

ASME SECTION XI VALVE TEST PROGRAM

2ND TEN YEAR INSPECTION INTERVAL

FOR THE D. C. COOK NUCLEAR POWER STATION UNIT NO. 2

Revision No: 2

Date: 8-31-87

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ASME SECTION XI VALVE TEST PROGRAM
2ND TEN YEAR INSPECTION INTERVAL
FOR THE D. C. COOK NUCLEAR POWER STATION UNIT NO. 2

INTRODUCTION

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Figure - 1

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1. Valve Testing Program

- A. The valve test program shall be conducted in accordance with Subsection IWV of Section XI of the 1983 edition of the ASME Boiler and Pressure Vessel Code through Summer 1983 Addenda, except for specific relief requests which are identified in the Valve Summary Sheet. | 2
- B. The valve test program is applicable for the second 10 year inspection interval which commences on July 1, 1986.
- C. The valve test program was developed employing the classification guidelines contained in 10 CFR 50.2(v) for Quality Group A and Regulatory Guide 1.26, Revision 3 for Quality Groups B and C. (Quality Group A is the same as ASME Class 1, Group B is 2, and Group C is 3). NRC staff guidance was provided by memorandum dated January 16, 1978.
- D. Figure 2 identifies the systems flow diagrams which were used to develop this valve test program.

Valve Summary Sheets contain the following:

- * System Name: Name of the system (e.g., Main Steam)
- * Flow Diagram: Unit Number - Flow diagram number - Revision Number (e.g., 2-5105B-42) | 2
- * Valve Number: Unique valve number (e.g., 2-DCR-310)
- * Revision Number: Any change of valve description, function or test requirement.

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* Valve Type: Type of valve, one of the following:

- REL - Relief and Safety
- CK - Check
- BF - Butterfly
- GA - Gate
- GL - Globe
- DA - Diaphragm
- 3W - Three-Way
- ND - Needle
- AG - Angle
- BL - Ball
- VB - Vacuum Breaker (Reverse Check Valve)

* Valve Size: Nominal valve size in inches

* Valve Actuator Type: Type of actuator, one of the following:

- SA - Self Actuated (e.g., CK or REL)
- MO - Motor Operated
- A - Air Operated
- M - Manual
- PO - Pneumatic
- SO - Solenoid Operated

* Flow Diagram Coordinates: Alpha/Numeric grid location of valve

* Valve position during normal plant operation or during performance of its safety function, one of the following:

- O - Open
- C - Closed
- O/C - Open/Closed or vice versa

* Code Class: ASME Code class of valve, either 1, 2, or 3

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* Valve Status-A/P: Active or passive

* Category: Section XI, Category of valve, either A, B, C, or D, as defined in IWV-2200

NOTE: Combinations are possible (e.g., AC)

* Primary Test Req'd: Test required per Section XI

* Test Performed: Testing that will be performed

NOTE: Test nomenclature is explained in Figure 3

* Test Mode: One of the following:

P - Power Operation (Every 3 months when unit is at power)

C - Cold Shutdown (Testing will be performed at cold shutdown frequency) See Note "F"

R - Refueling (Testing will be performed at refueling outage frequency)

* Code Relief: Whether or not a code relief is being requested; will be one of the following:

NO - Valve is to be tested per code, no comments

NO, NOTE X - Valve is to be tested per code, but there are comments

YES, NOTE Y - Code relief is requested. Alternate testing is proposed in lieu of that required by code, the note explains why the code relief is requested.

E. Alternative testing is to be performed on a check valve as indicated under relief request notes in lieu of stroke testing required by Section XI, IWV-3521. It is accomplished by disassembly or radiography method as explained below:

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- a. Disassembly Method The valve bonnet is removed, the disc is manually full-stroke exercised, and the valve internals are visually inspected. The results of this test are documented.
- b. Radiography Method This method is used as an alternative testing method to determine the disc position under no flow or reverse flow conditions (i.e., valve seated). This, in conjunction with the full forward flow testing of the valve, can provide assurance that the disc was free to move from full-open position with the forward flow to the closed position with no flow or reverse flow conditions. The results of this test are documented.

The valve grouping for sample disassembly or radiography under alternative testing is done based on the following criteria:

1. valve type and design similarities
2. system design, flow, and service conditions
3. frequency of valve operation
4. manufacturer
5. size range (i.e. up to 2", 2-1/2" to 4", 4" and 6", 6" and 8", etc.)

The alternative testing to be performed for a particular valve is indicated on the valve summary sheets and relief request notes. This will be accomplished by selecting one representative valve from each group of valves during refueling outages. If the inspection results are unacceptable, all the valves in that group will be inspected.



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F. Conditions for Valve Testing During Cold Shutdowns

Cold shutdown testing of valves identified in the IST Program is acceptable when the following conditions are met:

1. Valve exercising need not be done more often than once every 3 months in case of frequent cold shutdowns.
2. The testing shall commence as soon as the cold shutdown condition is achieved, but not later than 48 hours after shutdown, and continue until complete or the plant is ready to return to power.
3. Completion of all valve testing is not a prerequisite to return to power. Any testing not completed during one cold shutdown should be performed during any subsequent cold shutdowns starting from the last test performed at the previous cold shutdown.
4. For planned cold shutdowns, where ample time is available and testing all the valves identified for the cold shutdown test frequency in the IST Program will be accomplished, exceptions to the 48 hours commencement of testing is allowed.

G. The following criteria have been used in developing limiting values of full-stroke time for the power operated valves:

- o Review of valve's design specification and/or manufacturer's test stroke times
- o Review of system response time requirements (Technical Specification, FSAR, etc.)
- o Valve's historical stroke time values at various system conditions

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Using the above criteria, the limiting stroke time for each valve is derived as follows:

1. Valve Stroke Time \leq 2 Seconds

<u>Historical Stroke Time Range in Seconds</u>	<u>Established Base Line on Curves in Seconds</u>	<u>Recommended Action Time (Limiting Stroke Time Values in Seconds)</u>
		Based Line Time x 2 + 1 Second=Recommended Action Time or Tech. Spec. Limit, whichever is less..
up to .74	0.5	= .5 x 2 + 1 = 2 Seconds
.75 to 1.24	1.0	= 1 x 2 + 1 = 3 Seconds
1.24 to 1.74	1.5	= 1.5 x 2 + 1 = 4 Seconds
1.75 to 2.49	2.0	= 2 x 2 + 1 = 5 Seconds

2. Valve Stroke Time - 2.5 to 10.49 Seconds

2.5 to 10.49	3 to 10	Base Line Time x 1.5 = Action Time
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3. Valve Stroke Time - 10.5 Seconds and Up

10.5 and Up

11.0 and Up

Base Line Time x 1.25 =
Action Time (or 15 seconds,
whichever is larger)

- H. Code relief is requested for timing tolerance and trending per paragraphs IWV-3413(b) and IWV-3417(a) for fast acting valves (those with the maximum limiting stroke times of five seconds or less, determined from historical stroke time values and/or valve design specification). The major influence in the stroke time testing of fast acting valves is the operator's response. Therefore, timing tolerances are influenced by operator action and trending is not indicative of valve performance. The fast acting valves have been identified as "ETF" and acceptance criteria has been defined in the program.

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2ND TEN YEAR INSPECTION INTERVAL OF
VALVE TEST PROGRAM FOR UNIT - 2

LIST OF DRAWINGS

Revision No: 2

Figure 2

Date: 8-31-87

<u>NO.</u>	<u>SYSTEM</u>	<u>FLOW DIAGRAM NO.</u>	<u>REVISION</u>
	Main Steam	2-5105B	42
	Steam Generating System	2-5105D	2
	Feedwater	2-5106	34
	Feedwater (Auxiliary)	2-5106A	41
	Essential Service Water	2-5113	36
	Non-Essential Service Water	2-5114A	27
	Station Drainage Containment	2-5124	20
	Reactor Coolant	2-5128	19
	Reactor Coolant	2-5128A	34
	CVCS-Reactor Letdown & Charging	2-5129	32
		2-5129A	20
	Component Cooling	2-5135	34
	Component Cooling	2-5135A	30
	Component Cooling	2-5135B	14
	Nuclear Sampling	2-5141	27
	Nuclear Sampling	2-5141A	30
	Post Accident Sampling-Containment Hydrogen	2-5141D	8
	Emergency Core Cooling (SIS)	2-5142	28

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VALVE TEST PROGRAM FOR UNIT - 2

LIST OF DRAWINGS

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Figure 2

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<u>SYSTEM</u>	<u>FLOW DIAGRAM NO.</u>	<u>REVISION NO.</u>
Emergency Core Cooling (RHR)	2-5143	35
Containment Spray	2-5144	29
Containment Penetration & Weld Channel Pressurization	2-5145	20
Ice Condenser Refrigeration	2-5146B	23
Containment Ventilation	2-5147A	35
Control Room Ventilation	2-5149	23
Emergency Diesel Generator	2-5151A	26
Emergency Diesel Generator	2-5151B	27
Emergency Diesel Generator	2-5151C	26
Emergency Diesel Generator	2-5151D	27
Make-Up Water & Primary Water System	12-5115A	41
Compressed Air System	12-5120B	22
CVCS-Boron Makeup	12-5131	19
Spent Fuel Pit Cooling & Clean-Up	12-5136	25
WDS Vents & Drains	12-5137A	21
Post Accident Liquid & Gas Sampling	12-5141C	8
Post Accident Liquid Sampling Inst. Panels	12-5141F	6

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DONALD C. COOK NUCLEAR PLANT
NOMENCLATURE FOR TEST METHOD
USED IN COLUMNS FOR PRIMARY TEST REQUIRED AND
TEST PERFORMED UNDER ASME SECTION XI

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Figure 3

<u>1) CATEGORY A-B VALVES</u>		<u>ASME CODE SECTION XI PARAGRAPH</u>
EF-1	Exercise valve (full stroke) for operability every 3 months.	(IWV-3411)
EF-2	Exercise valve (full stroke) for operability at a cold shutdown or refueling outage frequency as indicated. Code relief requests are provided in the flow diagram notes.	(Code Relief Requested)
EF-3	Exercise valve (part stroke) for operability during operation; Exercise (full stroke) at a cold shutdown or refueling outage frequency as indicated. Justification for exercising the valve during cold shutdowns is provided in the flow diagram notes. Code relief request is provided if full stroke test is deferred to coincide with refueling outages.	(IWV-3412)
EF-4	Exercise valve (full stroke) for operability prior to return to service	(IWV-3416)
EF-5	Valves with remote position indicator shall be observed at least once every 2 years to verify that valve operation is accurately indicated.	(IWV-3300)
EF-6	This note was intentionally deleted.	
EF-7	Exercise valve (with fail-safe actuators) to observe failure mode every 3 months	(IWV-3415)
EF-8	Exercise valve (with fail-safe actuators) to observe failure mode at a cold shutdown or refueling frequency, as indicated.	(IWV-3415)
ET-XXX	Exercise valve - power operated (full stroke) and measure time. (E.G., ET-015 means recommended action stroke time is 15 seconds)	(IWV-3413 & 3417)
ETF-YYY	Exercise valve - power operated (full stroke) and measure time for fast acting valves. Code relief is requested in Item H. (E.G., ETF-005 means recommended action stroke time is 5 seconds)	(IWV-3413 & 3417)

DONALD C. COOK NUCLEAR PLANT
NOMENCLATURE FOR TEST METHOD
USED IN COLUMNS FOR PRIMARY TEST REQUIRED AND
TEST PERFORMED UNDER ASME SECTION XI

Revision No: 2

Figure 3

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<u>2) CATEGORY C VALVES</u>		<u>ASME CODE SECTION XI PARAGRAPH</u>
CF-1	Exercise valve (full stroke) for operability every 3 months.	(IWV-3521)
CF-2	Exercise valve (full stroke) for operability at a cold shutdown or refueling outage frequency as indicated. Code relief requests are provided in the flow diagram notes.	(Code Relief Requested)
CF-3	Exercise valve partial stroke during plant operation and full stroke for operability at a cold shutdown or refueling outage frequency as indicated. Justification for exercising valves during cold shutdown is provided in the flow diagram notes. Code relief requests are provided if full stroke testing is deferred to coincide with refueling outages.	(IWV-3522)
CF-4	Exercise valve (full stroke) for operability prior to return to service.	(IWV-3416)
TF-1	Safety and relief valve tests (setpoint) to Section XI, Table IWV-3510-1.	(IWV-3510)

<u>3) CATEGORY A or AC VALVES</u>	
SLT-1	Seat leakage test valve in accordance with requirements of paragraph IWV-3420 of ASME Code, Section XI, at refueling outage frequency but not less than once every two years. Permissible leakage values for each category A or AC valve are listed in Attachment - "A".
SLT-2	Seat leakage test valve in accordance with 10CFR 50, Appendix J, in lieu of ASME Code Section XI except for paragraphs IWV-3426 and IWV-3427 which are applicable. Permissible leakage values for each category A or AC valve are listed in Attachment-"A".

DONALD C. COOK NUCLEAR PLANT
NOMENCLATURE FOR TEST METHOD
USED IN COLUMNS FOR PRIMARY TEST REQUIRED AND
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Figure 3

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SLT-2A In lieu of the requirements of ASME Code Section XI, paragraphs IWV-3423 and IWV-3424, valves are seat leakage tested as part of the Appendix "J" containment isolation test by imposing a static head of water on the downstream side of the valve and verifying that the leakage within the specified value of Attachment "A" for each category valve. This testing method demonstrates that the containment spray and RHR Check Valve leakage over 30 days is limited to the water resident in the containment spray headers downstream of the check valves. The leakage specified would not deplete the water inventory so as to expose these valves to a post-LOCA environment for a minimum of 30 days in the event that a spray system must be shut down and drained. This testing method is as stated in Response to Question 22.15(5) of the original FSAR Appendix "Q", Amendment 81, dated August 1978.

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DONALD C. COOK NUCLEAR PLANT

VALVE TEST PROGRAM

RELIEF REQUEST NOTES

Flow Diagram No: 2-5105D-2

Revision No: 2

Date: 8-31-87

NOTE 1: FW-118-1 thru -4: The function of these valves is to provide feed-water flow from the main feedwater pumps to the steam generators. These valves cannot be exercised during power operation because closing of these valves would require securing feedwater flow to the steam generator and partial stroking may cause instability of steam generator water level which could result in reactor trip. Further three loop operation is not allowed per D. C. Cook Nuclear Plant Technical Specification 3.4.1.1. These valves will be confirmed closed by disassembly on a sampling basis during refueling outages.

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NOTE 2: MRV-210, -220, -230 and -240: These steam generator stop valves cannot be full stroke exercised during power operation because this would require securing steam from a steam generator which could result in a reactor trip. Three loop operation is not allowed for D. C. Cook per Technical Specification 3.4.1.1. Valves MRV-211, -221, -231, -241, -212, -222, -232, and -242 which activate MRV-210, -220, -230, and -240 are tested quarterly in accordance with IWV-3410. MRV-210, -220, -230 and -240 are part stroke tested quarterly by use of external hydraulics and full stroke tested during hot standby (Mode 3) at cold shutdown frequency.

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VALVE TEST PROGRAM

RELIEF REQUEST NOTES

Flow Diagram No: 2-5105D-2

Revision No: 2

Date: 8-31-87

NOTE 3: MS-108-2 and 108-3: These check valves are located in the steam supply lines to the Auxiliary Feedwater Pump Turbine. These valves operate during normal IST feedwater pump testing. Normal design flow rate for this pump is 900 GPM.. However, flow is restricted to a maximum of approximately 700 GPM through the 3" test line used during pump test. The check valves are therefore stroked to the extent that they pass the required steam flow to drive the turbine driven auxiliary feedpump at a flow rate of 700 GPM. Steam flow through these valves to the auxiliary feedwater pump turbine is verified to within 10% (at 900 GPM flow to SG's, 31,000 lbs/hr steam vs. 700 GPM flow through test line, 28,000 lbs/hr steam) of the maximum requirement. This is considered adequate for full stroke testing. In addition, due to the plant design, the only method available to verify the valve closure is disassembly. The valve is not equipped with position indication. The valves will be verified closed by disassembly on a sampling basis at a refueling frequency.

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DONALD C. COOK NUCLEAR PLANT

VALVE TEST PROGRAM

RELIEF REQUEST NOTES

Flow Diagram No: 2-5106-34

Revision No: 2

Date: 8-31-87

NOTE 1: FMO-201, -202, -203, -204 & FRV-210, -220, -230, -240: The function of these valves is to provide feedwater flow from the feedwater pumps to the steam generators. These valves cannot be exercised during power operation because closing these valves would require securing feed flow to the steam generator and partial stroking may cause instability of steam generator water level which could result in reactor trip. Further, three loop operation is not allowed per D. C. Cook Nuclear Plant Technical Specification 3.4.1.1. These valves will be full stroke exercised during unit start-up or shutdown.

DONALD C. COOK NUCLEAR PLANT

VALVE TEST PROGRAM

RELIEF REQUEST NOTES

Flow Diagram No: 2-5106A-41

Revision No: 2

Date: 8-31-87

NOTE 1: FW-132-1, -2, -3, -4: These auxiliary feedwater (AFW) check valves function to supply AFW to the steam generators whenever the AFW System is caused to operate. These check valves cannot be full or partial stroke exercised during power operation without energizing the AFW System and delivering cold water to the steam generators. This would result in thermal shock to the steam generator nozzles. These valves are full stroke exercised during startup. The valves will be verified closed quarterly by monitoring temperature of Auxiliary Feed Line as required by the plant procedure during shift inspection tours.

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NOTE 2: FW-134 & FW-135: These valves are located on the suction and discharge lines of the Turbine Driven Auxiliary Feedpump. The maximum flow rate through the Turbine Driven Auxiliary Feedpump during IST is approximately 700 gpm using the pump test line. The design flow rate is 900 gpm. In order to pass design flow of 900 GPM through, these valves would require delivering cold auxiliary feedwater to the steam generator nozzles. This would result in thermal shock to the steam generator nozzles. We consider that 700 GPM (78% of design flow) is sufficient to demonstrate full stroke operability. In addition, these valves are full stroke tested with flow rate of 900 GPM at a refueling outage frequency.

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DONALD C. COOK NUCLEAR PLANT

VALVE TEST PROGRAM

RELIEF REQUEST NOTES

Flow Diagram No: 2-5106A-41

Revision No: 2

Date: 8-31-87

NOTE 3: FW-138-1, -2, -3, -4: These auxiliary feedwater (AFW) check valves function to supply AFW to the steam generators whenever the AFW System is caused to operate. These check valves cannot be full or partial stroke exercised during power operation without energizing the AFW System and delivering cold water to the steam generators. This would result in thermal shock to the steam generator nozzles. The valves will be verified closed quarterly by monitoring temperature of Auxiliary Feed Line as required by the plant procedure during shift inspection tours. These valves are full stroke exercised when the plant is returned to power after cold shutdown.

NOTE 4: FW-149 and 150: The required full stroking of these check valves is satisfied when Turbine Driven Auxiliary Feedpump completes its required testing.

NOTE 5: FW-153 and 160: These check valves installed on the Emergency Leak Off (ELO) lines open when the Motor Driven Auxiliary Feedwater Pumps (MDAFP) start. This can be established when the MDAFP pump is operating through the test line. A pressure decrease in the pump discharge line is verified by a local pressure indicator when the parallel path ELO is opened. The pressure decrease indicates that flow is established through the ELO line and that the check valve is opened.

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VALVE TEST PROGRAM

RELIEF REQUEST NOTES

Flow Diagram No: 2-5113-36

Revision No: 2

Date: 8-31-87

NOTE 1: ESW-141, -142, -143, -144: These valves will be disassembled and inspected internally per procedure no. 12MHP-5021.032.00IL during every refueling outage.

NOTE 2: ESW-145, -240, -243: These valves are normally closed and are required to be open when the condensate storage tank is exhausted. Exercising the valves could cause lake water contamination of the steam generators. Lake water chemistry can potentially impact steam generator tube integrity. We believe that testing at a refueling outage frequency is sufficient to demonstrate operability of this long term valve. The valves will be full stroke tested during refueling outages. Since the valves are manual, stroke timing is not required.

NOTE 3: WRV-722, -724, -726, -728: These valves are located in the essential service water supply lines to the emergency diesel generators air after coolers. These three-way valves regulate water flow to maintain the temperature at which the after cooler air discharge thermostatic controller has been set. Water flow is regulated by passing a portion of the flow through the air coolers and bypassing the excess flow around the air after coolers. We are requesting code relief from the testing requirements since (1), these valves function only as regulating valves and not open/closed valves (2), these valves are demonstrated operable during diesel generator testing. Diesel generators are tested on a staggered basis, every 31 days per Technical Specification 4.8.1.1.2; and (3), these valves are demonstrated operable during diesel generator 24 hour runs performed each refueling outage. The valves will be "fail-safe" tested during refueling outages. The valves cannot be stroke timed because they are thermostatic valves whose position is controlled by process fluid temperature. There is no external control available.

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VALVE TEST PROGRAM

RELIEF REQUEST NOTES

Flow Diagram No: 2-5114A-27

Revision No: 2

Date: 8-31-87

NOTE 1: See "Attachment-A" for permissible seat leakage values.

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VALVE TEST PROGRAM

RELIEF REQUEST NOTES

Flow Diagram No: 2-5124-20

Revision No: 2

Date: 8-31-87

NOTE 1: See "Attachment-A" for permissible seat leakage values.

NOTE-2: NS-357: This check valve is located on the return line of the post accident sampling system inside the containment. Since the line is open-ended inside the containment and the check valve is not equipped with the position indication, the valve will be full stroke exercised in the open position by performing a flow test quarterly and will be confirmed closed during refueling outages.

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VALVE TEST PROGRAM

RELIEF REQUEST NOTES

Flow Diagram No: 2-5128-19

Revision No: 2

Date: 8-31-87

NOTE 1: NSO-021, -022, -023, & -024: These four one-inch solenoid operated isolation valves are installed (two in each leg in series) in the reactor head vent. These valves cannot be tested during power operation, hot standby, or hot shutdown because the valve design is such that testing of either valve can cause "burping" (momentary opening) of the second valve, resulting in the release of radioactive fluid and create an airborne situation in containment. The valves also cannot be tested during cold shutdown unless RCS is operating at half loop, because testing of these valves can create a similar situation as that described above. Since half loop operation of the RCS is not a normal evolution during cold shutdowns, full stroke test for these valves will be scheduled at refueling frequency.

Exercising the solenoid operated valves for verification of valve position (valve stem movement) will be performed during each refueling outage by performing a flow test through each valve because the valve stem is completely enclosed and cannot be observed. The reactor coolant discharged during the flow testing of the valves is collected in a container to minimize liquid contamination spill, radiation, and potential airborne situation in deference of ALARA consideration and personnel protection. The above tests are consistent with Technical Specification requirements.

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DONALD C. COOK NUCLEAR PLANT
VALVE TEST PROGRAM
RELIEF REQUEST NOTES

Flow Diagram No: 2-5128A-34
Revision No: 2
Date: 8-31-87

- NOTE 1: CS-442-1 thru 4: These containment isolation check valves are on the seal water supply line to the RC pumps. These valves cannot be part or full stroke exercised to the closed position during power operation because cooling flow is required to the RCP seals. During cold shutdown, seal water must be maintained to prevent backflow through the seals with possible damage from dirt. The valves will be full stroke exercised at refueling outage frequency.
- NOTE 2: See Attachment-"A" for permissible seat leakage values.
- NOTE 3: NRV-151, -152, -153: These pressurizer power operated relief valves are normally closed during power operation (passive valves). The system is considered out of service (as defined per IWV-3416) during power operation. The valves will be full stroke exercised prior to placing them into service for RCS cold overpressurization protection.
- NOTE 4: PW-275: This containment isolation check valve is located in the primary water supply line to the pressurizer relief tank. The valve is not equipped with position indication. The valve cannot be full stroke tested to closed position during power operation or at a cold shutdown frequency due to lack of sufficient differential pressure to back seat the valve. The valve and necessary test connections are located inside the containment. Due to the plant design, the only method available to verify the valve closure is leak testing. The valve will be verified closed during seat leakage testing per Appendix "J" program at refueling frequency.
- NOTE 5: N-159: This containment isolation check valve is located in the nitrogen supply line to the pressurizer relief tank. The valve is not equipped with position indication. The valve cannot be full stroke tested to closed position during power operation or at a cold shutdown frequency due to lack of sufficient differential pressure to back seat the valve. The valve and necessary test connections are located inside the containment. Due to the plant design, the only method available to verify the valve closure is leak testing. The valve will be verified closed during seat leakage testing per Appendix "J" program at refueling frequency.

