of this preliminary review, each plant should perform an evaluation to determine if adequate NPSH is available to operate the RHR pump under saturated conditions in the RCS, which could occur as a result of a loss of RHR event at mid-loop.

Should you have any questions concerning the above information, please contact Russ Oft at Westinghouse, 412-374-4465 or myself, 603-474-9574, X3347.

Very truly yours,



L.A. Walsh, Chairman
Operations Subcommittee
Westinghouse Owners Group

LAW/RRoft

cc: Steering Committee
J.B. George, TU Electric
C.K. McCoy, Georgia Power
K.J. Voytall

PWR "E" PP

CURVE NO. 11-3

DATE

6-18-71

CURVES ARE APPROXIMATE. PUMP GUARANTEED FOR ONE SET OF CONDITIONS CAPACITY, HEAD AND EFFICIENCY GUARANTEES ARE BASED ON SHOP TEST AND WHEN HANDLING CLEAR, COLD, FRESH WATER AT A TEMPERATURE OF NOT OVER 85° F. AND NOT OVER 15 FOOT SUCTION LIFT.

IMPELLER PATT. NO. 8X20A35

DIA. 17 1/8"

DIFFUSOR PATT. NO.

BHP 5 G. = 1.0

TOTAL HEAD IN FEET

PER CENT EFFICIENCY

EFFICIENCY

TOTAL HEAD

NPSH

NPSH FT

GALLONS PER MINUTE

THIS CERTIFIES THAT THIS CURVE IS BASED ON ACTUAL TEST PERFORMANCE.

W. P. Albert

W. P. Albert

6/18/71

WESTINGHOUSE SPIN NO. AEP-ACAPR-H-01

CHARACTERISTIC CURVE

NO. 8X20 TYPE H1 PUMP

1750 R. P. M.

PUMP NO. A6929

ORDER NO. 016-52294

INGERSOLL-RAND COMPANY ITEM 3A

CAMERON PUMP DIVISION

DATE 6-18-71

CURVE F-315

NED CALCULATION
VERIFICATION CHECKLIST

Calculation I.D. No.: HXP 900904JEW

Title: NPSH Calculation

Authorized Verifier: John J. Ripke Date: 10/30/00

Calculation Preparer: John E. Wagner Date: 10/30/90

The Authorized Verifier shall use this checklist to evaluate the calculation against the following questions. A basis for the response to all questions (reference to a section of the calculation or other document) shall be noted in the "Remarks" column as well as resolution of verifier's comments.

	Checked*	Remarks
1. Was an appropriate method used?	<u>X</u>	Use of the friction calc program, HFLC5 to calculate NPSH available is appropriate
2. Are the results reasonable compared to the input?	<u>X</u>	Operation at a flow rate of 4500 GPM without cavitation is reasonable
3. Are the results numerically correct?	<u>X</u>	The application of the HFLC5 output to the basic NPSH equation and curve was checked.
4. Are the equations used correct and the reference documented?	<u>X</u>	The calculation was based on the proper application of commonly used NPSH equation/methodology
5. Were the correct inputs used and their sources documented?	<u>X</u>	The correct inputs were used and were documented on pages 2+3
6. Are the assumption(s) reasonable?	<u>NA</u>	No assumptions were made.
7. Does the calculation include the necessary documentation as described in NEP 6.4, Sections 5.0 and 6.0?	<u>X</u>	The calculation was properly documented.
8. Is the calculation deemed acceptable?	<u>X</u>	The reviewer has determined that this calculation correctly determines the NPSH conditions of the described system

* Legend X = Yes NA = Not Applicable

100



Pipe Length and Fittings
14" PIPE

TEE BRANCH

3.5 ft

90° SR

2.97 ft

45°
TEERUN

9.35 ft

45°

20.89 ft

90°

6.58 ft

90°

15.25 ft

90°

4.60 ft

45°

68.5 ft

45°

3.29 ft

45°

14.16 ft

90°

2.83 ft

TOTAL = 152.32

1 90° EL SR

4 90° EL LR

5 45° EL

1 TEERUN

1 TEE BRANCH

42 ECCW Print 2-CON-39

18" PIPE

TEE BRANCH

14" X 18" DIFFUSER

21.5 ft

90°

9.91 ft

2.16 ft

TEE BRANCH

2.25 ft

BUTTERFLY VALVE

90° EL SR

4.33 ft

2.66 ft

TEE BRANCH

TOTAL = 42.81 ft

1 90° EL SR

1 90° EL LR

3 TEE BRANCH

1 BUTTERFLY VALVE

1 14 X 18 DIFFUSER

Walter E. McGray
10/10/97

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