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DONALD C. COOK NUCLEAR PLANT
VALVE TEST PROGRAM
RELIEF REQUEST NOTES

Flow Diagram No: 2-5128A-34

Revision No: 2

Date: 8-31-87

NOTE 6: NSO-061, -062, -063, -064: These four one-inch operated isolation valves are installed (two in each leg in series) in the pressurizer vent. These valves cannot be tested during power operation, hot standby, or hot shutdown because the valve design is such that testing of either valve can cause "burping" (momentary opening) of the second valve resulting in the release of radioactive fluid and create an airborne situation in containment. The valves also cannot be tested during cold shutdown unless RCS is operating at half loop, because testing of these valves can create a similar situation as that described above. Since half loop operation of the RCS is not a normal evolution during cold shutdowns, full stroke test for these valves will be scheduled at refueling frequency.

Exercising the solenoid operated valves for verification of valve position (valve stem movement) will be performed during each refueling outage by performing a flow test through each valve because the valve stem is completely enclosed and cannot be observed. The reactor coolant discharged during flow testing of the valves is collected in a container to minimize contaminated liquid spill, radiation, and potential airborne situation in deference of ALARA consideration and personnel protection. The above tests are consistent with Technical Specification requirements.

NOTE 7: SI-189: This check valve is located in the safety valves discharge (Emergency Core Cooling SVs, RHR, SVs, centrifugal charging pump SVs, etc.) collection header leading to the pressurizer relief tank. Isolating this valve for testing would result in dead heading all safety valves in the above systems. This would result in loss of overpressurization protection and could put the plant in an unsafe condition. Test will be run at a refueling frequency when there is not potential for overpressurization.



VALVE TEST PROGRAM

RELIEF REQUEST NOTES

Flow Diagram No: 2-5129-32

Revision No: 2

Date: 8-31-87

NOTE 1: CS-292: This valve is in the emergency boration path from the boric acid system to the charging pump suction header. Flow through this path is normally not provided at power because of the resultant large negative reactivity insertion. The valve will be full stroke exercised in the open position at a cold shutdown frequency. The check valve is not equipped with position indication. Due to the plant design, the only methods available to verify the valve closure is either radiography or disassembly which can only be performed during the refueling outage when the system is not required to be operable. The radiography method is an acceptable method to verify the valve closure (disc against the seat) under no flow condition because it provides visual observation of the valve in the closed position. The flow testing of the valve verifies that it is open. This provides assurance that the disc is free to move from the open position with flow to the closed position with no flow or reverse flow.

NOTE 2: CS-299E, -299W: These check valves located on the discharge lines of the 'E' and 'W' charging pumps function as pressure isolation valves to protect the low pressure charging pump suction lines. These valves cannot be full-stroke exercised during: (1) power operation because the charging pumps cannot achieve maximum flow rate with the reactor at full pressure, and (2) cold shutdown because the flow required could cause a low temperature overpressure condition. The valves will be part-stroke exercised quarterly and full stroke exercised during refueling outages.

NOTE 3: See Attachment-"A" for permissible seat leakage values.

DONALD C. COOK NUCLEAR PLANT

VALVE TEST PROGRAM

RELIEF REQUEST NOTES

Flow Diagram No: 2-5129-32

Revision No: 2

Date: 8-31-87

- NOTE 4: CS-321: This containment isolation check valve's function is to supply borated water from the volume control tank to the regenerative heat exchanger through the charging pumps for chemical shim control and reactor coolant system makeup. Isolation of this system would result in loss of control of pressurizer level which could result in reactor trip. This valve is tested in the open direction quarterly and confirmed closed during refueling outages in deference to ALARA consideration.
- NOTE 5: CS-328L1, -329L1, -328L4, -329L4: These check valve's function is to provide the interface point between the RCS and the CVCS. Since the discharge piping of the CVCS is designed to a pressure rating higher than the RCS, these valves do not perform a pressure isolation function. The higher pressure (RCS) to low pressure (CVCS Suction) isolation is accomplished by other valves which are tested to category "A" requirement. The valves will be full stroke exercised to open position quarterly.
- NOTE 6: QCR-300, -301: These air operated containment isolation valves are located on the letdown return line. Exercising these valves during power operation would result in letdown isolation which could result in loss of pressurizer level control which could result in a plant shutdown. The valves will be full stroke exercised and fail safe tested at a cold shutdown frequency and seat leakage tested during refueling outages.
- NOTE 7: QMO-200, -201: These motor operated gate valves are installed on the CVCS charging line to provide borated water for RCS chemical shim control and reactor coolant system makeup. Isolation of this system would result in loss of control of pressurizer level which could result in reactor trip. The valves will be tested at a cold shutdown frequency.

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DONALD C. COOK NUCLEAR PLANT

VALVE TEST PROGRAM

RELIEF REQUEST NOTES

Flow Diagram No: 2-5129-32

Revision No: 2

Date: 8-31-87

NOTE 8: QRV-200: This control valve is required for emergency boration in Modes 5 and 6. The valve is normally open to permit charging. The valve cannot be full stroke tested during power operation because it would interrupt charging flow which could effect pressurizer level. The valve will be part stroke exercised during power operation and will be full stroke exercised at cold shutdown frequency. The valve cannot be "fail safe" tested nor stroke timed since position indication is not provided.

NOTE 9: QRV-251: This control valve is required for emergency boration in Modes 5 and 6. The valve is normally open to permit charging flow when the centrifugal charging pumps are running. The valve cannot be full stroke tested during power operation because it would interrupt both charging and seal water flow which could effect the pressurizer level and damage the reactor coolant pump seals. The valve will be part stroke exercised during power operation and full stroke exercised at cold shutdown frequency. This valve cannot be "fail safe" tested nor stroke timed since no position indication is provided.

NOTE 10: SI-185: This normally closed valve functions to transfer the suction source of the charging pumps to the refueling water storage tanks. This valve cannot be full stroke exercised during: (1) power operation without introducing a high concentration of boric acid in the RCS, and (2) cold shutdown because the only full flow path available is into the reactor coolant system and the system does not have sufficient volume to accommodate that flow without a possible low temperature overpressure condition. The valve will be full stroke exercised during refueling outages.

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DONALD C. COOK NUCLEAR PLANT

VALVE TEST PROGRAM

RELIEF REQUEST NOTES

Flow Diagram No: 2-5129A-20

Revision No: 2

Date: 8-31-87

NOTE 1: QCM-250, -350: These motor-operated reactor coolant pump seal water return isolation valves cannot be exercised during power operation because it would interrupt reactor coolant pump seal water flow and could cause damage to the seals. Therefore, the valves are full stroke exercised at cold shutdown frequency.

NOTE 2: See "Attachment-A" for permissible seat leakage values.

NOTE 3: QMO-451, -452: These motor-operated gate valves function as volume control tank isolation valves. Exercising these valves during power operation could result in a loss of pressurizer level control which could cause a reactor trip. These valves are full stroke exercised at cold shutdown frequency.

DONALD C. COOK NUCLEAR PLANT

VALVE TEST PROGRAM

RELIEF REQUEST NOTES

Flow Diagram No: 2-5135-34

Revision No: 2

Date: 8-31-87

- NOTE 1: CCM-451, -452, -453, -454, -458 and -459: These valves cannot be tested during power operation without securing cooling water to the RC pump. Isolation of these valves could cause failure of the RCP. Valves to be tested at cold shutdown frequency.
- NOTE 2: See "Attachment-A" for permissible seat leakage values.
- NOTE 3: CCR-455, -456, and -457: These valves cannot be tested during power operation without securing cooling water to reactor the support coolers. These valves must remain open to prevent overheating of the concrete around the reactor supports during the normal operation. Valves to be tested at a cold shutdown frequency.
- NOTE 4: CCW-135: This check valve cannot be tested during power operation without securing cooling water to the reactor support coolers. The valve must remain open to prevent overheating of the concrete around the reactor supports during the normal operation. The valve will be verified closed by seat leakage testing during refueling outages.
- NOTE 5: CRV-470: This air-operated valve is located in the Component Cooling Water (CCW) return from the letdown heat exchanger and controls the temperature of the letdown flow leaving the heat exchanger. The position of this valve is set by QTC-302. The valve cannot be "fail-safe" tested nor stroke timed since no control switches are installed to perform those tests. The valve will be full stroke exercised quarterly.

DONALD C. COOK NUCLEAR PLANT

VALVE TEST PROGRAM

RELIEF REQUEST NOTES

Flow Diagram No: 2-5135A-30

Revision No: 2

Date: 8-31-87

NOTE 1: CMO-411, -412, -413, -414, -415 & -416: These valves remain open during initial safety injection, but may be closed during recirculation phase or passive failure. Therefore, the valve time will be recorded from open to close position.

DONALD C. COOK NUCLEAR PLANT

VALVE TEST PROGRAM

RELIEF REQUEST NOTES

Flow Diagram No: 2-5135B-14

Revision No: 2

Date: 8-31-87

NOTE 1: See "Attachment-A" for permissible seat leakage values.

NOTE 2: CCW-243-25, CCW-243-72, CCW-244-25 and CCW-244-72: These check valves are located in the penetration cooling supply headers of the CCW System inside the containment. The valves are open during power operation and cold shutdown to provide cooling water to the main steam penetrations. These valves are not equipped with position indication. The valves are confirmed closed by seat leakage testing in accordance with Appendix "J" during refueling outage.

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DONALD C. COOK NUCLEAR PLANT

VALVE TEST PROGRAM

RELIEF REQUEST NOTES

Flow Diagram No: 2-5141-27

Revision No: 2

Date: 8-31-87

NOTE 1: See "Attachment-A" for permissible seat leakage values.

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VALVE TEST PROGRAM

RELIEF REQUEST NOTES

Flow Diagram No: 2-5141D-8

Revision No: 2

Date: 8-31-87

NOTE 1: See "Attachment-A" for permissible seat leakage values.

NOTE 2: NS-283: This containment isolation check valve is located in the sample return line of the Post-Accident Containment Hydrogen Monitoring System. The valve cannot be full stroke exercised to closed position quarterly or at a cold shutdown frequency because: 1) due to the plant design (line is open ended in the containment), the only method available to verify the valve closure is leak testing, and 2) check valve is not equipped with position indication. The valve will be full stroke exercised in the open position by performing a flow test quarterly and will be confirmed closed during the seat leakage testing per Appendix "J" program at a refueling frequency.

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DONALD C. COOK NUCLEAR PLANT

VALVE TEST PROGRAM

RELIEF REQUEST NOTES

Flow Diagram No: 2-5142-28

Revision No: 2

Date: 8-31-87

- NOTE 1: ICM-250 and ICM-251: These normally closed valves cannot be operated during normal plant operation without introducing Boron into a nonheat traced line. Boron could crystallize and plug the line. Valves will be tested at cold shutdown frequency.
- NOTE 2: See "Attachment-A" for permissible seat leakage values.
- NOTE 3: IMO-261: This valve cannot be tested when SI pumps are required to be operable. Testing would result in isolation of both pumps. This valve will be tested at cold shutdown frequency.
- NOTE 4: IMO-262 and -263: These motor operated valves are located in series in the re-circulation line of the Safety Injection pumps. Exercising any of these valves will make the SI pumps inoperable. These valves will be full-stroke exercised at cold shutdown frequency.
- NOTE 5: SI-110N, SI-110S and SI-101: Safety Injection (SI) pump discharge valves, SI-110N and -110S, cannot be exercised during power operation because SI pumps cannot overcome reactor pressure. Therefore, no flow path exists and, because minimum flow lines branch off upstream of these valves, they cannot be part-stroke tested during pump testing. The common (SI pumps) suction check valve, SI-101 is part-stroke exercised at power operation during pump testing. These valves cannot be exercised during cold shutdowns because SI pumps are required to be inoperable by Technical Specification 3.5.3 to protect against low temperature over-pressurization of the Reactor Coolant System. These valves will be full-stroke exercised at refueling frequency in conjunction with the full flow test.

DONALD C. COOK NUCLEAR PLANT

VALVE TEST PROGRAM

RELIEF REQUEST NOTES

Flow Diagram No: 2-5142-28

Revision No: 2

Date: 8-31-87

NOTE 6: SI-142 L1, L2, L3, and L4: These check valves are located in the supply lines from the Boron Injection Tank to the reactor coolant cold legs (loop 1 through 4). These valves cannot be tested during power operation because this would require injecting highly concentrated boric acid solution from the Boron Injection Tank into the Reactor Coolant System resulting in probable plant shutdown.

These valves cannot be partially-stroke exercised using the BIT bypass line because this could result in bypassing the BIT, thereby not achieving design flow through the BIT if an accident occurred.

These valves cannot be full-stroked exercised during cold shutdown because this would require injecting the BIT into the RCS which could significantly delay startup from cold shutdown condition (the BIT would have to be brought to the proper Boron concentration and the RCS would have to be diluted sufficiently to allow startup). These valves will be full stroke exercised during refueling outages.

DONALD C. COOK NUCLEAR PLANT

VALVE TEST PROGRAM

RELIEF REQUEST NOTES

Flow Diagram No: 2-5143-35

Revision No: 2

Date: 8-31-87

NOTE 1: See "Attachment-A" for permissible seat leakage values.

NOTE 2: IMO-128 and ICM-129: These valves function as the normal return from the RCS to the RHR for heatup and cooldown. These valves are normally closed and cannot be operated during normal plant operation because they are interlocked to remain closed at RCS pressure above 450 psig. The valves will be full stroke exercised prior to placing them into service at a cold shutdown frequency.

NOTE 3: IMO-310, -320, -314, -324: These valves remain open during injection phase of a safety injection, but will be closed during recirculation phase. Therefore, stroke timing will be from open to close position.

NOTE 4: IMO-315, -316, -325, -326: Valves IMO-315 and -325 are normally closed valves, located in the RHR and SI Supply Header to RCS hot legs. Valves IMO-316 and -326 are normally open valves located in the RHR and SI Supply Header to RCS cold legs. These valves should not be exercised during power operation because failure in a non-conservative position would result in less than minimum number of injection flow path as required by the FSAR. The valves will be full stroke tested at cold shutdown frequency.

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VALVE TEST PROGRAM

RELIEF REQUEST NOTES

Flow Diagram No: 2-5143-35

Revision No: 2

Date: 8-31-87

NOTE 5: SI-166 L1, L2, L3, L4: These check valves function to prevent back-flow from the RCS into the accumulators during normal operation. These valves function to supply flow from the accumulators to the RCS during an accident condition. These valves cannot be exercised during power operation because the accumulators do not have sufficient head to overcome RCS pressure.

These valves cannot be exercised during cold shutdown because this would result in a possible low temperature overpressurization of the RCS. Full stroke testing during refueling outages is not possible because of the resulting water surge into the reactor and the potential for high airborne radiation contamination. These valves will be partially-stroke exercised during refueling outages and disassembled for internal inspection on a sampling basis.

NOTE 6: SI-161, L1, L2, L3, L4: These check valves are located in the supply lines from the Residual Heat Removal and Safety Injection Pumps to the RCS cold legs (loop 1 through 4). These valves cannot be exercised during power operation because the RHR pumps and SI pumps do not develop sufficient head to overcome RCS pressure. The valves cannot be full stroke exercised during cold shutdown due to potential low temperature overpressurization damage to reactor vessel. These valves will be part stroke exercised at a cold shutdown frequency and will be full stroke exercised at the refueling frequency when reactor vessel head is removed.

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VALVE TEST PROGRAM

RELIEF REQUEST NOTES

Flow Diagram No: 2-5143-35

Revision No: 2

Date: 8-31-87

- NOTE 7: RH-108E and RH-108W: These valves cannot be full stroke exercised quarterly because no full flow path exists. The valves will be full stroke exercised at cold shutdown frequency (during RHR operation).
- NOTE 8: RH-133, -134: These check valves function to circulate water from the RHR pumps to the RCS cold legs when the RHR system is aligned for heat removal operation. These valves cannot be exercised during power operation because the RHR pumps do not develop sufficient head to overcome RCS pressure. These valves will be exercised during start up pursuant to Technical Specification 3.4.6.2.
- NOTE 9: SI-148: Check valve SI-148 is located in the Refueling Water Storage Tank (RWST) supply line to the RHR system. The design flow through the valve is 6000 GPM. Flow to the core is not possible when the RCS pressure is above the shut-off pressure of the RHR pumps (195 psig). In order to full stroke exercise this valve, both RHR pumps must be operated and the RHR system manually aligned to recirculate flow back to the RWST. This configuration places both RHR trains inoperable since neither train can provide design flow to the core. In order to preclude placing the unit in an unsafe condition, a partial stroke test is performed quarterly with both trains operable. The valve cannot be full stroke exercised during cold shutdown since water solid RCS cannot accommodate the introduction of 6000 GPM from the RHR system. In addition, during cold shutdown, the RHR system is required to be operable for RCS temperature control. The valve will be full stroke exercised when the reactor cavity is being flooded at a refueling frequency.

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VALVE TEST PROGRAM

RELIEF REQUEST NOTES

Flow Diagram No: 2-5143-29

Revision No: 2

Date: 8-31-87

- NOTE 10: SI-151 E, W: These check valves are located in the RHR supply lines to either the hot or cold legs. These valves cannot be exercised during power operation because the RHR pumps do not develop sufficient head to overcome RCS pressure. These valves will be exercised at a cold shutdown frequency.
- NOTE 11: SI-152 N, S: These check valves function to provide Safety Injection pump discharge to either the hot or cold legs. These valves cannot be exercised during power operation because the SI pumps do not develop sufficient pressure to overcome RCS pressure. These valves cannot be exercised during cold shutdown because the safety injection pumps are required to be inoperable by Technical Specification Section 3.5.3, to protect against low temperature overpressurization of the RCS. Also, during cold shutdown, there may not be sufficient volume in the RCS to accommodate the amount of water needed to full stroke. These valves will be full stroke exercised at refueling frequency in conjunction with full flow test.

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VALVE TEST PROGRAM
RELIEF REQUEST NOTES

Flow Diagram No: 2-5143-35

Revision No: 2

Date: 8-31-87

- NOTE 12: SI-158 L1, L2, L3, L4: Check valves SI-158 are located in the supply lines from the Residual Heat Removal and Safety Injection Pumps to the RCS hot legs (loop 1 through 4). These valves cannot be exercised during power operation because the RHR and SI pumps do not develop sufficient head to overcome RCS pressure. The valve cannot be full stroke exercised during cold shutdown due to potential low temperature cold overpressurization damage to the reactor vessel. These valves will be part stroke exercised at a cold shutdown frequency and will be full stroke exercised at the refueling frequency when reactor vessel head is removed.
- NOTE 13: SI-170 L1, L2, L3, and L4: These valves are located on the RCS cold leg (loops 1 through 4) injection lines from the accumulators, RHR, and SI systems. They cannot be exercised during power operations because the RHR and SI pumps do not develop sufficient head to overcome RCS pressure. The valves will be part-stroke exercised at a cold shutdown frequency. Due to the plant design, the valves are sized as such that full stroke testing cannot be attained without discharging the accumulators and operating SI and RHR pumps simultaneously. The only method available to verify the full stroke exercising is disassembly method. The valve is not equipped with position indication. The valves will be disassembled on a sampling basis during refueling outages to verify full stroke exercising.
- NOTE 14: N-102 This check valve is located in the nitrogen supply header to the accumulators for blanketing purposes. The valve cannot be full stroke tested to closed position during power operation or cold shutdown because, due to the plant design, the only method available to verify the valve closure is leak testing. The valve and necessary test connections are located inside the containment. The valve is not equipped with position indication. The valve will be verified closed during seat leakage testing per Appendix "J" program at refueling frequency.

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DONALD C. COOK NUCLEAR PLANT

VALVE TEST PROGRAM

RELIEF REQUEST NOTES

Flow Diagram No: 2-5144-29

Revision No: 2

Date: 8-31-87

NOTE 1: CTS-138E & W, CTS-103E & W: These check valves are located in the lines supply water from the RWST to the Containment Spray Pumps suction (CTS-138E & W) and the discharge line from the Containment Spray Pumps (CTS-103E & W) to the ring header in containment. The valves cannot be full stroke exercised during power operation, cold shutdown, or refueling without spraying the containment. These valves are partial stroked during Containment Spray Pump Testing. These valves are identical in type, design, and operate at similar frequencies, flows, and temperatures. Since the design conditions and size are sufficient to warrant grouping them, the valves will be disassembled during refueling on a sample basis.

NOTE 2: CTS-131E & W, CTS-127E & W, RH-141, RH-142: These check valves are located in the supply lines to the containment spray ring headers. During normal plant operation, they are in the closed position. They are exposed to the containment atmosphere on the downstream side and are isolated from fluid pressure in the upstream side by closed MOVs. These valves cannot be exercised during power operation, cold shutdown, or refueling because flow through these valves would result in spraying the containment. This could cause problems with wet lagging, corrosion of components inside containment, etc. The only practical method of verifying operability of these check valves is by disassembly. These valves are identical in type and design and operate at similar conditions and frequencies. Since the design conditions and size are sufficiently similar to warrant grouping them, the valves will be disassembled on a sample basis during refueling outages.

NOTE 3: CTS-127E & W, CTS-131E & W, RH-141, RH-142: These valves are to be seat leakage tested in accordance with the special testing procedure because of the configuration at D. C. Cook Plant. The permissible seat leakage values of these valves are listed in Attachment-"A".

DONALD C. COOK NUCLEAR PLANT

VALVE TEST PROGRAM

RELIEF REQUEST NOTES

Flow Diagram No: 2-5145-20

Revision No: 2

Date: 8-31-87

NOTE 1: See "Attachment-A" for permissible seat leakage values.

DONALD C. COOK NUCLEAR PLANT

VALVE TEST PROGRAM

RELIEF REQUEST NOTES

Flow Diagram No: 2-5146B-23

Revision No: 2

Date: 8-31-87

NOTE 1: R-156 and R-157: These check valves are installed in parallel lines to the glycol main supply and return lines mainly to relieve glycol thermal expansion. These valves and necessary test connections are located inside the containment. Due to the plant design, the only method available to verify valve closure is leak testing. The valves are not equipped with position indication. The valves will be full stroke exercised in the open direction quarterly and verified closed by seat leakage testing per Appendix "J" at refueling frequency.

NOTE 2: See "Attachment-A" for permissible seat leakage values.

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DONALD C. COOK NUCLEAR PLANT

VALVE TEST PROGRAM

RELIEF REQUEST NOTES

Flow Diagram No: 2-5147A-35

Revision No: 2

Date: 8-31-87

NOTE 1: See "Attachment-A" for permissible leakage values.

DONALD C. COOK NUCLEAR PLANT

VALVE TEST PROGRAM

RELIEF REQUEST NOTES

Flow Diagram No: 2-5149-23

Revision No: 2

Date: 8-31-87

NOTE 1: VRV-325, -315: These valves are located at the outlet of the control room air conditioner water pump. These three-way valves function to modulate water flow through the air handler package based on cooling requirements. These valves are demonstrated operable during normal control room air conditioning operation. The valves cannot be stroked timed because they are not equipped with position indicator and stroke times are not repeatable.

DONALD C. COOK NUCLEAR PLANT

VALVE TEST PROGRAM

RELIEF REQUEST NOTES

Flow Diagram No: 2-5151A-26

Revision No: 2

Date: 8-31-87

NOTE 1: The required full stroking of the check valves is satisfied when the diesel generator successfully completes its required testing per Technical Specification 4.8.1.1.2.

NOTE 2: QT-114-2AB: This valve is located at the discharge of the engine driven lube oil pump (diesel-generator). This three-way thermostatic valve functions to maintain the correct lube oil temperature by maintaining the correct proportion of oil flowing through the lube oil cooler and bypassing the lube oil cooler to maintain a preset lube oil temperature. We are requesting exemption from testing requirements since (1) this valve functions only as a regulating valve and not opened/closed; (2) this valve is demonstrated operable during diesel generator testing. Diesel generators are tested every 31 days on a staggered basis per Technical Specification 4.8.1.1.2. The valves will be verified operable by observing proper temperatures during diesel testing.

DONALD C. COOK NUCLEAR PLANT

VALVE TEST PROGRAM

RELIEF REQUEST NOTES

Flow Diagram No: 2-5151B-27

Revision No: 2

Date: 8-31-87

- NOTE 1: The required full stroking of the check valves is satisfied when the diesel generator successfully completes its required testing per Technical Specification 4.8.1.1.2.
- NOTE 2: QT-132-2AB: This valve is located at the discharge of the emergency diesel engine jacket water pump. This three-way thermostatic valve functions to maintain the correct proportion of water flowing through the diesel engine water cooler and bypassing the diesel engine jacket water cooler to maintain a preset jacket water temperature. We are requesting exemption from the testing requirements since (1) this valve functions only as regulating valve and not open/closed valve; (2) this valve is demonstrated operable during diesel generator testing. Diesel generators are tested on a staggered basis, every 31 days per Technical Specification 4.8.1.1.2. The valve will be verified operable by observing proper temperatures during diesel testing.
- NOTE 3: XRV-220, (Jet Assist), -221 and -222 (Starting Air): The starting air valves are installed on parallel air supply lines to a diesel generator. The valves are not equipped with position indication devices to directly measure valve stroke times. Stroke timing is verified by measuring diesel starting times (Technical Specification acceptance ten seconds or less). The valves on a staggered basis are valved out one at a time to verify the operability of the opposite valve during diesel testing. The consistent compliance of the diesel generator start times (typically seven to nine seconds) demonstrates the valve performance. Position indication is confirmed during the above testing when only one starting air train is used to start the diesel generators. The starting air valves do not require or have a fail safe position because there is a redundant starting air supply system for each diesel generator.
(continued)

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DONALD C. COOK NUCLEAR PLANT

VALVE TEST PROGRAM

RELIEF REQUEST NOTES

Flow Diagram No: 2-5151B-27

Revision No: 2

Date: 8-31-87

(Note 3 continued)

Similarly, stroke timing of jet assist valve XRV-220 is verified by measuring diesel starting times. The jet assist valve's fail safe position is "closed". The jet assist valve's function is to facilitate diesel generator quick start. Its failure to open upon a diesel start signal could cause the effected diesel generator to achieve operating conditions more slowly. Based on single failure criteria, the other diesel will start within the required ten seconds, thereby meeting system requirements. If this valve were to fail open, (open is the position to assist diesel generator starting) then the starting air reservoirs would bleed down, rendering that diesel generator inoperable. Therefore, in order to assure the starting air supply, the fail safe mode for this valve is closed.

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DONALD C. COOK NUCLEAR PLANT

VALVE TEST PROGRAM

RELIEF REQUEST NOTES

Flow Diagram No: 2-5151C-26

Revision No: 2

Date: 8-31-87

NOTE 1: The required full stroking of the check valves is satisfied when the diesel generator successfully completes its required testing per Technical Specification 4.8.1.1.2.

NOTE 2: QT-114-2CD: This valve is located at the discharge of the engine driven lube oil pump (diesel-generator). This three-way thermostatic valve functions to maintain the correct lube oil temperature by maintaining the correct proportion of oil flowing through the lube oil cooler and bypassing the lube oil cooler to maintain a preset lube oil temperature. We are requesting exemption from testing requirements since (1) this valve functions only as a regulating valve and not opened/closed valve; (2) this valve is demonstrated operable during diesel generator testing. Diesel generators are tested every 31 days on a staggered basis per Technical Specification 4.8.1.1.2. The valves will be verified operable by observing proper temperatures during diesel testing.

DONALD C. COOK NUCLEAR PLANT

VALVE TEST PROGRAM

RELIEF REQUEST NOTES

Flow Diagram No: 2-5151D-27

Revision No: 2

Date: 8-31-87

NOTE 1: The required full stroking of the check valves is satisfied when the diesel generator successfully completes its required testing per Technical Specification 4.8.1.1.2.

NOTE 2: QT-132-2CD: This valve is located at the discharge of the emergency diesel engine jacket water pump. This three-way thermostatic valve functions to maintain the correct proportion of water flowing through the diesel engine water cooler and bypassing the diesel engine jacket water cooler to maintain a preset jacket water temperature. We are requesting exemption from the testing requirements since (1) this valve functions only as a regulating valve and not open/closed valve; (2) this valve is demonstrated operable during diesel generator testing. Diesel generators are tested on a staggered basis, every 31 days per Technical Specification 4.8.1.1.2. The valve will be verified operable by observing proper temperatures during diesel testing.

NOTE 3: XRV-225, (Jet Assist), -226 and -227 (Starting Air): The starting air valves are installed on parallel air supply lines to a diesel generator. The valves are not equipped with position indication devices to directly measure valve stroke times. Stroke timing is verified by measuring diesel starting times (Technical Specification acceptance ten seconds or less). The valves on a staggered basis are valved out one at a time to verify the operability of the opposite valve during diesel testing. The consistent compliance of the diesel generator start times (typically seven to nine seconds) demonstrates the valve performance. Position indication is confirmed during the above testing when only one starting air train is used to start the diesel generators. The starting air valves do not require or have a fail safe position because there is a redundant starting air supply system for each diesel generator. (continued)

DONALD C. COOK NUCLEAR PLANT

VALVE TEST PROGRAM

RELIEF REQUEST NOTES

Flow Diagram No: 2-5151D-27

Revision No: 2

Date: 8-31-87

(Note 3 continued)

Similarly, stroke timing of jet assist valve XRV-225 is verified by measuring diesel starting times. The jet assist valve's fail safe position is "closed". The jet assist valve's function is to facilitate diesel generator quick start. Its failure to open upon a diesel start signal could cause the effected diesel generator to achieve operating conditions more slowly. Based on single failure criteria, the other diesel will start within the required ten seconds, thereby meeting system requirements. If this valve were to fail open, (open is the position to assist diesel generator starting) then the starting air reservoirs would bleed down, rendering that diesel generator inoperable. Therefore, in order to assure the starting air supply, the fail safe mode for this valve is closed.

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DONALD C. COOK NUCLEAR PLANT

VALVE TEST PROGRAM

RELIEF REQUEST NOTES

Flow Diagram No: 12-5115A-41 - Unit-2

Revision No: 2

Date: 8-31-87

NOTE 1: See "Attachment-A" for permissible seat leakage values.

DONALD C. COOK NUCLEAR PLANT

VALVE TEST PROGRAM

RELIEF REQUEST NOTES

Flow Diagram No: 12-5120B-22 - Unit-2

Revision No: 2

Date: 8-31-87

NOTE 1: PA-342: This check valve is located in the maintenance air supply line into the containment. The valve cannot be tested during power operation and cold shutdown because: 1) this line is generally isolated by removing a spool piece and inserting a blind flange, and 2) the valve and test connections are located inside the containment. The valve is not equipped with position indication. Due to the plant design, the only method available to verify the valve closure is leak testing. The valve will be verified closed during the seat leakage testing per Appendix "J" at a refueling frequency.

2

NOTE 2: See "Attachment-A" for permissible seat leakage values.

NOTE 3: XCR-100, -101, -102, -103: These air operated containment isolation valves located in the control air supply lines to the containment. These valves cannot be full stroke tested during power operation without causing a loss of containment control air. Testing of these valves can potentially cause: 1) disruption of air flow to air operated valves in the containment; as a result, they would go to their fail safe position, e.g., close position for containment isolation valves, 2) systems from performing their design function, i.e, termination of system flow and change in RCS pressure and temperature, and 3) challenge to system safeguard protection which may result in a unit trip. The valves will be full stroke exercised at a cold shutdown frequency.

2

DONALD C. COOK NUCLEAR PLANT

VALVE TEST PROGRAM

RELIEF REQUEST NOTES

Flow Diagram No: 12-5131-19 - Unit-2

Revision No: 2

Date: 8-31-87

NOTE 1: CS-427S: This valve is in the emergency boration path.
This valve is not normally operated at power because of
the resulting large reactivity insertion. The valve will
be full stroke exercised at a cold shutdown frequency.

DONALD C. COOK NUCLEAR PLANT

VALVE TEST PROGRAM

RELIEF REQUEST NOTES

Flow Diagram No: 12-5136-25

Revision No: 2

Date: 8-31-87

NOTE 1: See "Attachment-A" for permissible seat leakage values.

DONALD C. COOK NUCLEAR PLANT

VALVE TEST PROGRAM

RELIEF REQUEST NOTES

Flow Diagram No: 12-5137A-21

Revision No: 2

Date: 8-31-87

NOTE 1: See "Attachment-A" for permissible seat leakage values.

NOTE 2: N-160: This containment isolation check valve is located in the Nitrogen Supply line to Reactor Coolant Drain Tank. This valve cannot be part or full stroke exercised due to lack of sufficient differential pressure to back seat the valve during power operation or cold shutdown. Due to the plant design, the only method available to verify the valve closure is leak testing. The valve is not equipped with position indication. This valve will be tested during seat leakage testing per Appendix "J" at a refueling frequency.

2

DONALD C. COOK NUCLEAR PLANT

VALVE TEST PROGRAM

RELIEF REQUEST NOTES

Flow Diagram No: 12-5141C-8

Revision No: 2

Date: 8-31-87

NOTE 1: See "Attachment-A" for permissible seat leakage values.

DONALD C. COOK NUCLEAR PLANT

VALVE TEST PROGRAM

RELIEF REQUEST NOTES

Flow Diagram No: 12-5141F-6

Revision No: 2

Date: 8-31-87

NOTE 1: ECR-36: This valve, located in the common sample return line of the lower containment radiation monitors, cannot be part or full stroke exercised during power operation or refueling because closure of the valve would isolate both radiation monitors, which are required to be operable (Technical Specification Table 3.3-6) during power operation (Mode 1 through 4) and refueling (Mode 6). The valve will be full stroke exercised at a cold shutdown frequency.

2

NOTE 2: See "Attachment-A" for permissible leakage values.

NOTE 3: SM-1: This containment isolation check valve for the containment radiation monitors' sample return cannot be full or part stroke exercised during power operation because these monitors are required to be operable in Modes 1, 2, 3, 4, and 6. The valve is not equipped with position indication. The valve is located in the open ended return line inside the containment. The only method available to verify the valve closure is leak testing. The valve will be tested during seat leakage testing per Appendix "J" at a refueling frequency.

2

DONALD C. COOK NUCLEAR PLANT

ASME SECTION XI VALVE TEST PROGRAM FOR UNIT #2

ATTACHMENT-A

Revision No: 2

Date: 8-31-87

1. CONTAINMENT ISOLATION VALVES (Category A or AC)

Testing Method: (SLT-2) Seat leakage test the valve in accordance with 10CFR50, Appendix J, in lieu of ASME Code Section XI except for paragraphs IWV-3426 and IWV-3427.

<u>Valve No</u>	<u>Flow Diagram</u>	<u>Size</u>	<u>Type</u>	<u>Permissible Leakage Values (SCCM)</u>
WCR-920,-922	5114A	3	DA	900
WCR-921,-923	5114A	3	DA	900
WCR-932,-934	5114A	3	DA	900
WCR-933,-935	5114A	3	DA	900
WCR-941,-945	5114A	3	DA	900
WCR-944,-948	5114A	3	DA	900
WCR-951,-955	5114A	3	DA	900
WCR-954,-958	5114A	3	DA	900
WCR-924,-926	5114A	3	DA	900
WCR-925,-927	5114A	3	DA	900
WCR-928,-930	5114A	3	DA	900



DONALD C. COOK NUCLEAR PLANT

ASME SECTION XI VALVE TEST PROGRAM FOR UNIT #2

ATTACHMENT-A

Revision No: 2

Date: 8-31-87

<u>Valve No.</u>	<u>Flow Diagram</u>	<u>Size</u>	<u>Type</u>	<u>Permissible Leakage Values (SCCM)</u>
WCR-929,-931	5114A	3	DA	900
WCR-942,-946	5114A	3	DA	900
WCR-952,-956	5114A	3	DA	900
WCR-943,-947	5114A	3	DA	900
WCR-953,-957	5114A	3	DA	900
WCR-960,-962	5114A	2	DA	750
WCR-961,-963	5114A	2	DA	750
WCR-964,-966	5114A	2	DA	750
WCR-965,-967	5114A	2	DA	750
ECR-10,-20	5141B	0.50	GL	750
ECR-11,-21	5141B	0.50	GL	750
ECR-12,-22	5141B	0.50	GL	750
ECR-13,-23	5141B	0.50	GL	750
ECR-14,-24	5141B	0.50	GL	750
ECR-15,-25	5141B	0.50	GL	750

DONALD C. COOK NUCLEAR PLANT

ASME SECTION XI VALVE TEST PROGRAM FOR UNIT #2

ATTACHMENT-A

Revision No: 2

Date: 8-31-87

<u>Valve No.</u>	<u>Flow Diagram</u>	<u>Size</u>	<u>Type</u>	<u>Permissible Leakage Values (SCCM)</u>
ECR-16,-26	5141B	0.50	GL	750
ECR-17,-27	5141B	0.50	GL	750
ECR-18,-28	5141B	0.50	GL	750
ECR-19,-29	5141B	0.50	GL	750
CS-442-1	5128A	2	CK	750
CS-442-2	5128A	2	CK	750
CS-442-3	5128A	2	CK	750
CS-442-4	5128A	2	CK	750
SI-189	5128A	4	CK	1200
SM-1	5141F	1	CK	750
N-102	5143	1	CK	750
N-159	5128A	0.75	CK	750
PW-275	5128A	3	CK	900
CS-321	5129	3	CK	1800
VCR-10,-11	5146B	4	DA	1200

DONALD C. COOK NUCLEAR PLANT

ASME SECTION XI VALVE TEST PROGRAM FOR UNIT #2

ATTACHMENT-A

Revision No: 2

Date: 8-31-87

<u>Valve No.</u>	<u>Flow Diagram</u>	<u>Size</u>	<u>Type</u>	<u>Permissible Leakage Values (SCCM)</u>
VCR-20,-21	5146B	4	DA	1200
DCR-203,-207	5137A	1	DA, GL	750
N-160, DCR-201	5137A	1	CK, DA	1125
DCR-610,-611	5137A	2.50	DA	750
DCR-620,-621	5137A	1	DA	750
DCR-205,-206	5137A	4	DA	1200
DCR-600,-601	5124	3	DA	900
QCR-300,-301	5129	2	GL	750
QCM-250,-350	5129A	4	GA	1200
QCR-919,-920	5115A	2	DA	750
SF-152,-154	5136	2.50	DA, GL	750
SF-159,-160	5137A	3	DA	900

DONALD C. COOK NUCLEAR PLANT

ASME SECTION XI VALVE TEST PROGRAM FOR UNIT #2

ATTACHMENT-A

Revision No: 2

Date: 8-31-87

<u>Valve No.</u>	<u>Flow Diagram</u>	<u>Size</u>	<u>Type</u>	<u>Permissible Leakage Values (SCCM)</u>
NCR-105,-106	5141	0.50	GL	750
NCR-107,-108	5141	0.50	GL	750
NCR-109,-110	5141	0.50	GL	750
RCR-100,-101	5128A	0.375	GL	750
DCR-202,-204	5137A	0.75	DA	750
ICR-5,-6	5141	0.50	GL	750
ECR-33,-35	5141F	0.75, 2	GL, DA	750
ICM-260	5142	4	GA (DD) *	2 600
ICM-265	5142	4	GA (DD) *	
ECR-31,-32	5141F	1	GL	750
XCR-100,-101	5120B	1	GL	750
XCR-102,-103	5120B	1	GL	750
GCR-301	5128A	0.75	DA	375
GCR-314	5143	1	GL	375

* Double Discs | 2

DONALD C. COOK NUCLEAR PLANT

ASME SECTION XI VALVE TEST PROGRAM FOR UNIT #2

ATTACHMENT-A

Revision No: 2

Date: 8-31-87

<u>Valve No.</u>	<u>Flow Diagram</u>	<u>Size</u>	<u>Type</u>	<u>Permissible Leakage Values (SCCM)</u>
SI-171,-172,-194	5143	0.75	GL	1125
NCR-252	5128A	3	GL	450
CCR-460,-462	5135	3	GL	900
CCR-457,CCW-135	5135	2,2.50	GL,CK	1125
CCR-455,-456	5135	2	GL	750
SM-4,-6	5147A	0.50	GL	750
ICM-251	5142	4	GA (DD) *	2 600
ICM-250	5142	4	GA (DD) *	
CA-181S	5145	0.50	CK	750
CA-181N	5145	0.50	CK	750
SM-8,-10	5147A	0.50	ND	750
CCW-243-25	5135B	1	CK	750
CCW-244-25	5135B	1	CK	750

* Double Discs | 2

DONALD C. COOK NUCLEAR PLANT

ASME SECTION XI VALVE TEST PROGRAM FOR UNIT #2

ATTACHMENT-A

Revision No: 2

Date: 8-31-87

<u>Valve No.</u>	<u>Flow Diagram</u>	<u>Size</u>	<u>Type</u>	<u>Permissible Leakage Values (SCCM)</u>
CCW-243-72	5135B	1	CK	750
CCW-244-72	5135B	1	CK	750
CCM-430	5135B	1.50	GL	375
CCM-431	5135B	1.50	GL	375
CCR-440	513B	1.50	GL	375
CCR-441	5135B	1.50	GL	375
CCM-432	5135B	1.50	GL	375
CCM-433	5135B	1.50	GL	375
R-156	5146B	0.375	CK	750
R-157	5146B	0.375	CK	750
NS-357	5124	0.50	CK	750
ECR-496,-497	5141C	0.50	GL	750
ECR-416	5141C	0.50	GL	375
ECR-417	5141C	0.50	GL	375
ECR-535	5141C	0.50	GL	375

DONALD C. COOK NUCLEAR PLANT

ASME SECTION XI VALVE TEST PROGRAM FOR UNIT #2

ATTACHMENT-A

Revision No: 2

Date: 8-31-87

<u>Valve No.</u>	<u>Flow Diagram</u>	<u>Size</u>	<u>Type</u>	<u>Permissible Leakage Values (SCCM)</u>
ECR-536	5141C	0.50	GL	375
ECR-36	5141F	2	DA	375
PCR-40	5120B	2	GA	375
PA-342	5120B	2	CK	750
NS-283	5141D	0.50	CK	750
NPX-151	5128A	0.50	GL	375
WCR-900,-902	5114A	6	DA	1800
WCR-901,-903	5114A	6	DA	1800
WCR-912,-914	5114A	6	DA	1800
WCR-913,-915	5114A	6	DA	1800
WCR-904,-906	5114A	6	DA	1800
WCR-905,-907	5114A	6	DA	1800
WCR-908,-910	5114A	6	DA	1800
WCR-909,-911	5114A	6	DA	1800
VCR-101,-201	5147A	14	BF	4200
VCR-102,-202	5147A	14	BF	4200

DONALD C. COOK NUCLEAR PLANT

ASME SECTION XI VALVE TEST PROGRAM FOR UNIT #2

ATTACHMENT-A

Revision No: 2

Date: 8-31-87

<u>Valve No.</u>	<u>Flow Diagram</u>	<u>Size</u>	<u>Type</u>	<u>Permissible Leakage Values (SCCM)</u>
VCR-103,-203	5147A	24	BF	7200
VCR-104,-204	5147A	30	BF	9000
VCR-105,-205	5147A	30	BF	9000
VCR-106,-206	5147A	24	BF	7200
VCR-107,-207	5147A	14	BF	4200
ICM-305	5143	18	GA(DD) *	2700
ICM-306	5143	18	GA(DD) *	2700
CCM-452,-454,-458	5135	8,4,8	BF, GL, BF	3000
CCM-451,-453,-459	5135	8,4,8	BF, GL, BF	3000

* Double Discs | 2

DONALD C. COOK NUCLEAR PLANT

ASME SECTION XI VALVE TEST PROGRAM FOR UNIT #2

ATTACHMENT-A

Revision No: 2

Date: 8-31-87

2. CONTAINMENT SPRAY VALVES (Category A or AC)

Testing Method: As described in "SLT-2A," Figure 3.

2

<u>Valve No.</u>	<u>Flow Diagram</u>	<u>Size</u>	<u>Type</u>	<u>Permissible Leakage Values (CCM)</u>
CTS-131W	5144	8	CK	3.73
CTS-131E	5144	8	CK	3.00
CTS-127W	5144	6	CK	22.55
CTS-127E	5144	6	CK	21.21
RH-141	5144	8	CK	20.70
RH-142	5144	8	CK	23.00

DONALD C. COOK NUCLEAR PLANT

ASME SECTION XI VALVE TEST PROGRAM FOR UNIT #2

ATTACHMENT-A

Revision No: 2

Date: 8-31-87

3. PRESSURE ISOLATION VALVES (Category A or AC)

Testing Method: (SLT-1) Seat leakage test the valve per ASME Code Section XI.

<u>Valve No.</u>	<u>Flow Diagram</u>	<u>Size</u>	<u>Type</u>	<u>Permissible Leakage Values (GPM)</u>
CS-299E	5129	4	CK	2.0
CS-299W	5129	4	CK	2.0
SI-152-N	5143	4	CK	5.0
SI-152-S	5143	4	CK	5.0
ICM-129	5143	14	GA(DD) * 2	10.0
SI-161-L1, -L4	5143	6	CK	10.0
SI-161-L2, -L3	5143	6	CK	10.0
SI-170-L1	5143	10	CK	5.0
SI-170-L2	5143	10	CK	1.0
SI-170-L3	5143	10	CK	1.0
SI-170-L4	5143	10	CK	5.0
SI-158-L1, -L4	5143	6	CK	10.0

* Double Discs

| 2

DONALD C. COOK NUCLEAR PLANT

ASME SECTION XI VALVE TEST PROGRAM FOR UNIT #2

ATTACHMENT-A

Revision No: 2

Date: 8-31-87

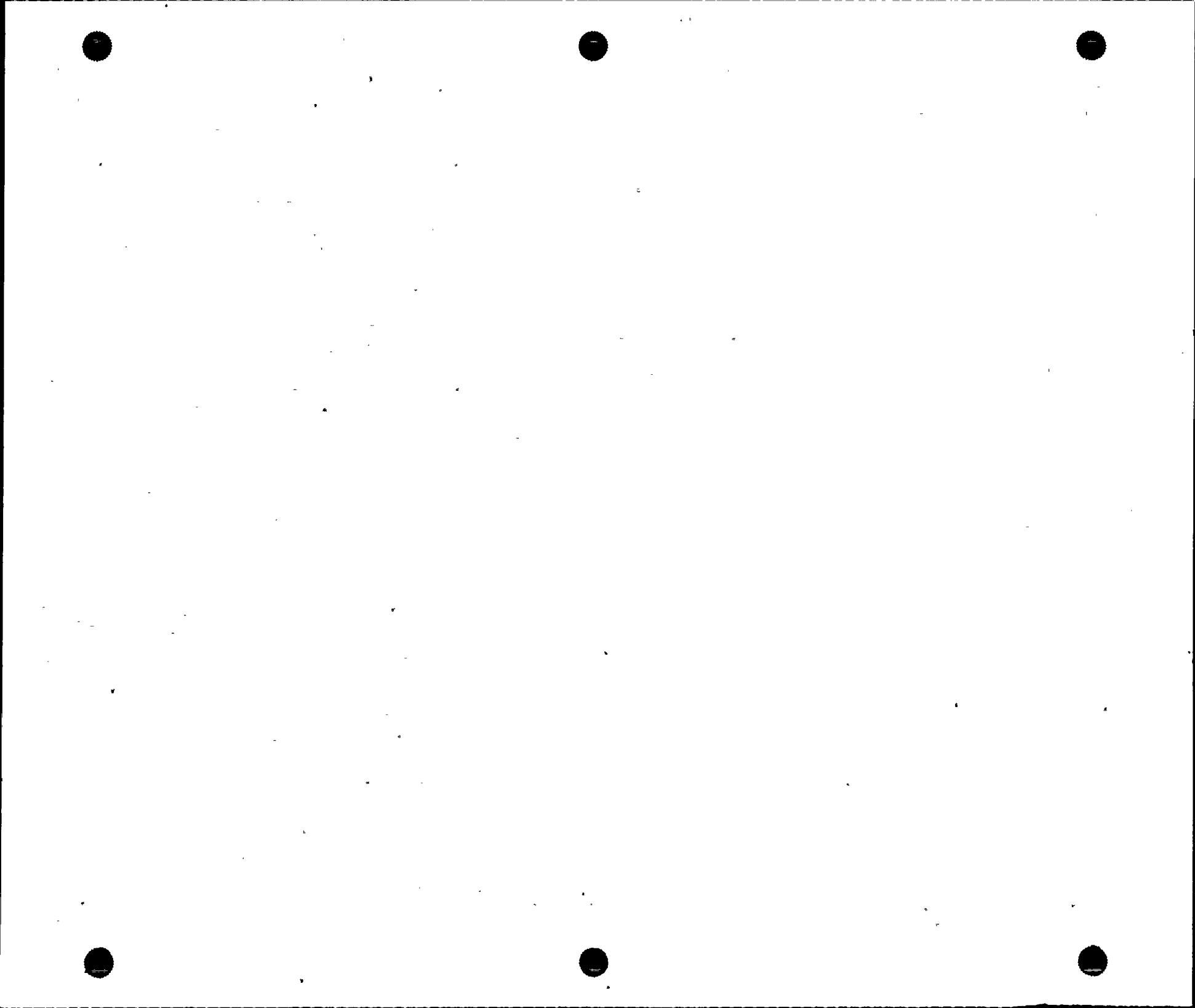
<u>Valve No.</u>	<u>Flow Diagram</u>	<u>Size</u>	<u>Type</u>	<u>Permissible Leakage Values (GPM).</u>
SI-158-L2, -L3	5143	6	CK	10.0
SI-151-E	5143	8	CK	5.0
SI-151-W	5143	8	CK	5.0
SI-166-L1	5143	10	CK	5.0
SI-166-L2	5143	10	CK	5.0
SI-166-L3	5143	10	CK	5.0
SI-166-L4	5143	10	CK	5.0
RH-133, -134	5143	8	CK	1.0

DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLOW DIAGRAM: 2-51510-27

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM-NAME:-- EMERGENCY-DIESEL-GENERATOR "CD"

VALVE		VALVE POSITION		ASME SECTION XI											
NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD	A/P	CAT	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)	
2-SV-120-CD	0	REL	0.25	SA	H/2	C	0	3	A	C	TF-1	TF-1	R	NO	
2-SV-139-CD	0	REL	1	SA	B/2	C	0	3	A	C	TF-1	TF-1	R	NO	
2-SV-61-CD	0	REL	1	SA	A/8	C	0	3	A	C	TF-1	TF-1	R	NO	
2-SV-78-CD1	0	REL	1	SA	E/3	C	0	3	A	C	TF-1	TF-1	R	NO	
2-SV-78-CD2	0	REL	1	SA	D/3	C	0	3	A	C	TF-1	TF-1	R	NO	
2-SV-79-CD1	0	REL	0.5	SA	E/1	C	0	3	A	C	TF-1	TF-1	R	NO	
2-SV-79-CD2	0	REL	0.5	SA	E/1	C	0	3	A	C	TF-1	TF-1	R	NO	
2-XRV-225	0	GA	1	A	B/3	C	0	3	A	B	EF-1 EF-7 ET-010	EF-1 EF-7 NOTE 3	P P P	NO NO YES, NOTE 3	
2-XRV-226	0	GL	3	A	B/4	C	0	3	A	B	EF-1 ET-010	EF-1 NOTE 3	P P	YES, NOTE 3 YES, NOTE 3	
2-XRV-227	0	GL	3	A	B/4	C	0	3	A	B	EF-1 ET-010	EF-1 NOTE 3	P P	YES, NOTE 3 YES, NOTE 3	



DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLOW DIAGRAM: 2-5151D-27

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM NAME: EMERGENCY DIESEL GENERATOR "CD"

VALVE				VALVE POSITION				ASME SECTION XI							
NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD A/P CL	A	P	CAT	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)
2-DG-102C	1	CK	1.5	SA	H/4	0	0/C	3	A	C	CF-1	CF-1	CF-1	P	NO, NOTE 1
2-DG-104C	1	CK	1.5	SA	F/4	0	0/C	3	A	C	CF-1	CF-1	CF-1	P	NO, NOTE 1
2-DG-128C	0	CK	1	SA	C/3	C	0	3	A	C	CF-1	CF-1	CF-1	P	NO, NOTE 1
2-DG-130C	0	CK	1	SA	C/3	C	0	3	A	C	CF-1	CF-1	CF-1	P	NO, NOTE 1
2-DG-132C	0	CK	3	SA	B/4	C	0	3	A	C	CF-1	CF-1	CF-1	P	NO, NOTE 1
2-DG-134C	0	CK	3	SA	B/4	C	0	3	A	C	CF-1	CF-1	CF-1	P	NO, NOTE 1
2-DG-140C	0	CK	0.5	SA	F/1	C	0/C	3	A	C	CF-1	CF-1	CF-1	P	NO
2-DG-142C	0	CK	0.5	SA	F/1	C	0/C	3	A	C	CF-1	CF-1	CF-1	P	NO
2-DG-146C	0	CK	2	SA	A/9	0	C	3	A	C	CF-1	CF-1	CF-1	P	NO, NOTE 1
2-DG-152C	0	CK	4	SA	D/9	C	0	3	A	C	CF-1	CF-1	CF-1	P	NO, NOTE 1
2-DG-154C	0	CK	4	SA	C/9	C	0	3	A	C	CF-1	CF-1	CF-1	P	NO, NOTE 1
2-QT-132-2CD	0	3M	6	SA	E/8	0	0	3	A	B	EF-1	NOTE 1	NOTE 1	P	NO, NOTE 2



DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLOW DIAGRAM: 2-5151C-26

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM NAME: EMERGENCY-DIESEL GENERATOR

VALVE

VALVE POSITION

ASME SECTION XI

NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD CL	A/P	CAT	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)
2-DL-114C	0	CK	1.5	SA	B/9	0	C	3	A	C	CF-1	CF-1	P	NO, NOTE 1
2-DL-116C	0	CK	1.5	SA	B/9	C	0	3	A	C	CF-1	CF-1	P	NO, NOTE 1
2-DL-132C	0	CK	1	SA	F/9	0	C	3	A	C	CF-1	CF-1	P	NO, NOTE 1
2-DL-158C	0	CK	6	SA	G/5	C	0	3	A	C	CF-1	CF-1	P	NO, NOTE 1
2-QT-114-2CD	0	3M	6	SA	H/5	0	0	3	A	B	EF-1	NOTE 2	P	NO, NOTE 2



DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLOID DIAGRAM: 2-5151B-27

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM NAME: EMERGENCY DIESEL GENERATOR

VALVE				VALVE POSITION				ASME SECTION XI						
NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD A/P CL	CAT	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)	
2-SV-120-AB	0	REL	0.25	SA	G/2	C	0	3	A	C	TF-1	TF-1	R NO	
2-SV-139-AB	0	REL	1	SA	B/2	C	0	3	A	C	TF-1	TF-1	R NO	
2-SV-61-AB	0	REL	1	SA	A/8	C	0	3	A	C	TF-1	TF-1	R NO	
2-SV-78-AB1	0	REL	1	SA	E/3	C	0	3	A	C	TF-1	TF-1	R NO	
2-SV-78-AB2	0	REL	1	SA	D/3	C	0	3	A	C	TF-1	TF-1	R NO	
2-SV-79-AB1	0	REL	0.5	SA	E/1	C	0	3	A	C	TF-1	TF-1	R NO	
2-SV-79-AB2	0	REL	0.5	SA	E/1	C	0	3	A	C	TF-1	TF-1	R NO	
2-XRV-220	0	GA	1	A	B/3	C	0	3	A	B	EF-1 EF-7 ET-010	EF-1 EF-7 NOTE 3	P NO P NO P YES, NOTE 3	
2-XRV-221	0	GL	3	A	B/4	C	0	3	A	B	EF-1 ET-010	EF-1 NOTE 3	P NO P YES, NOTE 3	
2-XRV-222	0	GL	3	A	B/4	C	0	3	A	B	EF-1 ET-010	EF-1 NOTE 3	P NO P YES, NOTE 3	

DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLOW DIAGRAM: 2-5151B-27

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM-NAME: EMERGENCY-DIESEL GENERATOR

VALVE				VALVE POSITION				ASME SECTION XI						
NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD CL	A/P	CAT	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)
2-DG-102A	0	CK	1.5	SA	H/4	0	O/C	3	A	C	CF-1	CF-1	P	NO, NOTE 1
2-DG-104A	0	CK	1.5	SA	G/4	0	O/C	3	A	C	CF-1	CF-1	P	NO, NOTE 1
2-DG-128A	0	CK	1	SA	C/4	C	0	3	A	C	CF-1	CF-1	P	NO, NOTE 1
2-DG-130A	0	CK	1	SA	C/4	C	0	3	A	C	CF-1	CF-1	P	NO, NOTE 1
2-DG-132A	0	CK	3	SA	B/4	C	0	3	A	C	CF-1	CF-1	P	NO, NOTE 1
2-DG-134A	0	CK	3	SA	B/4	C	0	3	A	C	CF-1	CF-1	P	NO, NOTE 1
2-DG-140A	0	CK	0.5	SA	F/1	C	O/C	3	A	C	CF-1	CF-1	P	NO
2-DG-142A	0	CK	0.5	SA	F/1	C	O/C	3	A	C	CF-1	CF-1	P	NO
2-DG-146A	0	CK	2	SA	A/8	0	C	3	A	C	CF-1	CF-1	P	NO, NOTE 1
2-DG-152A	0	CK	4	SA	D/8	C	0	3	A	C	CF-1	CF-1	P	NO, NOTE 1
2-DG-154A	0	CK	4	SA	C/8	C	0	3	A	C	CF-1	CF-1	P	NO, NOTE 1
2-QT-132-2AB	0	3W	6	SA	E/8	0	0	3	A	B	EF-1	NOTE 2	P	NO, NOTE 2

DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLOW DIAGRAM: 2-5151A-26

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM NAME: EMERGENCY DIESEL GENERATOR

VALVE				VALVE POSITION				ASME SECTION XI						
NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD A/P CL	CAT	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)	
2-DF-108A	0	CK	1.5	SA	L/3	C	0	3	A C	CF-1	CF-1	P	NO, NOTE 1	
2-DF-109A	0	CK	1.5	SA	K/3	C	0	3	A C	CF-1	CF-1	P	NO, NOTE 1	
2-DF-114A	0	CK	1.5	SA	J/3	C	0	3	A C	CF-1	CF-1	P	NO, NOTE 1	
2-DF-115A	0	CK	1.5	SA	H/3	C	0	3	A C	CF-1	CF-1	P	NO, NOTE 1	
2-DL-114A	0	CK	1.5	SA	B/9	0	C	3	A C	CF-1	CF-1	P	NO, NOTE 1	
2-DL-116A	0	CK	1.5	SA	B/9	C	0	3	A C	CF-1	CF-1	P	NO, NOTE 1	
2-DL-132A	0	CK	1	SA	F/9	0	C	3	A C	CF-1	CF-1	P	NO, NOTE 1	
2-DL-158A	0	CK	6	SA	G/6	C	0	3	A C	CF-1	CF-1	P	NO, NOTE 1	
2-QT-114-2AB	0	3H	6	SA	H/5	0	0	3	A B	EF-1	NOTE 2	P	NO, NOTE 2	



DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLOW DIAGRAM: 2-5149-23

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM-NAME: CONTROL ROOM VENTILATION

VALVE				VALVE POSITION				ASME SECTION XI						
NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD	A/P	CAT	PRIN TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)
2-DN-163-N	0	GA	2.5	M	F/2	0	O/C	3	A	B	EF-1	EF-1	P	NO
2-DN-163-S	0	GA	2.5	M	G/2	0	O/C	3	A	B	EF-1	EF-1	P	NO
2-DN-166-N	0	GA	2.5	M	E/5	0	O/C	3	A	B	EF-1	EF-1	P	NO
2-DN-166-S	0	GA	2.5	M	J/5	0	O/C	3	A	B	EF-1	EF-1	P	NO
2-VRV-315	0	3H	2.5	A	F/5	0	0	3	A	B	EF-1	EF-1	P	NO, NOTE 1
											EF-7	EF-7	P	NO, NOTE 1
											ET-NA	NOTE 1	-	YES, NOTE 1
2-VRV-325	0	3H	2.5	A	G/5	0	0	3	A	B	EF-1	EF-1	P	NO, NOTE 1
											EF-7	EF-7	P	NO, NOTE 1
											ET-NA	NOTE 1	-	YES, NOTE 1



DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLOW DIAGRAM: 2-5147A-35

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM-NAME: CONTAINMENT-VENTILATION

VALVE				VALVE POSITION				ASME SECTION XI						
NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD ICL	A/P	CAT	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)
2-VCR-203	0	BF	24	A	J/5	C	C	2	P	A	EF-1	EF-1	P	NO
											EF-5	EF-5	-	NO
											EF-7	EF-7	P	NO
											ET-005	ET-005	P	NO
											SLT-1	SLT-2	R	YES, NOTE 1
2-VCR-204	0	BF	30	A	J/6	C	C	2	P	A	EF-1	EF-1	P	NO
											EF-5	EF-5	-	NO
											EF-7	EF-7	P	NO
											ET-005	ET-005	P	NO
											SLT-1	SLT-2	R	YES, NOTE 1
2-VCR-205	0	BF	30	A	J/3	C	C	2	P	A	EF-1	EF-1	P	NO
											EF-5	EF-5	-	NO
											EF-7	EF-7	P	NO
											ET-005	ET-005	P	NO
											SLT-1	SLT-2	R	YES, NOTE 1
2-VCR-206	0	BF	24	A	J/3	C	C	2	P	A	EF-1	EF-1	P	NO
											EF-5	EF-5	-	NO
											EF-7	EF-7	P	NO
											ET-005	ET-005	P	NO
											SLT-1	SLT-2	R	YES, NOTE 1
2-VCR-207	0	BF	14	A	J/4	O/C	C	2	A	A	EF-1	EF-1	P	NO
											EF-5	EF-5	-	NO
											EF-7	EF-7	P	NO
											ET-005	ET-005	P	NO
											SLT-1	SLT-2	R	YES, NOTE 1



DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLOW DIAGRAM: 2-5147A-35

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM-NAME: CONTAINMENT-VENTILATION

VALVE				VALVE POSITION				ASME SECTION XI						
NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD CL	A/P	CAT	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)
2-VCR-105	0	BF	30	A	J/3	C	C	2	P	A	EF-1 EF-5 EF-7 ET-005 SLT-1	EF-1 EF-5 EF-7 ET-005 SLT-2	P - P P R	NO NO NO NO YES, NOTE 1
2-VCR-106	0	BF	24	A	J/3	C	C	2	P	A	EF-1 EF-5 EF-7 ET-005 SLT-1	EF-1 EF-5 EF-7 ET-005 SLT-2	P - P P R	NO NO NO NO YES, NOTE 1
2-VCR-107	0	BF	14	A	J/4	O/C	C	2	A	A	EF-1 EF-5 EF-7 ET-005 SLT-1	EF-1 EF-5 EF-7 ET-005 SLT-2	P - P P R	NO NO NO NO YES, NOTE 1
2-VCR-201	0	BF	14	A	J/8	C	C	2	P	A	EF-1 EF-5 EF-7 ET-005 SLT-1	EF-1 EF-5 EF-7 ET-005 SLT-2	P - P P R	NO NO NO NO YES, NOTE 1
2-VCR-202	0	BF	14	A	J/9	C	C	2	P	A	EF-1 EF-5 EF-7 ET-005 SLT-1	EF-1 EF-5 EF-7 ET-005 SLT-2	P - P P R	NO NO NO NO YES, NOTE 1

DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLOW DIAGRAM: 2-5147A-35

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM NAME: CONTAINMENT VENTILATION

VALVE				VALVE POSITION				ASME SECTION XI						
NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD	A/P	CAT	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)
2-SM-10	0	GA	0.5	M	A/4	C	C	2	P	A	SLT-1	SLT-2	R	YES, NOTE 1
2-SM-4	0	GA	0.5	M	A/2	C	C	2	P	A	SLT-1	SLT-2	R	YES, NOTE 1
2-SM-6	0	GA	0.5	M	A/2	C	C	2	P	A	SLT-1	SLT-2	R	YES, NOTE 1
2-SM-8	0	GA	0.5	M	A/4	C	C	2	P	A	SLT-1	SLT-2	R	YES, NOTE 1
2-VCR-101	0	BF	14	A	J/8	C	C	2	P	A	EF-1 EF-5 EF-7 ET-005 SLT-1	EF-1 EF-5 EF-7 ET-005 SLT-2	P - P P R	NO NO NO NO YES, NOTE 1
2-VCR-102	0	BF	14	A	J/9	C	C	2	P	A	EF-1 EF-5 EF-7 ET-005 SLT-1	EF-1 EF-5 EF-7 ET-005 SLT-2	P - P P R	NO NO NO NO YES, NOTE 1
2-VCR-103	0	BF	24	A	J/5	C	C	2	P	A	EF-1 EF-5 EF-7 ET-005 SLT-1	EF-1 EF-5 EF-7 ET-005 SLT-2	P - P P R	NO NO NO NO YES, NOTE 1
2-VCR-104	0	BF	30	A	J/6	C	C	2	P	A	EF-1 EF-5 EF-7 ET-004 SLT-1	EF-1 EF-5 EF-7 ET-004 SLT-2	P - P P R	NO NO NO NO YES, NOTE 1



DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLOW DIAGRAM: 2-5146B-23

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM NAME: ICE CONDENSER REFRIGERATION

VALVE				VALVE POSITION				ASME SECTION XI						
NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD A/P CL	CAT	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)	
2-R-156	2	CK	0.375	SA	L/4	C	O/C	2	A	AC	CF-1 SLT-1	CF-1 SLT-2	R R	YES, NOTE 1 YES, NOTE 2
2-R-157	2	CK	0.375	SA	L/6	C	O/C	2	A	AC	CF-1 SLT-1	CF-1 SLT-2	R R	YES, NOTE 1 YES, NOTE 2
2-VCR-10	0	DA	4	A	M/5	O	C	2	A	A	EF-1 EF-5 EF-7 ET-005 SLT-1	EF-1 EF-5 EF-7 ET-005 SLT-2	P - P P R	NO NO NO NO YES, NOTE 2
2-VCR-11	0	DA	4	A	L/5	O	C	2	A	A	EF-1 EF-5 EF-7 ET-005 SLT-1	EF-1 EF-5 EF-7 ET-005 SLT-2	P - P P R	NO NO NO NO YES, NOTE 2
2-VCR-20	0	DA	4	A	M/7	O	C	2	A	A	EF-1 EF-5 EF-7 ET-005 SLT-1	EF-1 EF-5 EF-7 ET-005 SLT-2	P - P P R	NO NO NO NO YES, NOTE 2
2-VCR-21	0	DA	4	A	L/7	O	C	2	A	A	EF-1 EF-5 EF-7 ET-006 SLT-1	EF-1 EF-5 EF-7 ET-006 SLT-2	P - P P R	NO NO NO NO YES, NOTE 2



DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLOW DIAGRAM: 2-5145-20

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM NAME: CPN/WELD CHANNEL PRESSURIZATION

VALVE		VALVE POSITION		ASME SECTION XI										
NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD CL	A/P	CAT	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)
2-CA-181-N	0	CK	0.5	SA	F/2	C	C	2	P	AC	SLT-1	SLT-2	R	YES, NOTE 1
2-CA-181-S	0	CK	0.5	SA	F/3	C	C	2	P	AC	SLT-1	SLT-2	R	YES, NOTE 1



DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLOW DIAGRAM: 2-5144-29

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM NAME: CONTAINMENT SPRAY

VALVE				VALVE POSITION				ASME SECTION XI						
NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD A/P CL	CAT	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)	
2-IMO-221	0	GA	10	MO	L/8	C	0	2	A B	EF-1 EF-5 ET-066	EF-1 EF-5 ET-066	P - P	NO NO NO	
2-IMO-222	0	GA	2	MO	L/9	0	O/C	2	A B	EF-1 EF-5 ET-030	EF-1 EF-5 ET-030	P - P	NO NO NO	
2-IMO-225	0	GA	12	MO	J/9	0	O/C	2	A B	EF-1 EF-5 ET-074	EF-1 EF-5 ET-074	P - P	NO NO NO	
2-RH-141	0	CK	8	SA	E/3	C	0	2	A AC	CF-1 SLT-1	CF-2 SLT-2A	R R	YES, NOTE 2 YES, NOTE 3	
2-RH-142	0	CK	8	SA	E/3	C	0	2	A AC	CF-1 SLT-1	CF-2 SLT-2A	R R	YES, NOTE 2 YES, NOTE 3	
2-SV-107	0	REL	1	SA	N/5	C	0	2	A C	TF-1	TF-1	R	NO	

DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLOW DIAGRAM: 2-5144-29

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM NAME: CONTAINMENT SPRAY

VALVE				VALVE POSITION				ASME SECTION XI						
NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD	A/P	CAT	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)
2-CTS-138-M	2	CK	12	SA	J/9	C	O/C	2	A	C	CF-1	CF-3	-	YES, NOTE 1
2-IMO-202	0	GA	2.5	MO	M/7	C	O	2	A	B	EF-1 EF-5 ET-018	EF-1 EF-5 ET-018	P - P	NO NO NO
2-IMO-204	0	GA	2.5	MO	M/7	C	O	2	A	B	EF-1 EF-5 ET-018	EF-1 EF-5 ET-018	P - P	NO NO NO
2-IMO-210	0	GA	10	MO	J/8	C	O	2	A	B	EF-1 EF-5 ET-066	EF-1 EF-5 ET-066	P - P	NO NO NO
2-IMO-211	0	GA	10	MO	J/8	C	O	2	A	B	EF-1 EF-5 ET-065	EF-1 EF-5 ET-065	P - P	NO NO NO
2-IMO-212	0	GA	2	MO	H/8	O	O/C	2	A	B	EF-1 EF-5 ET-033	EF-1 EF-5 ET-033	P - P	NO NO NO
2-IMO-215	0	GA	12	MO	G/9	O	O/C	2	A	B	EF-1 EF-5 ET-074	EF-1 EF-5 ET-074	P - P	NO NO NO
2-IMO-220	0	GA	10	MO	L/8	C	O	2	A	B	EF-1 EF-5 ET-065	EF-1 EF-5 ET-065	P - P	NO NO NO

DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SURTARY SHEET - UNIT 2
FLOW DIAGRAM: 2-5144-29

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM-NAME: CONTAINMENT SPRAY

VALVE

VALVE-POSITION

ASME SECTION XI

NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD	A/P	CAT	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)
2-CTS-103-E	2	CK	10	SA	J/9	C	0	2	A	C	CF-1	CF-3	-	YES, NOTE 1
2-CTS-103-W	2	CK	10	SA	L/9	C	0	2	A	C	CF-1	CF-3	-	YES, NOTE 1
2-CTS-109	0	VB	1	SA	M/6	C	0	2	A	C	TF-1	TF-1	R	NO
2-CTS-110	0	VB	1	SA	M/6	C	0	2	A	C	TF-1	TF-1	R	NO
2-CTS-120-E	0	CK	2	SA	H/8	C	0	2	A	C	CF-1	CF-1	P	NO
2-CTS-120-W	0	CK	2	SA	K/8	C	0	2	A	C	CF-1	CF-1	P	NO
2-CTS-127-E	0	CK	6	SA	E/5	C	0	2	A	AC	CF-1 SLT-1	CF-2 SLT-2A	R R	YES, NOTE 2 YES, NOTE 3
2-CTS-127-W	0	CK	6	SA	E/4	C	0	2	A	AC	CF-1 SLT-1	CF-2 SLT-2A	R R	YES, NOTE 2 YES, NOTE 3
2-CTS-131-E	0	CK	8	SA	E/2	C	0	2	A	AC	CF-1 SLT-1	CF-2 SLT-2A	R R	YES, NOTE 2 YES, NOTE 3
2-CTS-131-W	0	CK	8	SA	E/2	C	0	2	A	AC	CF-1 SLT-1	CF-2 SLT-2A	R R	YES, NOTE 2 YES, NOTE 3
2-CTS-138-E	2	CK	12	SA	G/9	C	0/C	2	A	C	CF-1	CF-3	-	YES, NOTE 1

DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLOW DIAGRAM: 2-5143-35

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM-NAME: EMERGENCY-CORE-COOLING - RHR

VALVE				VALVE POSITION				ASME SECTION XI						
NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD CL	A/P	CAT	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)
2-SV-100-3	0	REL	1	SA	D/1	C	0	2	A	C	TF-1	TF-1	R	NO
2-SV-100-4	0	REL	1	SA	D/1	C	0	2	A	C	TF-1	TF-1	R	NO
2-SV-102	0	REL	0.75	SA	E/5	C	0	2	A	C	TF-1	TF-1	R	NO
2-SV-103	0	REL	3	SA	F/8	C	0	2	A	C	TF-1	TF-1	R	NO
2-SV-104E	0	REL	2	SA	G/4	C	0	2	A	C	TF-1	TF-1	R	NO
2-SV-104H	0	REL	2	SA	K/4	C	0	2	A	C	TF-1	TF-1	R	NO
2-SV-105E	0	REL	2	SA	D/9	C	0	2	A	C	TF-1	TF-1	R	NO
2-SV-105H	0	REL	2	SA	D/9	C	0	2	A	C	TF-1	TF-1	R	NO

DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLOW DIAGRAM: 2-5143-35

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM-NAME: EMERGENCY-CORE-COOLING - RHR

VALVE		VALVE POSITION		ASME SECTION-XI										
NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD	A/P	CAT	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)
2-SI-166-L4	0	CK	10	SA	C/4	C	0	1	A	AC	CF-1 SLT-1	CF-2 SLT-1	R R	YES, NOTE 5 NO, NOTE 1
2-SI-170-L1	2	CK	10	SA	A/4	C	0	1	A	AC	CF-1 SLT-1	CF-3 SLT-1	R R	YES, NOTE 13 NO, NOTE 1
2-SI-170-L2	2	CK	10	SA	A/5	C	O/C	1	A	AC	CF-1 SLT-1	CF-3 SLT-1	R R	YES, NOTE 13 NO, NOTE 1
2-SI-170-L3	2	CK	10	SA	A/5	C	O/C	1	A	AC	CF-1 SLT-1	CF-3 SLT-1	R R	YES, NOTE 13 NO, NOTE 1
2-SI-170-L4	2	CK	10	SA	A/4	C	0	1	A	AC	CF-1 SLT-1	CF-3 SLT-1	R R	YES, NOTE 13 NO, NOTE 1
2-SI-171	0	GL	0.75	M	H/6	C	C	2	P	A	SLT-1	SLT-2	R	YES, NOTE 1
2-SI-172	0	GL	0.75	M	H/6	C	C	2	P	A	SLT-1	SLT-2	R	YES, NOTE 1
2-SI-194	0	GL	0.75	M	G/6	C	C	2	P	A	SLT-1	SLT-2	R	YES, NOTE 1
2-SV-100-1	0	REL	1	SA	D/1	C	0	2	A	C	TF-1	TF-1	R	NO
2-SV-100-2	0	REL	1	SA	D/1	C	0	2	A	C	TF-1	TF-1	R	NO

DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLOW DIAGRAM: 2-5143-35

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM-NAME: EMERGENCY CORE COOLING - RHR

VALVE				VALVE POSITION				ASME SECTION XI						
NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD A/P CL	CAT	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)	
2-SI-158-L3	0	CK	6	SA	B/7	C	0	1	A	AC	CF-1 SLT-1	CF-2 SLT-1	C R	YES, NOTE 12 NO, NOTE 1
2-SI-158-L4	0	CK	6	SA	B/7	C	0	1	A	AC	CF-1 SLT-1	CF-2 SLT-1	C R	YES, NOTE 12 NO, NOTE 1
2-SI-161-L1	0	CK	6	SA	B/6	C	0	1	A	AC	CF-1 SLT-1	CF-2 SLT-1	C R	YES, NOTE 6 NO, NOTE 1
2-SI-161-L2	0	CK	6	SA	B/5	C	0	1	A	AC	CF-1 SLT-1	CF-2 SLT-1	C R	YES, NOTE 6 NO, NOTE 1
2-SI-161-L3	0	CK	6	SA	B/5	C	0	1	A	AC	CF-1 SLT-1	CF-2 SLT-1	C R	YES, NOTE 6 NO, NOTE 1
2-SI-161-L4	0	CK	6	SA	B/6	C	0	1	A	AC	CF-1 SLT-1	CF-2 SLT-1	C R	YES, NOTE 6 NO, NOTE 1
2-SI-166-L1	0	CK	10	SA	C/4	C	0	1	A	AC	CF-1 SLT-1	CF-2 SLT-1	R R	YES, NOTE 5 NO, NOTE 1
2-SI-166-L2	0	CK	10	SA	C/4	C	0	1	A	AC	CF-1 SLT-1	CF-2 SLT-1	R R	YES, NOTE 5 NO, NOTE 1
2-SI-166-L3	0	CK	10	SA	C/4	C	0	1	A	AC	CF-1 SLT-1	CF-2 SLT-1	R R	YES, NOTE 5 NO, NOTE 1

DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLOW DIAGRAM: 2-5143-35

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM NAME: EMERGENCY CORE COOLING - RHR

VALVE				VALVE POSITION				ASME SECTION XI						
NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD	A/P	CAT	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)
2-RH-108N	0	CK	8	SA	N/9	C	0	2	A	C	CF-1	CF-3	C	NO, NOTE 7
2-RH-133	0	CK	8	SA	C/5	C	C	1	P	AC	CF-1 SLT-1	CF-2 SLT-1	C R	YES, NOTE 8 NO, NOTE 1
2-RH-134	0	CK	8	SA	C/5	C	C	1	P	AC	CF-1 SLT-1	CF-2 SLT-1	C R	YES, NOTE 8 NO, NOTE 1
2-SI-148	2	CK	12	SA	G/7	C	0	2	A	C	CF-1	CF-3	-	YES, NOTE 9
2-SI-151-E	0	CK	8	SA	D/7	C	0	2	A	AC	CF-1 SLT-1	CF-2 SLT-1	C R	YES, NOTE 10 NO, NOTE 1
2-SI-151-W	0	CK	8	SA	D/7	C	0	2	A	AC	CF-1 SLT-1	CF-2 SLT-1	C R	YES, NOTE 10 NO, NOTE 1
2-SI-152-N	0	CK	4	SA	D/8	C	0	2	A	AC	CF-1 SLT-1	CF-2 SLT-1	R R	YES, NOTE 11 NO, NOTE 1
2-SI-152-S	0	CK	4	SA	D/7	C	0	2	A	AC	CF-1 SLT-1	CF-2 SLT-1	R R	YES, NOTE 11 NO, NOTE 1
2-SI-158-L1	0	CK	6	SA	B/8	C	0	1	A	AC	CF-1 SLT-1	CF-2 SLT-1	C R	YES, NOTE 12 NO, NOTE 1
2-SI-158-L2	0	CK	6	SA	B/7	C	0	1	A	AC	CF-1 SLT-1	CF-2 SLT-1	C R	YES, NOTE 12 NO, NOTE 1

DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLOH DIAGRAM: 2-5143-35

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM NAME: EMERGENCY-CORE-COOLING - RHR

VALVE				VALVE POSITION				ASME SECTION XI									
NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT.	CD	A/P	CAT	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)			
2-IMO-325	0	GA	8	MO	C/7	C	C/O	1	A	B	EF-1 EF-5 ET-028	EF-2 EF-5 ET-028	C - C	YES, NOTE 4 NO NO			
2-IMO-326	0	GA	8	MO	C/7	O	O/C	2	A	B	EF-1 EF-5 ET-028	EF-2 EF-5 ET-028	C - C	YES, NOTE 4 NO NO			
2-IMO-330	0	GA	8	MO	G/4	C	O	2	A	B	EF-1 EF-5 ET-053	EF-1 EF-5 ET-053	P - P	NO NO NO			
2-IMO-331	0	GA	8	MO	L/5	C	O	2	A	B	EF-1 EF-5 ET-051	EF-1 EF-5 ET-051	P - P	NO NO NO			
2-IMO-340	0	GA	8	MO	H/5	C	O	2	A	B	EF-1 EF-5 ET-015	EF-1 EF-5 ET-015	P - P	NO NO NO			
2-IMO-350	0	GA	8	MO	L/5	C	O	2	A	B	EF-1 EF-5 ET-015	EF-1 EF-5 ET-015	P - P	NO NO NO			
2-N-102	2	CK	1	SA	F/5	O/C	C	2	A	AC	CF-1 SLT-1	CF-2 SLT-2	R R	YES, NOTE 14 YES, NOTE 1			
2-RH-108E	0	CK	8	SA	K/9	C	O	2	A	C	CF-1	CF-3	C	NO, NOTE 7			



DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLOW DIAGRAM: 2-5143-35

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM NAME: EMERGENCY CORE COOLING - RHR

VALVE				VALVE POSITION				ASME SECTION XI							
NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD	A/P	CAT	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)	
2-IMO-312	0	GL	2	MO	J/5	0	O/C	2	A	B	EF-1 EF-5 ET-012	EF-1 EF-5 ET-012	P - P	NO NO NO	
2-IMO-314	0	GA	8	MO	K/6	0	O/C	2	A	B	EF-1 EF-5 ET-034	EF-1 EF-5 ET-034	P - P	NO NO NO, NOTE 3	
2-IMO-315	0	GA	8	MO	C/7	C	C/O	1	A	B	EF-1 EF-5 ET-033	EF-2 EF-5 ET-033	C - C	YES, NOTE 4 NO NO	
2-IMO-316	0	GA	8	MO	C/7	0	O/C	2	A	B	EF-1 EF-5 ET-033	EF-2 EF-5 ET-033	C - C	YES, NOTE 4 NO NO	
2-IMO-320	0	GA	14	MO	L/9	0	O/C	2	A	B	EF-1 EF-5 ET-128	EF-1 EF-5 ET-128	P - P	NO NO NO, NOTE 3	
2-IMO-322	0	GL	2	MO	M/5	0	O/C	2	A	B	EF-1 EF-5 ET-011	EF-1 EF-5 ET-011	P - P	NO NO NO	
2-IMO-324	0	GA	8	MO	M/6	0	O/C	2	A	B	EF-1 EF-5 ET-035	EF-1 EF-5 ET-035	P - P	NO NO NO, NOTE 3	

DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLOD DIAGRAM: 2-5143-35

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM NAME: EMERGENCY CORE COOLING - RHR

VALVE				VALVE POSITION				ASME SECTION XI							
NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	ICD A/P CL	CAT	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)		
2-GCR-314	0	GL	1	A	G/2	O	C	2	A	A	EF-1 EF-5 EF-7 ET-003 SLT-1	EF-1 EF-5 EF-7 ET-003 SLT-2	P - P P R	NO NO NO NO YES, NOTE 1	
2-ICH-129	2	GA	14	MO	E/8	C	C	1	P	A	EF-1 EF-5 ET-121 SLT-1	EF-2 EF-5 ET-121 SLT-1	- - - R	YES, NOTE 2 NO NO NO, NOTE 1	
2-ICH-305	0	GA	18	MO	D/9	C	O	2	A	A	EF-1 EF-5 ET-049 SLT-1	EF-1 EF-5 ET-049 SLT-2	P - P R	NO NO NO YES, NOTE 1	
2-ICH-306	0	GA	18	MO	D/9	C	O	2	A	A	EF-1 EF-5 ET-049 SLT-1	EF-1 EF-5 ET-049 SLT-2	P - P R	NO NO NO YES, NOTE 1	
2-IMO-128	2	GA	14	MO	B/8	C	C	1	P	B	EF-1 EF-5 ET-134	EF-2 EF-5 ET-134	- - -	YES, NOTE 2 NO NO	
2-IMO-310	0	GA	14	MO	H/9	O	O/C	2	A	B	EF-1 EF-5 ET-148	EF-1 EF-5 ET-148	P - P	NO NO NO, NOTE 3	



DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLOW DIAGRAM: 2-5142-28

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM NAME: EMERGENCY CORE COOLING - SIS

VALVE		VALVE POSITION		ASME SECTION XI										
NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD	A/P	CAT	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)
2-SI-142-L3	0	CK	1.5	SA	C/2	C	0	1	A	C	CF-1	CF-2	R	YES, NOTE 6
2-SI-142-L4	0	CK	1.5	SA	C/1	C	0	1	A	C	CF-1	CF-2	R	YES, NOTE 6
2-SV-96	0	REL	0.75	SA	J/8	C	0	2	A	C	TF-1	TF-1	R	NO
2-SV-97	0	REL	0.75	SA	J/4	C	0	2	A	C	TF-1	TF-1	R	NO
2-SV-98-N	0	REL	0.75	SA	C/9	C	0	2	A	C	TF-1	TF-1	R	NO
2-SV-98-S	0	REL	0.75	SA	E/8	C	0	2	A	C	TF-1	TF-1	R	NO

DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLOW DIAGRAM: 2-5142-28

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM-NAME: EMERGENCY-CORE-COOLING - SIS

VALVE				VALVE POSITION				ASME SECTION XI							
NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD A/P CL	CAT	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)		
2-IRV-252	0	GL	1	A	J/5	O	C	3	A B	EF-1 EF-5 EF-7 ET-005	EF-1 EF-5 EF-7 ET-005	P - P P	NO NO NO NO		
2-IRV-255	0	GL	1	A	H/6	O	C	2	A B	EF-1 EF-5 EF-7 ET-004	EF-1 EF-5 EF-7 ET-004	P - P P	NO NO NO NO		
2-SI-101	0	CK	8	SA	M/8	C	O	2	A C	CF-1	CF-3	R	YES, NOTE 5		
2-SI-104-N	0	CK	0.75	SA	E/9	C	O	2	A C	CF-1	CF-1	P	NO		
2-SI-104-S	0	CK	0.75	SA	J/9	C	O	2	A C	CF-1	CF-1	P	NO		
2-SI-110-N	0	CK	4	SA	E/9	C	O	2	A C	CF-1	CF-2	R	YES, NOTE 5		
2-SI-110-S	0	CK	4	SA	H/9	C	O	2	A C	CF-1	CF-2	R	YES, NOTE 5		
2-SI-126	0	CK	1	SA	H/6	O	C	2	A C	CF-1	CF-1	P	NO		
2-SI-142-L1	0	CK	1.5	SA	C/1	C	O	1	A C	CF-1	CF-2	R	YES, NOTE 6		
2-SI-142-L2	0	CK	1.5	SA	C/2	C	O	1	A C	CF-1	CF-2	R	YES, NOTE 6		



DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLOW DIAGRAM: 2-5142-28

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM-NAME:--EMERGENCY-CORE-COOLING--SIS

VALVE				VALVE POSITION				ASME SECTION XI									
NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD	A/P	CAT	CL	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)		
2-IMO-262	0	GL	2	MO	L/8	0	O/C	2	A	B		EF-1 EF-5 ET-012	EF-2 EF-5 ET-012	C - C	YES, NOTE 4 NO NO		
2-IMO-263	0	GL	2	MO	L/8	0	O/C	2	A	B		EF-1 EF-5 ET-012	EF-2 EF-5 ET-012	C - C	YES, NOTE 4 NO NO		
2-IMO-270	0	GA	4	MO	E/9	0	O/C	2	A	B		EF-1 EF-5 ET-024	EF-1 EF-5 ET-024	P - P	NO NO NO		
2-IMO-275	0	GA	4	MO	E/8	0	O/C	2	A	B		EF-1 EF-5 ET-020	EF-1 EF-5 ET-020	P - P	NO NO NO		
2-IMO-361	0	GA	4	MO	G/9	C	C/O	2	A	B		EF-1 EF-5 ET-009	EF-1 EF-5 ET-009	P - P	NO NO NO		
2-IMO-362	0	GA	4	MO	G/9	C	C/O	2	A	B		EF-1 EF-5 ET-009	EF-1 EF-5 ET-009	P - P	NO NO NO		
2-IRV-251	0	GL	1	A	H/5	0	C	2	A	B		EF-1 EF-5 EF-7 ET-004	EF-1 EF-5 EF-7 ET-004	P - P P	NO NO NO NO		

DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLOW DIAGRAM: 2-5142-28

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM-NAME: EMERGENCY-CORE-COOLING - SIS

VALVE				VALVE POSITION				ASME SECTION XI									
NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD	A/P	CAT	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)			
2-ICM-250	0	GA	4	MO	H/2	C	O/C	2	A	A	EF-1	EF-2	C	YES, NOTE 1			
											EF-5	EF-5	-	NO			
											ET-012	ET-012	C	NO			
											SLT-1	SLT-2	R	YES, NOTE 2			
2-ICM-251	0	GA	4	MO	H/3	C	O/C	2	A	A	EF-1	EF-2	C	YES, NOTE 1			
											EF-5	EF-5	-	NO			
											ET-012	ET-012	C	NO			
											SLT-1	SLT-2	R	YES, NOTE 2			
2-ICM-260	0	GA	4	MO	C/9	O	O/C	2	A	A	EF-1	EF-1	P	NO			
											EF-5	EF-5	-	NO			
											ET-012	ET-012	P	NO			
											SLT-1	SLT-2	R	YES, NOTE 2			
2-ICM-265	0	GA	4	MO	C/8	O	O/C	2	A	A	EF-1	EF-1	P	NO			
											EF-5	EF-5	-	NO			
											ET-012	ET-012	P	NO			
											SLT-1	SLT-2	R	YES, NOTE 2			
2-IMO-255	0	GA	4	MO	J/7	C	O	2	A	B	EF-1	EF-1	P	NO			
											EF-5	EF-5	-	NO			
											ET-015	ET-015	P	NO			
2-IMO-256	0	GA	4	MO	J/6	C	O	2	A	B	EF-1	EF-1	P	NO			
											EF-5	EF-5	-	NO			
											ET-012	ET-012	P	NO			
2-IMO-261	0	GA	8	MO	M/8	O	O/C	2	A	B	EF-1	EF-2	C	YES, NOTE 3			
											EF-5	EF-5	-	NO			
											ET-015	ET-015	C	NO			



DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLOW DIAGRAM: 2-5141D-8

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM-NAME: PAS-CONTAINMENT-HYDROGEN

VALVE

VALVE POSITION

ASME SECTION XI

NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD A/P CL	CAT	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)
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2-NS-283	2	CK	0.5	SA	C/8	C	O/C	2	A	AC	CF-1 SLT-1	CF-2 SLT-2	R R	YES, NOTE 2 YES, NOTE 1
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DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLOW DIAGRAM: 2-5141D-8

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM-NAME: PAS-CONTAINMENT-HYDROGEN

VALVE

VALVE POSITION

ASME SECTION XI

NUMBER	REV	TYPE	SIZE	ACT	F.D.	POWER	SAFETY	CD A/P	CAT	PRIM TEST	TEST	TEST	RELIEF REQUEST(S)	
				TYPE	COORD	OPER	FUNCT	CL		REQUIRED	PERFORMED	MODE		
2-ECR-25	0	GL	0.5	A	B/1	C	O/C	2	A	A	EF-1 EF-5 EF-7 ETF-002 SLT-1	EF-1 EF-5 EF-7 ETF-002 SLT-2	P - P P R	NO NO NO NO YES, NOTE 1
2-ECR-26	0	GL	0.5	A	B/3	C	O/C	2	A	A	EF-1 EF-5 EF-7 ETF-002 SLT-1	EF-1 EF-5 EF-7 ETF-002 SLT-2	P - P P R	NO NO NO NO YES, NOTE 1
2-ECR-27	0	GL	0.5	A	B/3	C	O/C	2	A	A	EF-1 EF-5 EF-7 ETF-002 SLT-1	EF-1 EF-5 EF-7 ETF-002 SLT-2	P - P P R	NO NO NO NO YES, NOTE 1
2-ECR-28	0	GL	0.5	A	B/4	C	O/C	2	A	A	EF-1 EF-5 EF-7 ETF-002 SLT-1	EF-1 EF-5 EF-7 ETF-002 SLT-2	P - P P R	NO NO NO NO YES, NOTE 1
2-ECR-29	0	GL	0.5	A	B/4	C	O/C	2	A	A	EF-1 EF-5 EF-7 ETF-002 SLT-1	EF-1 EF-5 EF-7 ETF-002 SLT-2	P - P P R	NO NO NO NO YES, NOTE 1

DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLOW DIAGRAM: 2-5141D-8

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM NAME: PAS CONTAINMENT HYDROGEN

VALVE				VALVE POSITION				ASME SECTION XI							
NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD CL	A/P	CAT	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)	
2-ECR-20	0	GL	0.5	A	C/8	C	O/C	2	A	A	EF-1 EF-5 EF-7 ETF-002 SLT-1	EF-1 EF-5 EF-7 ETF-002 SLT-2	P - P P R	NO NO NO NO YES, NOTE 1	
2-ECR-21	0	GL	0.5	A	B/2	C	O/C	2	A	A	EF-1 EF-5 EF-7 ETF-002 SLT-1	EF-1 EF-5 EF-7 ETF-002 SLT-2	P - P P R	NO NO NO NO YES, NOTE 1	
2-ECR-22	0	GL	0.5	A	B/2	C	O/C	2	A	A	EF-1 EF-5 EF-7 ETF-002 SLT-1	EF-1 EF-5 EF-7 ETF-002 SLT-2	P - P P R	NO NO NO NO YES, NOTE 1	
2-ECR-23	0	GL	0.5	A	B/1	C	O/C	2	A	A	EF-1 EF-5 EF-7 ETF-002 SLT-1	EF-1 EF-5 EF-7 ETF-002 SLT-2	P - P P R	NO NO NO NO YES, NOTE 1	
2-ECR-24	0	GL	0.5	A	B/3	C	O/C	2	A	A	EF-1 EF-5 EF-7 ETF-002 SLT-1	EF-1 EF-5 EF-7 ETF-002 SLT-2	P - P P R	NO NO NO NO YES, NOTE 1	

DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLOW DIAGRAM: 2-5141D-8

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM-NAME: PAS-CONTAINMENT-HYDROGEN

VALVE		VALVE POSITION		ASME SECTION XI									
NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD A/P CL	CAT	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)
2-ECR-15	0	GL	0.5	A	A/1	C	O/C	2	A	A	EF-1	EF-1	P NO
											EF-5	EF-5	- NO
											EF-7	EF-7	P NO
											ETF-002	ETF-002	P NO
											SLT-1	SLT-2	R YES, NOTE 1
2-ECR-16	0	GL	0.5	A	A/3	C	O/C	2	A	A	EF-1	EF-1	P NO
											EF-5	EF-5	- NO
											EF-7	EF-7	P NO
											ETF-002	ETF-002	P NO
											SLT-1	SLT-2	R YES, NOTE 1
2-ECR-17	0	GL	0.5	A	A/3	C	O/C	2	A	A	EF-1	EF-1	P NO
											EF-5	EF-5	- NO
											EF-7	EF-7	P NO
											ETF-002	ETF-002	P NO
											SLT-1	SLT-2	R YES, NOTE 1
2-ECR-18	0	GL	0.5	A	A/4	C	O/C	2	A	A	EF-1	EF-1	P NO
											EF-5	EF-5	- NO
											EF-7	EF-7	P NO
											ETF-002	ETF-002	P NO
											SLT-1	SLT-2	R YES, NOTE 1
2-ECR-19	0	GL	0.5	A	A/4	C	O/C	2	A	A	EF-1	EF-1	P NO
											EF-5	EF-5	- NO
											EF-7	EF-7	P NO
											ETF-002	ETF-002	P NO
											SLT-1	SLT-2	R YES, NOTE 1

DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLOW DIAGRAM: 2-5141D-8

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM-NAME: PAS-CONTAINMENT HYDROGEN

VALVE				VALVE POSITION				ASME SECTION XI						
NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD	A/P	CAT	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)
2-ECR-10	0	GL	0.5	A	C/8	C	O/C	2	A	A	EF-1	EF-1	P	NO
											EF-5	EF-5	-	NO
											EF-7	EF-7	P	NO
											ETF-002	ETF-002	P	NO
											SLT-1	SLT-2	R	YES, NOTE 1
2-ECR-11	0	GL	0.5	A	A/2	C	O/C	2	A	A	EF-1	EF-1	P	NO
											EF-5	EF-5	-	NO
											EF-7	EF-7	P	NO
											ETF-002	ETF-002	P	NO
											SLT-1	SLT-2	R	YES, NOTE 1
2-ECR-12	0	GL	0.5	A	A/2	C	O/C	2	A	A	EF-1	EF-1	P	NO
											EF-5	EF-5	-	NO
											EF-7	EF-7	P	NO
											ETF-002	ETF-002	P	NO
											SLT-1	SLT-2	R	YES, NOTE 1
2-ECR-13	0	GL	0.5	A	A/1	C	O/C	2	A	A	EF-1	EF-1	P	NO
											EF-5	EF-5	-	NO
											EF-7	EF-7	P	NO
											ETF-002	ETF-002	P	NO
											SLT-1	SLT-2	R	YES, NOTE 1
2-ECR-14	0	GL	0.5	A	A/3	C	O/C	2	A	A	EF-1	EF-1	P	NO
											EF-5	EF-5	-	NO
											EF-7	EF-7	P	NO
											ETF-002	ETF-002	P	NO
											SLT-1	SLT-2	R	YES, NOTE 1



DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLOW DIAGRAM: 2-5141A-30

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM-NAME: NUCLEAR-SAMPLING

VALVE				VALVE POSITION				ASME SECTION XI						
NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD A/P ICL	CAT	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)	
2-MCR-253	0	GL	0.5	A	B/1	O	C	2	A B	EF-1	EF-1	P	NO	
										EF-5	EF-5	-	NO	
										EF-7	EF-7	P	NO	
										ETF-002	ETF-002	P	NO	
2-MCR-254	0	GL	0.5	A	B/1	O	C	2	A B	EF-1	EF-1	P	NO	
										EF-5	EF-5	-	NO	
										EF-7	EF-7	P	NO	
										ETF-002	ETF-002	P	NO	

DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLOW DIAGRAM: 2-5141A-30

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM-NAME: NUCLEAR SAMPLING

VALVE				VALVE POSITION				ASME SECTION XI						
NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD A/P ICL	CAT	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)	
2-DCR-301	0	GL	0.5	A	B/2	0	C	2	A B	EF-1 EF-5 EF-7 ET-003	EF-1 EF-5 EF-7 ET-003	P - P P	NO NO NO NO	
2-DCR-302	0	GL	0.5	A	B/3	0	C	2	A B	EF-1 EF-5 EF-7 ETF-002	EF-1 EF-5 EF-7 ETF-002	P - P P	NO NO NO NO	
2-DCR-303	0	GL	0.5	A	B/3	0	C	2	A B	EF-1 EF-5 EF-7 ETF-002	EF-1 EF-5 EF-7 ETF-002	P - P P	NO NO NO NO	
2-DCR-304	0	GL	0.5	A	B/3	0	C	2	A B	EF-1 EF-5 EF-7 ETF-002	EF-1 EF-5 EF-7 ETF-002	P - P P	NO NO NO NO	
2-MCR-251	0	GL	0.5	A	B/2	0	C	2	A B	EF-1 EF-5 EF-7 ETF-002	EF-1 EF-5 EF-7 ETF-002	P - P P	NO NO NO NO	
2-MCR-252	0	GL	0.5	A	B/2	0	C	2	A B	EF-1 EF-5 EF-7 ETF-002	EF-1 EF-5 EF-7 ETF-002	P - P P	NO NO NO NO	

DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLOW DIAGRAM: 2-5141-27

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM-NAME: NUCLEAR-SAMPLING

VALVE				VALVE POSITION				ASME SECTION XI						
NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD	A/P	CAT	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)
2-NCR-108	0	GL	0.5	A	D/6	0	C	2	A	A	EF-1	EF-1	P	NO
											EF-5	EF-5	-	NO
											EF-7	EF-7	P	NO
											ETF-002	ETF-002	P	NO
											SLT-1	SLT-2	R	YES, NOTE 1
2-NCR-109	0	GL	0.5	A	D/5	0	C	2	A	A	EF-1	EF-1	P	NO
											EF-5	EF-5	-	NO
											EF-7	EF-7	P	NO
											ETF-002	ETF-002	P	NO
											SLT-1	SLT-2	R	YES, NOTE 1
2-NCR-110	0	GL	0.5	A	D/6	0	C	2	A	A	EF-1	EF-1	P	NO
											EF-5	EF-5	-	NO
											EF-7	EF-7	P	NO
											ETF-002	ETF-002	P	NO
											SLT-1	SLT-2	R	YES, NOTE 1



DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLOW DIAGRAM: 2-5141-27

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM-NAME: NUCLEAR-SAMPLING

VALVE

VALVE POSITION

ASME SECTION XI

NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD CL	A/P	CAT	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)
2-ICR-5	0	GL	0.5	A	C/5	0	C	2	A	A	EF-1 EF-5 EF-7 ETF-002 SLT-1	EF-1 EF-5 EF-7 ETF-002 SLT-2	P - P P R	NO NO NO NO YES, NOTE 1
2-ICR-6	0	GL	0.5	A	D/5	0	C	2	A	A	EF-1 EF-5 EF-7 ETF-002 SLT-1	EF-1 EF-5 EF-7 ETF-002 SLT-2	P - P P R	NO NO NO NO YES, NOTE 1
2-NCR-105	0	GL	0.5	A	C/7	0	C	2	A	A	EF-1 EF-5 EF-7 ETF-002 SLT-1	EF-1 EF-5 EF-7 ETF-002 SLT-2	P - P P R	NO NO NO NO YES, NOTE 1
2-NCR-106	0	GL	0.5	A	C/7	0	C	2	A	A	EF-1 EF-5 EF-7 ETF-002 SLT-1	EF-1 EF-5 EF-7 ETF-002 SLT-2	P - P P R	NO NO NO NO YES, NOTE 1
2-NCR-107	0	GL	0.5	A	D/6	0	C	2	A	A	EF-1 EF-5 EF-7 ETF-002 SLT-1	EF-1 EF-5 EF-7 ETF-002 SLT-2	P - P P R	NO NO NO NO YES, NOTE 1

DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLOW DIAGRAM: 2-5135B-14

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM-NAME: COMPONENT-COOLING

VALVE				VALVE POSITION				ASME SECTION XI						
NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD A/P CL	CAT	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)	
2-CCH-243-25	2	CK	1	SA	C/5	0	C	2	A	AC	CF-1 SLT-1	CF-2 SLT-2	R R	YES, NOTE 2 YES, NOTE 1
2-CCH-243-72	2	CK	1	SA	C/5	0	C	2	A	AC	CF-1 SLT-1	CF-2 SLT-2	R R	YES, NOTE 2 YES, NOTE 1
2-CCH-244-25	2	CK	1	SA	C/6	0	C	2	A	AC	CF-1 SLT-1	CF-2 SLT-2	R R	YES, NOTE 2 YES, NOTE 1
2-CCH-244-72	2	CK	1	SA	C/6	0	C	2	A	AC	CF-1 SLT-1	CF-2 SLT-2	R R	YES, NOTE 2 YES, NOTE 1
2-SV-122-25B	0	REL	1.5	SA	B/6	C	0	3	A	C	TF-1	TF-1	R	NO
2-SV-122-72B	0	REL	1.5	SA	B/6	C	0	3	A	C	TF-1	TF-1	R	NO

DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLOW DIAGRAM: 2-5135B-14

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM-NAME: COMPONENT-COOLING

VALVE				VALVE POSITION				ASME SECTION XI									
NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD CL	A/P	CAT	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)			
2-CCM-430	0	GL	1.5	MO	D/6	C	O/C	2	A	A	EF-1 EF-5 ET-018 SLT-1	EF-1 EF-5 ET-018 SLT-2	P - P R	NO NO NO YES, NOTE 1			
2-CCM-431	0	GL	1.5	MO	D/6	C	O/C	2	A	A	EF-1 EF-5 ET-015 SLT-1	EF-1 EF-5 ET-015 SLT-2	P - P R	NO NO NO YES, NOTE 1			
2-CCM-432	0	GL	1.5	MO	D/6	C	O/C	2	A	A	EF-1 EF-5 ET-018 SLT-1	EF-1 EF-5 ET-018 SLT-2	P - P R	NO NO NO YES, NOTE 1			
2-CCM-433	0	GL	1.5	MO	D/6	C	O/C	2	A	A	EF-1 EF-5 ET-019 SLT-1	EF-1 EF-5 ET-019 SLT-2	P - P R	NO NO NO YES, NOTE 1			
2-CCR-440	0	GL	1.5	A	D/6	O	C	2	A	A	EF-1 EF-5 EF-7 ET-008 SLT-1	EF-1 EF-5 EF-7 ET-008 SLT-2	P - P P R	NO NO NO NO YES, NOTE 1			
2-CCR-441	0	GL	1.5	A	D/6	O	C	2	A	A	EF-1 EF-5 EF-7 ET-005 SLT-1	EF-1 EF-5 EF-7 ET-005 SLT-2	P - P P R	NO NO NO NO YES, NOTE 1			

DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLOW DIAGRAM: 2-5135A-30

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM NAME: COMPONENT COOLING

VALVE				VALVE POSITION				ASME SECTION XI							
NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD A/P CL	A	B	CAT	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)
2-CMO-416	0	BF	16	MO	G/5	0	O/C	3	A	B		EF-1 EF-5 ET-103	EF-1 EF-5 ET-103	P - P	NO NO NO, NOTE 1
2-CMO-419	0	BF	14	MO	E/5	C	0	3	A	B		EF-1 EF-5 ET-053	EF-1 EF-5 ET-053	P - P	NO NO NO
2-CMO-420	0	BF	16	MO	H/4	C/O	0	3	A	B		EF-1 EF-5 ET-066	EF-1 EF-5 ET-066	P - P	NO NO NO
2-CMO-429	0	BF	14	MO	E/5	C	0	3	A	B		EF-1 EF-5 ET-054	EF-1 EF-5 ET-054	P - P	NO NO NO
2-CRV-412	0	GL	4	A	K/1	0	C	3	A	B		EF-1 EF-5 EF-7 ET-019	EF-1 EF-5 EF-7 ET-019	P - P P	NO NO NO NO
2-SV-60	0	REL	3	SA	L/1	C	0	3	A	C		TF-1	TF-1	R	NO
2-SV-72	0	REL	1	SA	E/5	C	0	3	A	C		TF-1	TF-1	R	NO

DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLOW DIAGRAM: 2-5135A-30

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM NAME: COMPONENT-COOLING

VALVE		VALVE POSITION		ASME SECTION XI											
NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD	A/P	CAT	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)	
2-CCN-176-E	0	CK	16	SA	L/4	C/O	0	3	A	C	CF-1	CF-1	P	NO	
2-CCN-176-W	0	CK	16	SA	K/4	C/O	0	3	A	C	CF-1	CF-1	P	NO	
2-CMO-410	0	BF	16	MO	H/4	C/O	0	3	A	B	EF-1 EF-5 ET-055	EF-1 EF-5 ET-055	P - P	NO NO NO	
2-CMO-411	0	BF	18	MO	M/5	0	0/C	3	A	B	EF-1 EF-5 ET-053	EF-1 EF-5 ET-053	P - P	NO NO NO, NOTE 1	
2-CMO-412	0	BF	16	MO	L/3	0	0/C	3	A	B	EF-1 EF-5 ET-054	EF-1 EF-5 ET-054	P - P	NO NO NO, NOTE 1	
2-CMO-413	0	BF	18	MO	L/5	0	0/C	3	A	B	EF-1 EF-5 ET-053	EF-1 EF-5 ET-053	P - P	NO NO NO, NOTE 1	
2-CMO-414	0	BF	16	MO	K/3	0	0/C	3	A	B	EF-1 EF-5 ET-054	EF-1 EF-5 ET-054	P - P	NO NO NO, NOTE 1	
2-CMO-415	0	BF	16	MO	H/5	0	0/C	3	A	B	EF-1 EF-5 ET-054	EF-1 EF-5 ET-054	P - P	NO NO NO, NOTE 1	

DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLO. DIAGRAM: 2-5135-34

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM-NAME: COMPONENT-COOLING

VALVE

VALVE POSITION

ASME SECTION XI

NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD CL	A/P	CAT	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)
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2-SV-68	0	REL	1	SA	J/2	C	0	3	A	C	TF-1	TF-1	R	NO
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2-SV-71	0	REL	1	SA	L/3	C	0	3	A	C	TF-1	TF-1	R	NO
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DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLOD DIAGRAM: 2-5135-34

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM NAME: COMPONENT COOLING

VALVE				VALVE POSITION				ASME SECTION XI						
NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD A/P CL	CAT	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)	
2-CCN-135	0	CK	2.5	SA	B/3	0	C	2	A	AC	CF-1 SLT-1	CF-2 SLT-2	R R	YES, NOTE 4 YES, NOTE 2
2-CRV-445	1	BL	6	A	L/5	0	C/O	3	A	B	EF-1 EF-5 EF-7 ET-034	EF-1 EF-5 EF-7 ET-034	P - P P	NO NO NO NO
2-CRV-470	0	GL	6	A	G/1	0	C	3	A	B	EF-1 EF-7 ET-NA	EF-1 NOTE 5 NOTE 5	P - -	NO YES, NOTE 5 YES, NOTE 5
2-CRV-485	0	BF	10	A	B/7	0	C	3	A	B	EF-1 EF-5 EF-7 ET-018	EF-1 EF-5 EF-7 ET-018	P - P P	NO NO NO NO
2-SV-122	0	REL	1	SA	D/3	C	0	3	A	C	TF-1	TF-1	R	NO
2-SV-62	0	REL	1	SA	D/3	C	0	3	A	C	TF-1	TF-1	R	NO
2-SV-63	0	REL	1	SA	E/3	C	0	3	A	C	TF-1	TF-1	R	NO
2-SV-64	0	REL	1	SA	C/3	C	0	3	A	C	TF-1	TF-1	R	NO
2-SV-65	0	REL	1	SA	H/1	C	0	3	A	C	TF-1	TF-1	R	NO

DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLOW DIAGRAM: 2-5135-34

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM-NAME: COMPONENT-COOLING

VALVE				VALVE POSITION				ASME SECTION XI						
NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD	A/P	CAT	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)
2-CCR-455	0	GL	2	A	B/3	0	C	2	A	A	EF-1	EF-2	C	YES, NOTE 3
											EF-5	EF-5	-	NO
											EF-7	EF-8	C	NO, NOTE 3
											ET-005	ET-005	C	NO
											SLT-1	SLT-2	R	YES, NOTE 2
2-CCR-456	0	GL	2	A	D/4	0	C	2	A	A	EF-1	EF-2	C	YES, NOTE 3
											EF-5	EF-5	-	NO
											EF-7	EF-8	C	NO, NOTE 3
											ET-005	ET-005	C	NO
											SLT-1	SLT-2	R	YES, NOTE 2
2-CCR-457	0	GL	2	A	D/4	0	C	2	A	A	EF-1	EF-2	C	YES, NOTE 3
											EF-5	EF-5	-	NO
											EF-7	EF-8	C	NO, NOTE 3
											ETF-005	ETF-005	C	NO
											SLT-1	SLT-2	R	YES, NOTE 2
2-CCR-460	0	GL	3	A	C/4	0	C	2	A	A	EF-1	EF-1	P	NO
											EF-5	EF-5	-	NO
											EF-7	EF-7	P	NO
											ET-010	ET-010	P	NO
											SLT-1	SLT-2	R	YES, NOTE 2
2-CCR-462	0	GL	3	A	A/4	0	C	2	A	A	EF-1	EF-1	P	NO
											EF-5	EF-5	-	NO
											EF-7	EF-7	P	NO
											ET-009	ET-009	P	NO
											SLT-1	SLT-2	R	YES, NOTE 2

DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLOW DIAGRAM: 2-5135-34

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM-NAME: COMPONENT COOLING

VALVE

VALVE POSITION

ASME SECTION XI

NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD CL	A/P	CAT	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)
2-CCM-451	0	BF	8	MO	E/4	0	C	2	A	A	EF-1 EF-5 ET-026 SLT-1	EF-2 EF-5 ET-026 SLT-2	C - C R	YES, NOTE 1 NO NO YES, NOTE 2
2-CCM-452	0	BF	8	MO	E/5	0	C	2	A	A	EF-1 EF-5 ET-023 SLT-1	EF-2 EF-5 ET-023 SLT-2	C - C R	YES, NOTE 1 NO NO YES, NOTE 2
2-CCM-453	0	GL	4	MO	E/4	0	C	2	A	A	EF-1 EF-5 ET-030 SLT-1	EF-2 EF-5 ET-030 SLT-2	C - C R	YES, NOTE 1 NO NO YES, NOTE 2
2-CCM-454	0	GL	4	MO	E/5	0	C	2	A	A	EF-1 EF-5 ET-030 SLT-1	EF-2 EF-5 ET-030 SLT-2	C - C R	YES, NOTE 1 NO NO YES, NOTE 2
2-CCM-458	0	BF	8	MO	A/2	0	C	2	A	A	EF-1 EF-5 ET-023 SLT-1	EF-2 EF-5 ET-023 SLT-2	C - C R	YES, NOTE 1 NO NO YES, NOTE 2
2-CCM-459	0	BF	8	MO	B/2	0	C	2	A	A	EF-1 EF-5 ET-024 SLT-1	EF-2 EF-5 ET-024 SLT-2	C - C R	YES, NOTE 1 NO NO YES, NOTE 2



DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLOW-DIAGRAM: 2-5129A-20

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM-NAME: CVCS - LETDOWN & CHARGING

VALVE

VALVE POSITION

ASME SECTION XI

NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD CL	A/P	CAT	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)
2-QCM-250	0	GA	4	MO	C/8	0	C	2	A	A	EF-1 EF-5 ET-015 SLT-1	EF-2 EF-5 ET-015 SLT-2	C - C R	YES, NOTE 1 NO NO YES, NOTE 2
2-QCM-350	0	GA	4	MO	D/8	0	C	2	A	A	EF-1 EF-5 ET-009 SLT-1	EF-2 EF-5 ET-009 SLT-2	C - C R	YES, NOTE 1 NO NO YES, NOTE 2
2-QMO-451	0	GA	4	MO	J/5	0	C	2	A	B	EF-1 EF-5 ET-009	EF-2 EF-5 ET-009	C - C	YES, NOTE 3 NO NO
2-QMO-452	0	GA	4	MO	J/5	0	C	2	A	B	EF-1 EF-5 ET-006	EF-2 EF-5 ET-006	C - C	YES, NOTE 3 NO NO
2-QRV-400	0	GL	2	A	K/4	C	0	2	A	B	EF-1 EF-5 EF-7 ET-005	EF-1 EF-5 EF-7 ET-005	P - P P	NO NO NO NO
2-SV-53	0	REL	3	SA	H/2	C	0	2	A	C	TF-1	TF-1	R	NO
2-SV-54	0	REL	2	SA	E/4	C	0	2	A	C	TF-1	TF-1	R	NO

DONALD C. COOK NUCLEAR PLANT
 SECOND TEN YEAR INTERVAL
 VALVE SUMMARY SHEET - UNIT 2
 FLOW DIAGRAM: 2-5129-32

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM-NAME: CVCS - LETDOWN & CHARGING

VALVE

VALVE POSITION

ASME SECTION XI

NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD CL	A/P	CAT	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)
2-SV-55	0	REL	0.75	SA	K/7	C	0	2	A	C	TF-1	TF-1	R	NO
2-SV-56	0	REL	0.75	SA	L/6	C	0	2	A	C	TF-1	TF-1	R	NO



DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLOW DIAGRAM: 2-5129-32

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM-NAME: CVCS - LETDOWN & CHARGING

VALVE		VALVE POSITION		ASME SECTION XI									
NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD A/P CL	CAT	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)
2-QM0-226	0	GA	2	MO	G/7	0	O/C	2	A B	EF-1 EF-5 ET-011	EF-1 EF-5 ET-011	P - P	NO NO NO
2-QRV-200	1	GL	3	A	H/3	0	0	2	A B	EF-1 EF-7 ET-NA	EF-3 NOTE 8 NOTE 8	P - -	NO, NOTE 8 YES, NOTE 8 YES, NOTE 8
2-QRV-251	1	GL	3	A	H/5	0	0	2	A B	EF-1 EF-7 ET-NA	EF-3 NOTE 9 NOTE 9	P - -	NO, NOTE 9 YES, NOTE 9 YES, NOTE 9
2-QRV-61	0	GL	3	A	C/2	C	0	2	A B	EF-1 EF-5 EF-7 ET-008	EF-1 EF-5 EF-7 ET-008	P - P P	NO NO NO NO
2-QRV-62	0	GL	3	A	C/2	O/C	0	2	A B	EF-1 EF-5 EF-7 ET-009	EF-1 EF-5 EF-7 ET-009	P - P P	NO NO NO NO
2-SI-185	0	CK	8	SA	K/5	C	0	2	A C	CF-1	CF-2	R	YES, NOTE 10
2-SV-51	0	REL	2	SA	E/2	C	0	2	A C	TF-1	TF-1	R	NO
2-SV-52	0	REL	2	SA	K/1	C	0	2	A C	TF-1	TF-1	R	NO

DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLOW DIAGRAM: 2-5129-32

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM-NAME: CVCS - LETDOWN & CHARGING

VALVE				VALVE POSITION				ASME SECTION XI						
NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD A/P CL	CAT	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)	
2-IMO-910	0	GA	8	MO	L/5	C	0	2	A B	EF-1 EF-5 ET-012	EF-1 EF-5 ET-012	P - P	NO NO NO	
2-IMO-911	0	GA	8	MO	L/6	C	0	2	A B	EF-1 EF-5 ET-012	EF-1 EF-5 ET-012	P - P	NO NO NO	
2-QCR-300	0	GL	2	A	E/1	0	C	2	A A	EF-1 EF-5 EF-7 ET-003 SLT-1	EF-2 EF-5 EF-8 ET-003 SLT-2	C - C C R	YES, NOTE 6 NO NO, NOTE 6 NO YES, NOTE 3	
2-QCR-301	0	GL	2	A	E/1	0	C	2	A A	EF-1 EF-5 EF-7 ET-005 SLT-1	EF-2 EF-5 EF-8 ET-005 SLT-2	C - C C R	YES, NOTE 6 NO NO, NOTE 6 NO YES, NOTE 3	
2-QMO-200	0	GA	3	MO	J/3	0	O/C	2	A B	EF-1 EF-5 ET-012	EF-2 EF-5 ET-012	C - C	YES, NOTE 7 NO NO	
2-QMO-201	0	GA	3	MO	J/3	0	O/C	2	A B	EF-1 EF-5 ET-012	EF-2 EF-5 ET-012	C - C	YES, NOTE 7 NO NO	
2-QMO-225	0	GA	2	MO	J/7	0	O/C	2	A B	EF-1 EF-5 ET-012	EF-1 EF-5 ET-012	P - P	NO NO NO	

DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLOW DIAGRAM: 2-5129-32

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM NAME: CVCS - LETDOWN & CHARGING

VALVE				VALVE POSITION				ASME SECTION XI						
NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD A/P CL	CAT	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)	
2-CS-292	0	CK	2	SA	H/6	C	O/C	2	A	C	CF-1	CF-2	C	YES, NOTE 1
2-CS-297-E	0	CK	2	SA	H/7	O/C	0	2	A	C	CF-1	CF-1	P	NO
2-CS-297-H	0	CK	2	SA	F/7	O/C	0	2	A	C	CF-1	CF-1	P	NO
2-CS-299-E	0	CK	4	SA	H/7	0	O/C	2	A	AC	CF-1 SLT-1	CF-3 SLT-1	P R	YES, NOTE 2 NO, NOTE 3
2-CS-299-H	0	CK	4	SA	F/7	0	O/C	2	A	AC	CF-1 SLT-1	CF-3 SLT-1	P R	YES, NOTE 2 NO, NOTE 3
2-CS-321	0	CK	3	SA	E/3	0	C/O	2	A	AC	CF-1 SLT-1	CF-2 SLT-2	- R	YES, NOTE 4 YES, NOTE 3
2-CS-328-L1	0	CK	3	SA	B/2	O/C	0	1	A	C	CF-1	CF-1	P	NO, NOTE 5
2-CS-328-L4	0	CK	3	SA	B/3	C/O	0	1	A	C	CF-1	CF-1	P	NO, NOTE 5
2-CS-329-L1	0	CK	3	SA	B/2	O/C	0	1	A	C	CF-1	CF-1	P	NO, NOTE 5
2-CS-329-L4	0	CK	3	SA	B/3	C/O	0	1	A	C	CF-1	CF-1	P	NO, NOTE 5
2-IMO-360	0	GA	4	MO	J/6	C	0	2	A	B	EF-1 EF-5 ET-009	EF-1 EF-5 ET-009	P - P	NO NO NO

DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLOW DIAGRAM: 2-5128A-34

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM-NAME: REACTOR-COOLANT

VALVE

VALVE POSITION

ASME SECTION XI

NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD CL	A/P	CAT	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)
2-SI-189	2	CK	4	SA	D/7	C	O/C	2	A	AC	CF-1 SLT-1	CF-2 SLT-2	R R	YES, NOTE 7 YES, NOTE 2
2-SV-45A	0	REL	6	SA	K/6	C	0	1	A	C	TF-1	TF-1	R	NO
2-SV-45B	0	REL	6	SA	J/6	C	0	1	A	C	TF-1	TF-1	R	NO
2-SV-45C	0	REL	6	SA	H/6	C	0	1	A	C	TF-1	TF-1	R	NO
2-SV-50	0	REL	2	SA	G/3	C	0	3	A	C	TF-1	TF-1	R	NO

DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLOW DIAGRAM: 2-5128A-34

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM-NAME: REACTOR COOLANT

VALVE				VALVE POSITION				ASME SECTION XI						
NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD CL	A/P	CAT	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)
2-NSO-062	2	GL	1	SO	M/6	C	O/C	2	A	B	EF-1 EF-5 EF-7 ETF-002	EF-2 EF-5 EF-8 ETF-002	R - R R	YES, NOTE 6 NO, NOTE 6 YES, NOTE 6 YES, NOTE 6
2-NSO-063	2	GL	1	SO	M/6	C	O/C	2	A	B	EF-1 EF-5 EF-7 ETF-002	EF-2 EF-5 EF-8 ETF-002	R - R R	YES, NOTE 6 NO, NOTE 6 YES, NOTE 6 YES, NOTE 6
2-NSO-064	2	GL	1	SO	M/6	C	O/C	2	A	B	EF-1 EF-5 EF-7 ETF-002	EF-2 EF-5 EF-8 ETF-002	R - R R	YES, NOTE 6 NO, NOTE 6 YES, NOTE 6 YES, NOTE 6
2-PH-275	2	CK	3	SA	B/9	O/C	C	2	A	AC	CF-1 SLT-1	CF-2 SLT-2	R R	YES, NOTE 4 YES, NOTE 2
2-RCR-100	0	GL	0.375	A	B/7	O	C	2	A	A	EF-1 EF-5 EF-7 ETF-002 SLT-1	EF-1 EF-5 EF-7 ETF-002 SLT-2	P - P P R	NO NO NO NO YES, NOTE 2
2-RCR-101	0	GL	0.375	A	B/7	O	C	2	A	A	EF-1 EF-5 EF-7 ETF-002 SLT-1	EF-1 EF-5 EF-7 ETF-002 SLT-2	P - P P R	NO NO NO NO YES, NOTE 2



DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLOW DIAGRAM: 2-5128A-34

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM-NAME: REACTOR-COOLANT

VALVE				VALVE POSITION				ASME SECTION XI						
NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD A/P CL	CAT	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)	
2-NMO-152	0	GA	3	MO	K/7	0	C	1	A B	EF-1 EF-5 ET-014	EF-1 EF-5 ET-014	P - P	NO NO NO	
2-NMO-153	0	GA	3	MO	K/6	0	C	1	A B	EF-1 EF-5 ET-014	EF-1 EF-5 ET-014	P - P	NO NO NO	
2-NPX-151	0	GL	0.5	M	N/8	C	C	2	P A	SLT-1	SLT-2	R	YES, NOTE 2	
2-NRV-151	0	GL	3	A	K/7	C	C/O	1	A B	EF-1 EF-5 EF-7 ET-008	EF-4 EF-5 EF-8 ET-008	- - - -	NO, NOTE 3 NO NO, NOTE 3 NO	
2-NRV-152	0	GL	3	A	K/7	C	C/O	1	A B	EF-1 EF-5 EF-7 ET-008	EF-4 EF-5 EF-8 ET-008	- - - -	NO, NOTE 3 NO NO, NOTE 3 NO	
2-NRV-153	0	GL	3	A	K/6	C	C/O	1	A B	EF-1 EF-5 EF-7 ET-008	EF-4 EF-5 EF-8 ET-008	- - - -	NO, NOTE 3 NO NO, NOTE 3 NO	
2-NSO-061	2	GL	1	SO	M/6	C	O/C	2	A B	EF-1 EF-5 EF-7 ETF-002	EF-2 EF-5 EF-8 ETF-002	R - R R	YES, NOTE 6 NO, NOTE 6 YES, NOTE 6 YES, NOTE 6	



DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLOW DIAGRAM: 2-5128A-34

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM-NAME: REACTOR COOLANT

VALVE			VALVE POSITION			ASME SECTION XI									
NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD	A/P	CAT	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)	
2-CS-442-1	0	CK	2	SA	B/4	0	C	2	A	AC	CF-1 SLT-1	CF-2 SLT-2	R R	YES, NOTE 1 YES, NOTE 2	
2-CS-442-2	0	CK	2	SA	B/4	0	C	2	A	AC	CF-1 SLT-1	CF-2 SLT-2	R R	YES, NOTE 1 YES, NOTE 2	
2-CS-442-3	0	CK	2	SA	B/4	0	C	2	A	AC	CF-1 SLT-1	CF-2 SLT-2	R R	YES, NOTE 1 YES, NOTE 2	
2-CS-442-4	0	CK	2	SA	B/4	0	C	2	A	AC	CF-1 SLT-1	CF-2 SLT-2	R R	YES, NOTE 1 YES, NOTE 2	
2-GCR-301	0	DA	0.75	A	B/8	0	C	2	A	A	EF-1 EF-5 EF-7 ETF-002 SLT-1	EF-1 EF-5 EF-7 ETF-002 SLT-2	P - P P R	NO NO NO NO YES, NOTE 2	
2-N-159	2	CK	0.75	SA	C/8	O/C	C	2	A	AC	CF-1 SLT-1	CF-2 SLT-2	R R	YES, NOTE 5 YES, NOTE 2	
2-NCR-252	0	GL	3	A	B/9	C	C	2	P	A	EF-1 EF-5 EF-7 ET-004 SLT-1	EF-1 EF-5 EF-7 ET-004 SLT-2	P - P P R	NO NO NO NO YES, NOTE 2	
2-NFO-151	0	GA	3	NO	K/7	0	C	1	A	B	EF-1 EF-5 ET-015	EF-1 EF-5 ET-015	P - P	NO NO NO	



DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLOW DIAGRAM: 2-5128-19

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM-NAME: REACTOR-COOLANT

VALVE				VALVE POSITION				ASME SECTION XI									
NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD	A/P	CAT	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)			
2-NSO-021	2	GL	1	SO	J/6	C	O/C	2	A	B	EF-1	EF-2	R	YES, NOTE 1			
											EF-5	EF-5	-	NO, NOTE 1			
											EF-7	EF-8	R	YES, NOTE 1			
											ETF-002	ETF-002	R	YES, NOTE 1			
2-NSO-022	2	GL	1	SO	J/6	C	O/C	2	A	B	EF-1	EF-2	R	YES, NOTE 1			
											EF-5	EF-5	-	NO, NOTE 1			
											EF-7	EF-8	R	YES, NOTE 1			
											ETF-002	ETF-002	R	YES, NOTE 1			
2-NSO-023	2	GL	1	SO	J/6	C	O/C	2	A	B	EF-1	EF-2	R	YES, NOTE 1			
											EF-5	EF-5	-	NO, NOTE 1			
											EF-7	EF-8	R	YES, NOTE 1			
											ETF-002	ETF-002	R	YES, NOTE 1			
2-NSO-024	2	GL	1	SO	J/6	C	O/C	2	A	B	EF-1	EF-2	R	YES, NOTE 1			
											EF-5	EF-5	-	NO, NOTE 1			
											EF-7	EF-8	R	YES, NOTE 1			
											ETF-002	ETF-002	R	YES, NOTE 1			

DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLOW DIAGRAM: 2-5124-20

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM-NAME: STATION DRAINAGE - CONTAINMENT

VALVE

VALVE POSITION

ASME SECTION XI

NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD A/P CL	CAT	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)	
2-DCR-600	0	DA	3	A	N/6	0	C	2	A	A	EF-1 EF-5 EF-7 ET-004 SLT-1	EF-1 EF-5 EF-7 ET-004 SLT-2	P - P P R	NO NO NO NO YES, NOTE 1
2-DCR-601	0	DA	3	A	N/6	0	C	2	A	A	EF-1 EF-5 EF-7 ET-004 SLT-1	EF-1 EF-5 EF-7 ET-004 SLT-2	P - P P R	NO NO NO NO YES, NOTE 1
2-NS-357	2	CK	0.5	SA	K/9	C	O/C	2	A	AC	CF-1 SLT-1	CF-2 SLT-2	R R	YES, NOTE 2 YES, NOTE 1

DONALD C. COOK NUCLEAR PLANT
 SECOND TEN YEAR INTERVAL
 VALVE SUMMARY SHEET - UNIT 2
 FLOW DIAGRAM: 2-5114A-27

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM-NAME: NON-ESSENTIAL SERVICE WATER

VALVE

VALVE POSITION

ASME SECTION XI

NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD CL	A/P	CAT	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)
2-NCR-967-3	0	DA	2	A	J/3	0	C	2	A	A	EF-1 EF-5 EF-7 ET-005 SLT-1	EF-1 EF-5 EF-7 ET-005 SLT-2	P - P P R	NO NO NO NO YES, NOTE 1

DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLOW DIAGRAM: 2-5114A-27

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM-NAME: NON-ESSENTIAL SERVICE WATER

VALVE				VALVE POSITION				ASME SECTION XI						
NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD A/P CL	CAT	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)	
2-MCR-962	0	DA	2	A	J/3	0	C	2	A	A	EF-1 EF-5 EF-7 ET-004 SLT-1	EF-1 EF-5 EF-7 ET-004 SLT-2	P - P P R	NO NO NO NO YES, NOTE 1
2-MCR-963	0	DA	2	A	J/3	0	C	2	A	A	EF-1 EF-5 EF-7 ET-004 SLT-1	EF-1 EF-5 EF-7 ET-004 SLT-2	P - P P R	NO NO NO NO YES, NOTE 1
2-MCR-964-3	0	DA	2	A	J/7	0	C	2	A	A	EF-1 EF-5 EF-7 ET-005 SLT-1	EF-1 EF-5 EF-7 ET-005 SLT-2	P - P P R	NO NO NO NO YES, NOTE 1
2-MCR-965-3	0	DA	2	A	J/7	0	C	2	A	A	EF-1 EF-5 EF-7 ET-005 SLT-1	EF-1 EF-5 EF-7 ET-005 SLT-2	P - P P R	NO NO NO NO YES, NOTE 1
2-MCR-966-3	0	DA	2	A	J/3	0	C	2	A	A	EF-1 EF-5 EF-7 ET-005 SLT-1	EF-1 EF-5 EF-7 ET-005 SLT-2	P - P P R	NO NO NO NO YES, NOTE 1

DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLOW DIAGRAM: 2-5114A-27

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM NAME: NON-ESSENTIAL SERVICE WATER

VALVE

VALVE POSITION

ASME SECTION XI

NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD CL	A/P	CAT	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)
2-MCR-956-2	0	DA	3	A	J/3	0	C	2	A	A	EF-1 EF-5 EF-7 ET-004 SLT-1	EF-1 EF-5 EF-7 ET-004 SLT-2	P - P P R	NO NO NO NO YES, NOTE 1
2-MCR-957-3	0	DA	3	A	J/3	0	C	2	A	A	EF-1 EF-5 EF-7 ET-003 SLT-1	EF-1 EF-5 EF-7 ET-003 SLT-2	P - P P R	NO NO NO NO YES, NOTE 1
2-MCR-958-4	0	DA	3	A	J/3	0	C	2	A	A	EF-1 EF-5 EF-7 ET-005 SLT-1	EF-1 EF-5 EF-7 ET-005 SLT-2	P - P P R	NO NO NO NO YES, NOTE 1
2-MCR-960	0	DA	2	A	J/7	0	C	2	A	A	EF-1 EF-5 EF-7 ET-005 SLT-1	EF-1 EF-5 EF-7 ET-005 SLT-2	P - P P R	NO NO NO NO YES, NOTE 1
2-MCR-961	0	DA	2	A	J/7	0	C	2	A	A	EF-1 EF-5 EF-7 ET-005 SLT-1	EF-1 EF-5 EF-7 ET-005 SLT-2	P - P P R	NO NO NO NO YES, NOTE 1



DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLON DIAGRAM: 2-5114A-27

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM NAME: NON-ESSENTIAL SERVICE WATER

VALVE				VALVE POSITION				ASME SECTION XI									
NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD	A/P	CAT	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)			
2-MCR-951	0	DA	3	A	K/6	0	C	2	A	A	EF-1	EF-1	P	NO			
											EF-5	EF-5	-	NO			
											EF-7	EF-7	P	NO			
											ET-005	ET-005	P	NO			
											SLT-1	SLT-2	R	YES, NOTE 1			
2-MCR-952-2	0	DA	3	A	K/6	0	C	2	A	A	EF-1	EF-1	P	NO			
											EF-5	EF-5	-	NO			
											EF-7	EF-7	P	NO			
											ET-004	ET-004	P	NO			
											SLT-1	SLT-2	R	YES, NOTE 1			
2-MCR-953-3	0	DA	3	A	K/6	0	C	2	A	A	EF-1	EF-1	P	NO			
											EF-5	EF-5	-	NO			
											EF-7	EF-7	P	NO			
											ET-005	ET-005	P	NO			
											SLT-1	SLT-2	R	YES, NOTE 1			
2-MCR-954-4	0	DA	3	A	K/6	0	C	2	A	A	EF-1	EF-1	P	NO			
											EF-5	EF-5	-	NO			
											EF-7	EF-7	P	NO			
											ET-005	ET-005	P	NO			
											SLT-1	SLT-2	R	YES, NOTE 1			
2-MCR-955	0	DA	3	A	J/3	0	C	2	A	A	EF-1	EF-1	P	NO			
											EF-5	EF-5	-	NO			
											EF-7	EF-7	P	NO			
											ET-005	ET-005	P	NO			
											SLT-1	SLT-2	R	YES, NOTE 1			

DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLOW DIAGRAM: 2-5114A-27

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM NAME: NON-ESSENTIAL SERVICE WATER

VALVE				VALVE POSITION				ASME SECTION XI						
NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD	A/P	CAT	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)
2-MCR-944-4	0	DA	3	A	J/6	0	C	2	A	A	EF-1 EF-5 EF-7 ET-004 SLT-1	EF-1 EF-5 EF-7 ET-004 SLT-2	P - P P R	NO NO NO NO YES, NOTE 1
2-MCR-945	0	DA	3	A	J/3	0	C	2	A	A	EF-1 EF-5 EF-7 ET-005 SLT-1	EF-1 EF-5 EF-7 ET-005 SLT-2	P - P P R	NO NO NO NO YES, NOTE 1
2-MCR-946-2	0	DA	3	A	J/3	0	C	2	A	A	EF-1 EF-5 EF-7 ET-004 SLT-1	EF-1 EF-5 EF-7 ET-004 SLT-2	P - P P R	NO NO NO NO YES, NOTE 1
2-MCR-947-3	0	DA	3	A	J/3	0	C	2	A	A	EF-1 EF-5 EF-7 ET-005 SLT-1	EF-1 EF-5 EF-7 ET-005 SLT-2	P - P P R	NO NO NO NO YES, NOTE 1
2-MCR-948-4	0	DA	3	A	J/3	0	C	2	A	A	EF-1 EF-5 EF-7 ET-005 SLT-1	EF-1 EF-5 EF-7 ET-005 SLT-2	P - P P R	NO NO NO NO YES, NOTE 1

DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLOID DIAGRAM: 2-5114A-27

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM-NAME: NON-ESSENTIAL-SERVICE WATER

VALVE				VALVE POSITION				ASME SECTION XI						
NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD A/P CL	CAT	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)	
2-MCR-934	0	DA	3	A	K/2	0	C	2	A	A	EF-1 EF-5 EF-7 ET-005 SLT-1	EF-1 EF-5 EF-7 ET-005 SLT-2	P - P P R	NO NO NO NO YES, NOTE 1
2-MCR-935	0	DA	3	A	J/2	0	C	2	A	A	EF-1 EF-5 EF-7 ET-005 SLT-1	EF-1 EF-5 EF-7 ET-005 SLT-2	P - P P R	NO NO NO NO YES, NOTE 1
2-MCR-941-1	0	DA	3	A	J/6	0	C	2	A	A	EF-1 EF-5 EF-7 ET-005 SLT-1	EF-1 EF-5 EF-7 ET-005 SLT-2	P - P P R	NO NO NO NO YES, NOTE 1
2-MCR-942-2	0	DA	3	A	J/6	0	C	2	A	A	EF-1 EF-5 EF-7 ET-004 SLT-1	EF-1 EF-5 EF-7 ET-004 SLT-2	P - P P R	NO NO NO NO YES, NOTE 1
2-MCR-943-3	0	DA	3	A	J/6	0	C	2	A	A	EF-1 EF-5 EF-7 ET-004 SLT-1	EF-1 EF-5 EF-7 ET-004 SLT-2	P - P P R	NO NO NO NO YES, NOTE 1



DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLOW DIAGRAM: 2-5114A-27

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM-NAME: NON-ESSENTIAL SERVICE WATER

VALVE

VALVE POSITION

ASME SECTION XI

NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD	A/P	CAT	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)
2-MCR-929-3	0	DA	3	A	K/6	0	C	2	A	A	EF-1 EF-5 EF-7 ET-005 SLT-1	EF-1 EF-5 EF-7 ET-005 SLT-2	P - P P R	NO NO NO NO YES, NOTE 1
2-MCR-930-3	0	DA	3	A	K/2	0	C	2	A	A	EF-1 EF-5 EF-7 ET-004 SLT-1	EF-1 EF-5 EF-7 ET-004 SLT-2	P - P P R	NO NO NO NO YES, NOTE 1
2-MCR-931-3	0	DA	3	A	J/2	0	C	2	A	A	EF-1 EF-5 EF-7 ET-004 SLT-1	EF-1 EF-5 EF-7 ET-004 SLT-2	P - P P R	NO NO NO NO YES, NOTE 1
2-MCR-932-4	0	DA	3	A	J/6	0	C	2	A	A	EF-1 EF-5 EF-7 ET-004 SLT-1	EF-1 EF-5 EF-7 ET-004 SLT-2	P - P P R	NO NO NO NO YES, NOTE 1
2-MCR-933	0	DA	3	A	K/6	0	C	2	A	A	EF-1 EF-5 EF-7 ET-005 SLT-1	EF-1 EF-5 EF-7 ET-005 SLT-2	P - P P R	NO NO NO NO YES, NOTE 1

DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLOW DIAGRAM: 2-5114A-27

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM-NAME: NON-ESSENTIAL SERVICE WATER

VALVE				VALVE POSITION				ASME SECTION XI						
NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD A/P CL	A/P	CAT	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)
2-MCR-924-2	0	DA	3	A	J/6	0	C	2	A	A	EF-1	EF-1	P	NO
											EF-5	EF-5	-	NO
											EF-7	EF-7	P	NO
											ET-005	ET-005	P	NO
											SLT-1	SLT-2	R	YES, NOTE 1
2-MCR-925-2	0	DA	3	A	K/6	0	C	2	A	A	EF-1	EF-1	P	NO
											EF-5	EF-5	-	NO
											EF-7	EF-7	P	NO
											ET-005	ET-005	P	NO
											SLT-1	SLT-2	R	YES, NOTE 1
2-MCR-926-2	0	DA	3	A	K/2	0	C	2	A	A	EF-1	EF-1	P	NO
											EF-5	EF-5	-	NO
											EF-7	EF-7	P	NO
											ET-005	ET-005	P	NO
											SLT-1	SLT-2	R	YES, NOTE 1
2-MCR-927-2	0	DA	3	A	J/2	0	C	2	A	A	EF-1	EF-1	P	NO
											EF-5	EF-5	-	NO
											EF-7	EF-7	P	NO
											ET-005	ET-005	P	NO
											SLT-1	SLT-2	R	YES, NOTE 1
2-MCR-928-3	0	DA	3	A	J/6	0	C	2	A	A	EF-1	EF-1	P	NO
											EF-5	EF-5	-	NO
											EF-7	EF-7	P	NO
											ET-005	ET-005	P	NO
											SLT-1	SLT-2	R	YES, NOTE 1



DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLOW DIAGRAM: 2-5114A-27

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM-NAME: NON-ESSENTIAL-SERVICE-WATER

VALVE

VALVE POSITION

ASME SECTION XI

NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD A/P CL	CAT	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)	
2-NCR-915-4	0	DA	6	A	K/4	0	C	2	A	A	EF-1 EF-5 EF-7 ET-006 SLT-1	EF-1 EF-5 EF-7 ET-006 SLT-2	P - P P R	NO NO NO NO YES, NOTE 1
2-NCR-920-1	0	DA	3	A	J/6	0	C	2	A	A	EF-1 EF-5 EF-7 ET-005 SLT-1	EF-1 EF-5 EF-7 ET-005 SLT-2	P - P P R	NO NO NO NO YES, NOTE 1
2-NCR-921-1	0	DA	3	A	K/6	0	C	2	A	A	EF-1 EF-5 EF-7 ET-005 SLT-1	EF-1 EF-5 EF-7 ET-005 SLT-2	P - P P R	NO NO NO NO YES, NOTE 1
2-NCR-922-1	0	DA	3	A	K/2	0	C	2	A	A	EF-1 EF-5 EF-7 ET-005 SLT-1	EF-1 EF-5 EF-7 ET-005 SLT-2	P - P P R	NO NO NO NO YES, NOTE 1
2-NCR-923-1	0	DA	3	A	J/2	0	C	2	A	A	EF-1 EF-5 EF-7 ET-006 SLT-1	EF-1 EF-5 EF-7 ET-006 SLT-2	P - P P R	NO NO NO NO YES, NOTE 1

DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLOW DIAGRAM: 2-5114A-27

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM-NAME: NON-ESSENTIAL SERVICE WATER

VALVE

VALVE POSITION

ASME SECTION XI

NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD CL	A/P	CAT	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)
2-NCR-910-3	0	DA	6	A	J/4	0	C	2	A	A	EF-1 EF-5 EF-7 ET-006 SLT-1	EF-1 EF-5 EF-7 ET-006 SLT-2	P - P P R	NO NO NO NO YES, NOTE 1
2-NCR-911-3	0	DA	6	A	K/4	0	C	2	A	A	EF-1 EF-5 EF-7 ET-005 SLT-1	EF-1 EF-5 EF-7 ET-005 SLT-2	P - P P R	NO NO NO NO YES, NOTE 1
2-NCR-912-4	0	DA	6	A	J/9	0	C	2	A	A	EF-1 EF-5 EF-7 ET-009 SLT-1	EF-1 EF-5 EF-7 ET-009 SLT-2	P - P P R	NO NO NO NO YES, NOTE 1
2-NCR-913-4	0	DA	6	A	K/9	0	C	2	A	A	EF-1 EF-5 EF-7 ET-008 SLT-1	EF-1 EF-5 EF-7 ET-008 SLT-2	P - P P R	NO NO NO NO YES, NOTE 1
2-NCR-914-4	0	DA	6	A	J/4	0	C	2	A	A	EF-1 EF-5 EF-7 ET-005 SLT-1	EF-1 EF-5 EF-7 ET-005 SLT-2	P - P P R	NO NO NO NO YES, NOTE 1

DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLOW DIAGRAM: 2-5114A-27

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM NAME: NON-ESSENTIAL SERVICE WATER

VALVE				VALVE POSITION				ASME SECTION XI						
NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD	A/P	CAT	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)
2-MCR-905-2	0	DA	6	A	K/9	0	C	2	A	A	EF-1 EF-5 EF-7 ET-006 SLT-1	EF-1 EF-5 EF-7 ET-006 SLT-2	P - P P R	NO NO NO NO YES, NOTE 1
2-MCR-906-2	0	DA	6	A	J/4	0	C	2	A	A	EF-1 EF-5 EF-7 ET-006 SLT-1	EF-1 EF-5 EF-7 ET-006 SLT-2	P - P P R	NO NO NO NO YES, NOTE 1
2-MCR-907	0	DA	6	A	K/4	0	C	2	A	A	EF-1 EF-5 EF-7 ET-005 SLT-1	EF-1 EF-5 EF-7 ET-005 SLT-2	P - P P R	NO NO NO NO YES, NOTE 1
2-MCR-908-3	0	DA	6	A	J/9	0	C	2	A	A	EF-1 EF-5 EF-7 ET-008 SLT-1	EF-1 EF-5 EF-7 ET-008 SLT-2	P - P P R	NO NO NO NO YES, NOTE 1
2-MCR-909-3	0	DA	6	A	K/9	0	C	2	A	A	EF-1 EF-5 EF-7 ET-006 SLT-1	EF-1 EF-5 EF-7 ET-006 SLT-2	P - P P R	NO NO NO NO YES, NOTE 1

DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLOW DIAGRAM: 2-5114A-27

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM NAME: NON-ESSENTIAL SERVICE WATER

VALVE				VALVE POSITION				ASME SECTION XI						
NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD	A/P	CAT	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)
2-MCR-900-1	0	DA	6	A	J/9	O	C	2	A	A	EF-1	EF-1	P	NO
											EF-5	EF-5	-	NO
											EF-7	EF-7	P	NO
											ET-008	ET-008	P	NO
											SLT-1	SLT-2	R	YES, NOTE 1
2-MCR-901-1	0	DA	6	A	K/9	O	C	2	A	A	EF-1	EF-1	P	NO
											EF-5	EF-5	-	NO
											EF-7	EF-7	P	NO
											ET-008	ET-008	P	NO
											SLT-1	SLT-2	R	YES, NOTE 1
2-MCR-902-1	0	DA	6	A	J/4	O	C	2	A	A	EF-1	EF-1	P	NO
											EF-5	EF-5	-	NO
											EF-7	EF-7	P	NO
											ET-005	ET-005	P	NO
											SLT-1	SLT-2	R	YES, NOTE 1
2-MCR-903-1	0	DA	6	A	K/4	O	C	2	A	A	EF-1	EF-1	P	NO
											EF-5	EF-5	-	NO
											EF-7	EF-7	P	NO
											ET-006	ET-006	P	NO
											SLT-1	SLT-2	R	YES, NOTE 1
2-MCR-904-2	0	DA	6	A	J/9	O	C	2	A	A	EF-1	EF-1	P	NO
											EF-5	EF-5	-	NO
											EF-7	EF-7	P	NO
											ET-008	ET-008	P	NO
											SLT-1	SLT-2	R	YES, NOTE 1

DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLOW DIAGRAM: 2-5113-36

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM-NAME: ESSENTIAL SERVICE WATER

VALVE

VALVE POSITION

ASME SECTION XI

NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD CL	A/P	CAT	PRIN TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)
2-HRV-722-CD	0	3W	4	A	K/8	0	0	3	A	B	EF-1 EF-7 ET-NA	NOTE 3 EF-8 NOTE 3	P R -	YES, NOTE 3 YES, NOTE 3 YES, NOTE 3
2-HRV-724-CD	0	3W	4	A	M/8	0	0	3	A	B	EF-1 EF-7 ET-NA	NOTE 3 EF-8 NOTE 3	P R -	YES, NOTE 3 YES, NOTE 3 YES, NOTE 3
2-HRV-726-AB	0	3W	4	A	K/8	0	0	3	A	B	EF-1 EF-7 ET-NA	NOTE 3 EF-8 NOTE 3	P R -	YES, NOTE 3 YES, NOTE 3 YES, NOTE 3
2-HRV-728-AB	0	3W	4	A	M/8	0	0	3	A	B	EF-1 EF-7 ET-NA	NOTE 3 EF-8 NOTE 1	P R -	YES, NOTE 3 YES, NOTE 3 YES, NOTE 3

DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLOW DIAGRAM: 2-5113-36

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM-NAME: ESSENTIAL-SERVICE-WATER

VALVE				VALVE POSITION				ASME SECTION XI						
NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD A/P CL	CAT	PRIN TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)	
2-MMO-726	0	BF	6	MO	K/6	C	0	3	A B	EF-1 EF-5 ET-023	EF-1 EF-5 ET-023	P - P	NO NO NO	
2-MMO-728	0	BF	6	MO	L/6	C	0	3	A B	EF-1 EF-5 ET-024	EF-1 EF-5 ET-024	P - P	NO NO NO	
2-MMO-734	0	BF	16	MO	H/3	O/C	0	3	A B	EF-1 EF-5 ET-054	EF-1 EF-5 ET-054	P - P	NO NO NO	
2-MMO-738	0	BF	16	MO	K/3	O/C	0	3	A B	EF-1 EF-5 ET-055	EF-1 EF-5 ET-055	P - P	NO NO NO	
2-MMO-744	0	BF	4	MO	J/6	C	0	3	A B	EF-1 EF-5 ET-036	EF-1 EF-5 ET-036	P - P	NO NO NO	
2-MMO-753	0	BF	6	MO	M/5	C	0	3	A B	EF-1 EF-5 ET-024	EF-1 EF-5 ET-024	P - P	NO NO NO	
2-MMO-754	0	BF	4	MO	K/6	C	0	3	A B	EF-1 EF-5 ET-026	EF-1 EF-5 ET-026	P - P	NO NO NO	



DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLOW DIAGRAM: 2-5113-36

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM-NAME: ESSENTIAL SERVICE WATER

VALVE

VALVE POSITION

ASME SECTION XI

NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD CL	A/P	CAT	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)
2-WMO-704-2H	0	BF	20	MO	H/8	C	0	3	A	B	EF-1 EF-5 ET-054	EF-1 EF-5 ET-054	P - P	NO NO NO
2-WMO-706	0	BF	20	MO	G/8	0	0/C	3	A	B	EF-1 EF-5 ET-056	EF-1 EF-5 ET-056	P - P	NO NO NO
2-WMO-708	0	BF	20	MO	G/6	0	0/C	3	A	B	EF-1 EF-5 ET-055	EF-1 EF-5 ET-055	P - P	NO NO NO
2-WMO-714	0	BF	12	MO	M/5	C	0	3	A	B	EF-1 EF-5 ET-034	EF-1 EF-5 ET-034	P - P	NO NO NO
2-WMO-718	0	BF	12	MO	M/5	C	0	3	A	B	EF-1 EF-5 ET-036	EF-1 EF-5 ET-036	P - P	NO NO NO
2-WMO-722	0	BF	6	MO	M/6	C	0	3	A	B	EF-1 EF-5 ET-023	EF-1 EF-5 ET-023	P - P	NO NO NO
2-WMO-724	0	BF	6	MO	M/6	C	0	3	A	B	EF-1 EF-5 ET-020	EF-1 EF-5 ET-020	P - P	NO NO NO



DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLOW DIAGRAM: 2-5113-36

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM-NAME: ESSENTIAL-SERVICE-WATER

VALVE			VALVE POSITION			ASME SECTION XI								
NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD	A/P	CAT	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)
2-ESW-170-S	0	BF	3	M	F/1	O	C	3	A	B	EF-1	EF-1	P	NO
2-ESW-171-N	0	BF	3	M	F/1	C	O	3	A	B	EF-1	EF-1	P	NO
2-ESW-171-S	0	BF	3	M	F/1	C	O	3	A	B	EF-1	EF-1	P	NO
2-ESW-240	0	BF	6	M	M/5	C	O	3	A	B	EF-1	EF-2	R	YES, NOTE 2
2-ESW-243	0	BF	4	M	J/7	C	O	3	A	B	EF-1	EF-2	R	YES, NOTE 2
2-SV-14-2E	0	REL	1	SA	L/1	C	O	3	A	C	TF-1	TF-1	R	NO
2-SV-14-2N	0	REL	1	SA	M/1	C	O	3	A	C	TF-1	TF-1	R	NO
2-SV-15-2E	0	REL	0.75	SA	G/3	C	O	3	A	C	TF-1	TF-1	R	NO
2-SV-15-2N	0	REL	0.75	SA	J/4	C	O	3	A	C	TF-1	TF-1	R	NO
2-SV-16-AB	0	REL	1	SA	L/8	C	O	3	A	C	TF-1	TF-1	R	NO
2-SV-16-CD	0	REL	1	SA	L/8	C	O	3	A	C	TF-1	TF-1	R	NO
2-WW-703-2E	0	BF	20	NO	H/8	C	O	3	A	B	EF-1 EF-5 ET-053	EF-1 EF-5 ET-053	P - P	NO NO NO

DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLOW DIAGRAM: 2-5113-36

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM-NAME: ESSENTIAL-SERVICE-WATER

VALVE		VALVE POSITION		ASME SECTION XI											
NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD A/P CL	CAT	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)		
2-ESW-102-2E	0	CK	20	SA	H/8	C	0	3	A	C	CF-1	CF-1	P	NO	
2-ESW-102-2H	0	CK	20	SA	H/8	C	0	3	A	C	CF-1	CF-1	P	NO	
2-ESN-141	0	CK	6	SA	K/6	C	0	3	A	C	CF-1	CF-1	P	NO, NOTE 1	
2-ESW-142	0	CK	6	SA	L/6	C	0	3	A	C	CF-1	CF-1	P	NO, NOTE 1	
2-ESW-143	0	CK	6	SA	M/6	C	0	3	A	C	CF-1	CF-1	P	NO, NOTE 1	
2-ESW-144	0	CK	6	SA	M/6	C	0	3	A	C	CF-1	CF-1	P	NO, NOTE 1	
2-ESW-145	0	BF	4	M	J/6	C	0	3	A	B	EF-1	EF-2	R	YES, NOTE 2	
2-ESW-168-N	0	BF	3	M	H/1	C	0	3	A	B	EF-1	EF-1	P	NO	
2-ESW-168-S	0	BF	3	M	H/1	C	0	3	A	B	EF-1	EF-1	P	NO	
2-ESW-169-N	0	BF	3	M	G/1	O	C	3	A	B	EF-1	EF-1	P	NO	
2-ESW-169-S	0	BF	3	M	G/1	O	C	3	A	B	EF-1	EF-1	P	NO	
2-ESW-170-N	0	BF	3	M	F/1	O	C	3	A	B	EF-1	EF-1	P	NO	

DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLOW DIAGRAM: 2-5106A-41

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM-NAME: FEEDWATER

VALVE				VALVE POSITION				ASME SECTION XI						
NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD A/P CL	CAT	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)	
2-FW-160	2	CK	1	SA	M/9	C	O/C	3	A	C	CF-1	CF-1	P	NO, NOTE 5
2-FW-161	0	CK	8	SA	J/7	C	O	3	A	C	CF-1	CF-1	P	NO
2-SV-140-1	0	REL	0.75	SA	E/1	C	O	3	A	C	TF-1	TF-1	R	NO
2-SV-140-2	0	REL	0.75	SA	D/1	C	O	3	A	C	TF-1	TF-1	R	NO
2-SV-169-A	0	REL	0.75	SA	K/9	C	O	3	A	C	TF-1	TF-1	R	NO
2-SV-169-B	0	REL	0.75	SA	G/9	C	O	3	A	C	TF-1	TF-1	R	NO

DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLOW DIAGRAM: 2-5106A-41

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM-NAME: FEEDWATER

VALVE		VALVE POSITION		ASME SECTION XI										
NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD CL	A/P	CAT	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)
2-FW-132-3	2	CK	4	SA	D/5	C	O/C	2	A	C	CF-1	NOTE 1	-	YES, NOTE 1
2-FW-132-4	2	CK	4	SA	C/4	C	O/C	2	A	C	CF-1	NOTE 1	-	YES, NOTE 1
2-FW-134	0	CK	10	SA	B/9	C	0	3	A	C	CF-1	CF-1	P	NO, NOTE 2
2-FW-135	0	CK	8	SA	E/8	C	0	3	A	C	CF-1	CF-1	P	NO, NOTE 2
2-FW-138-1	2	CK	4	SA	C/4	C	O/C	2	A	C	CF-1	NOTE 3	-	YES, NOTE 3
2-FW-138-2	2	CK	4	SA	D/5	C	O/C	2	A	C	CF-1	NOTE 3	-	YES, NOTE 3
2-FW-138-3	2	CK	4	SA	D/5	C	O/C	2	A	C	CF-1	NOTE 3	-	YES, NOTE 3
2-FW-138-4	2	CK	4	SA	C/4	C	O/C	2	A	C	CF-1	NOTE 3	-	YES, NOTE 3
2-FW-149	0	CK	0.75	SA	D/2	C	0	3	A	C	CF-1	CF-1	P	NO, NOTE 4
2-FW-150	0	CK	0.75	SA	D/3	C	0	3	A	C	CF-1	CF-1	P	NO, NOTE 4
2-FW-153	2	CK	1	SA	H/9	C	O/C	3	A	C	CF-1	CF-1	P	NO, NOTE 5
2-FW-159	0	CK	6	SA	L/7	C	0	3	A	C	CF-1	CF-1	P	NO



DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLOW DIAGRAM: 2-5106A-41

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM NAME: FEEDWATER

VALVE				VALVE POSITION				ASME SECTION XI						
NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD A/P CL	CAT	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)	
2-FMO-242	0	GL	4	MO	B/4	O	O	3	A B	EF-1 EF-5 ET-030	EF-1 EF-5 ET-030	P - P	NO NO NO	
2-FRV-247	0	GL	1	A	M/9	C	C/O	3	A B	EF-1 EF-5 EF-7 ET-008	EF-1 EF-5 EF-7 ET-008	P - P P	NO NO NO NO	
2-FRV-257	0	GL	1	A	H/9	C	C/O	3	A B	EF-1 EF-5 EF-7 ET-005	EF-1 EF-5 EF-7 ET-005	P - P P	NO NO NO NO	
2-FRV-258	0	GL	1	A	E/9	C	C/O	3	A B	EF-1 EF-5 EF-7 ET-005	EF-1 EF-5 EF-7 ET-005	P - P P	NO NO NO NO	
2-FW-124	0	CK	8	SA	F/7	C	O	3	A C	CF-1	CF-1	P	NO	
2-FW-128	0	CK	6	SA	H/7	C	O	3	A C	CF-1	CF-1	P	NO	
2-FW-132-1	2	CK	4	SA	C/4	C	O/C	2	A C	CF-1	NOTE 1	-	YES, NOTE 1	
2-FW-132-2	2	CK	4	SA	D/5	C	O/C	2	A C	CF-1	NOTE 1	-	YES, NOTE 1	

DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLOW DIAGRAM: 2-5106A-41

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM-NAME: FEEDWATER

VALVE				VALVE POSITION				ASME SECTION XI							
NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD A/P CL	CAT	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)		
2-FMO-211	0	GL	4	MO	B/4	0	0	3	A B	EF-1 EF-5 ET-025	EF-1 EF-5 ET-025	P - P	NO NO NO		
2-FMO-212	0	GL	4	MO	B/4	0	0	3	A B	EF-1 EF-5 ET-031	EF-1 EF-5 ET-031	P - P	NO NO NO		
2-FMO-221	0	GL	4	MO	C/5	0	0	3	A B	EF-1 EF-5 ET-021	EF-1 EF-5 ET-021	P - P	NO NO NO		
2-FMO-222	0	GL	4	MO	C/5	0	0	3	A B	EF-1 EF-5 ET-036	EF-1 EF-5 ET-036	P - P	NO NO NO		
2-FMO-231	0	GL	4	MO	C/5	0	0	3	A B	EF-1 EF-5 ET-026	EF-1 EF-5 ET-026	P - P	NO NO NO		
2-FMO-232	0	GL	4	MO	C/5	0	0	3	A B	EF-1 EF-5 ET-034	EF-1 EF-5 ET-034	P - P	NO NO NO		
2-FMO-241	0	GL	4	MO	B/4	0	0	3	A B	EF-1 EF-5 ET-024	EF-1 EF-5 ET-024	P - P	NO NO NO		



DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLOW DIAGRAM: 2-5106-34

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM-NAME: FEEDWATER

VALVE

VALVE POSITION

ASME SECTION XI

NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD CL	A/P	CAT	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)
2-FRV-240	0	AG	14	A	H/5	0	C	2	A	B	EF-1 EF-5 EF-7 ET-005	EF-2 EF-5 EF-8 ET-005	C - C C	YES, NOTE 1 NO NO, NOTE 1 NO

DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLOW DIAGRAM: 2-5106-34

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM NAME: FEEDWATER

VALVE				VALVE POSITION				ASME SECTION XI						
NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD	A/P	CAT	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)
2-FMO-201	0	GA	14	MO	G/5	0	C	2	A	B	EF-1 EF-5 ET-020	EF-2 EF-5 ET-020	C - C	YES, NOTE 1 NO NO
2-FMO-202	0	GA	14	MO	E/9	0	C	2	A	B	EF-1 EF-5 ET-018	EF-2 EF-5 ET-018	C - C	YES, NOTE 1 NO NO
2-FMO-203	0	GA	14	MO	F/9	0	C	2	A	B	EF-1 EF-5 ET-019	EF-2 EF-5 ET-019	C - C	YES, NOTE 1 NO NO
2-FMO-204	0	GA	14	MO	H/5	0	C	2	A	B	EF-1 EF-5 ET-019	EF-2 EF-5 ET-019	C - C	YES, NOTE 1 NO NO
2-FRV-210	0	AG	14	A	G/5	0	C	2	A	B	EF-1 EF-5 EF-7 ET-005	EF-2 EF-5 EF-8 ET-005	C - C C	YES, NOTE 1 NO NO, NOTE 1 NO
2-FRV-220	0	AG	14	A	E/9	0	C	2	A	B	EF-1 EF-5 EF-7 ET-005	EF-2 EF-5 EF-8 ET-005	C - C C	YES, NOTE 1 NO NO, NOTE 1 NO
2-FRV-230	0	AG	14	A	F/9	0	C	2	A	B	EF-1 EF-5 EF-7 ET-005	EF-2 EF-5 EF-8 ET-005	C - C C	YES, NOTE 1 NO NO, NOTE 1 NO

DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLOW DIAGRAM: 2-5105D-2

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM NAME: STEAM GENERATING SYSTEM

VALVE				VALVE POSITION				ASME SECTION XI						
NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD A/P CL	CAT	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)	
2-SV-3-1	0	REL	6	SA	B/5	C	0	2	A C	TF-1	TF-1	R	NO	
2-SV-3-2	0	REL	6	SA	M/5	C	0	2	A C	TF-1	TF-1	R	NO	
2-SV-3-3	0	REL	6	SA	M/1	C	0	2	A C	TF-1	TF-1	R	NO	
2-SV-3-4	0	REL	6	SA	B/1	C	0	2	A C	TF-1	TF-1	R	NO	

DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLOW DIAGRAM: 2-5105D-2

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM-NAME: STEAM-GENERATING-SYSTEM

VALVE

VALVE POSITION

ASME SECTION XI

NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD CL	A/P	CAT	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)
2-SV-1B-1	0	REL	6	SA	B/5	C	0	2	A	C	TF-1	TF-1	R	NO
2-SV-1B-2	0	REL	6	SA	L/5	C	0	2	A	C	TF-1	TF-1	R	NO
2-SV-1B-3	0	REL	6	SA	L/1	C	0	2	A	C	TF-1	TF-1	R	NO
2-SV-1B-4	0	REL	6	SA	B/1	C	0	2	A	C	TF-1	TF-1	R	NO
2-SV-2A-1	0	REL	6	SA	B/5	C	0	2	A	C	TF-1	TF-1	R	NO
2-SV-2A-2	0	REL	6	SA	L/5	C	0	2	A	C	TF-1	TF-1	R	NO
2-SV-2A-3	0	REL	6	SA	L/1	C	0	2	A	C	TF-1	TF-1	R	NO
2-SV-2A-4	0	REL	6	SA	B/1	C	0	2	A	C	TF-1	TF-1	R	NO
2-SV-2B-1	0	REL	6	SA	B/5	C	0	2	A	C	TF-1	TF-1	R	NO
2-SV-2B-2	0	REL	6	SA	L/5	C	0	2	A	C	TF-1	TF-1	R	NO
2-SV-2B-3	0	REL	6	SA	L/1	C	0	2	A	C	TF-1	TF-1	R	NO
2-SV-2B-4	0	REL	6	SA	B/1	C	0	2	A	C	TF-1	TF-1	R	NO

DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLOW DIAGRAM: 2-5105D-2

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM-NAME: STEAM-GENERATING-SYSTEM

VALVE				VALVE POSITION				ASME SECTION XI									
NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD	A/P	CAT	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)			
2-MRV-240	1	GA	28	PO	B/3	O	C	2	A	B	EF-1 EF-5 EF-7 ET-005	EF-3 EF-5 EF-8 ET-005	P - C C	NO, NOTE 2 NO NO, NOTE 2 NO			
2-MRV-241	0	AG	2	A	A/1	C	O	2	A	B	EF-1 EF-5 EF-7 ETF-002	EF-1 EF-5 EF-7 ETF-002	P - P P	NO NO NO NO			
2-MRV-242	0	AG	2	A	A/1	C	O	2	A	B	EF-1 EF-5 EF-7 ETF-002	EF-1 EF-5 EF-7 ETF-002	P - P P	NO NO NO NO			
2-MS-108-2	2	CK	4	SA	K/4	C	O/C	3	A	C	CF-1	CF-2	R	YES, NOTE 3			
2-MS-108-3	2	CK	4	SA	K/4	C	O/C	3	A	C	CF-1	CF-2	R	YES, NOTE 3			
2-SV-1A-1	0	REL	6	SA	C/5	C	O	2	A	C	TF-1	TF-1	R	NO			
2-SV-1A-2	0	REL	6	SA	K/5	C	O	2	A	C	TF-1	TF-1	R	NO			
2-SV-1A-3	0	REL	6	SA	K/1	C	O	2	A	C	TF-1	TF-1	R	NO			
2-SV-1A-4	0	REL	6	SA	C/1	C	O	2	A	C	TF-1	TF-1	R	NO			

DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLOW DIAGRAM: 2-5105D-2

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM-NAME: STEAM-GENERATING-SYSTEM

VALVE				VALVE POSITION				ASME SECTION XI									
NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD A/P CL	CAT	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)				
2-MRV-220	0	GA	28	PO	L/7	0	C	2	A	B	EF-1 EF-5 EF-7 ET-005	EF-3 EF-5 EF-8 ET-005	P - C C	NO, NOTE 2 NO NO, NOTE 2 NO			
2-MRV-221	0	AG	2	A	M/5	C	0	2	A	B	EF-1 EF-5 EF-7 ET-003	EF-1 EF-5 EF-7 ET-003	P - P P	NO NO NO NO			
2-MRV-222	0	AG	2	A	M/5	C	0	2	A	B	EF-1 EF-5 EF-7 ET-003	EF-1 EF-5 EF-7 ET-003	P - P P	NO NO NO NO			
2-MRV-230	0	GA	28	PO	M/3	0	C	2	A	B	EF-1 EF-5 EF-7 ET-005	EF-3 EF-5 EF-8 ET-005	P - C C	NO, NOTE 2 NO NO, NOTE 2 NO			
2-MRV-231	0	AG	2	A	M/1	C	0	2	A	B	EF-1 EF-5 EF-7 ETF-002	EF-1 EF-5 EF-7 ETF-002	P - P P	NO NO NO NO			
2-MRV-232	0	AG	2	A	M/1	C	0	2	A	B	EF-1 EF-5 EF-7 ET-003	EF-1 EF-5 EF-7 ET-003	P - P P	NO NO NO NO			

DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLOW DIAGRAM: 2-5105D-2

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM-NAME: STEAM GENERATING SYSTEM

VALVE				VALVE POSITION				ASME SECTION XI						
NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD A/P CL	CAT	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)	
2-FN-118-1	0	CK	14	SA	B/9	0	C	2	A	C	CF-1	CF-2	R	YES, NOTE 1
2-FN-118-2	2	CK	14	SA	K/8	0	C	2	A	C	CF-1	CF-2	R	YES, NOTE 1
2-FN-118-3	0	CK	14	SA	J/3	0	C	2	A	C	CF-1	CF-2	R	YES, NOTE 1
2-FN-118-4	0	CK	14	SA	C/3	0	C	2	A	C	CF-1	CF-2	R	YES, NOTE 1
2-MCM-221	0	GL	4	MO	K/4	0	O/C	2	A	B	EF-1 EF-5 ET-049	EF-1 EF-5 ET-049	P - P	NO NO NO
2-MCM-231	0	GL	4	MO	K/4	0	O/C	2	A	B	EF-1 EF-5 ET-050	EF-1 EF-5 ET-050	P - P	NO NO NO
2-MRV-210	0	GA	28	PO	B/7	0	C	2	A	B	EF-1 EF-5 EF-7 ET-005	EF-3 EF-5 EF-8 ET-005	P - C C	NO, NOTE 2 NO NO, NOTE 2 NO
2-MRV-211	0	AG	2	A	A/5	C	O	2	A	B	EF-1 EF-5 EF-7 ET-003	EF-1 EF-5 EF-7 ET-003	P - P P	NO NO NO NO
2-MRV-212	0	AG	2	A	A/5	C	O	2	A	B	EF-1 EF-5 EF-7 ET-004	EF-1 EF-5 EF-7 ET-004	P - P P	NO NO NO NO

DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLOW DIAGRAM: 2-5105B-42

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM-NAME: MAIN-STEAM

VALVE				VALVE POSITION				ASME SECTION XI							
NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD	A/P	CAT	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)	
2-DCR-310	2	GL	2	A	B/3	O	C	2	A	B	EF-1	EF-1	P	NO	
											EF-5	EF-5	-	NO	
											EF-7	EF-7	P	NO	
											ET-006	ET-006	P	NO	
2-DCR-320	2	GL	2	A	B/2	O	C	2	A	B	EF-1	EF-1	P	NO	
											EF-5	EF-5	-	NO	
											EF-7	EF-7	P	NO	
											ET-006	ET-006	P	NO	
2-DCR-330	2	GL	2	A	B/1	O	C	2	A	B	EF-1	EF-1	P	NO	
											EF-5	EF-5	-	NO	
											EF-7	EF-7	P	NO	
											ET-008	ET-008	P	NO	
2-DCR-340	2	GL	2	A	B/2	O	C	2	A	B	EF-1	EF-1	P	NO	
											EF-5	EF-5	-	NO	
											EF-7	EF-7	P	NO	
											ET-009	ET-009	P	NO	
2-QT-506	0	GL	4	MO	A/8	C	O	3	A	B	EF-1	EF-1	P	NO	
											EF-5	EF-5	-	NO	
											ET-023	ET-023	P	NO	

DONALD C. COOK NUCLEAR PLANT
 SECOND TEN YEAR INTERVAL
 VALVE SUMMARY SHEET - UNIT 2
 FLOW DIAGRAM: 12-5141F-6

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM NAME: PAL SAMPLING & INST. PANELS U-2

VALVE

VALVE POSITION

ASME SECTION XI

NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD A/P	CAT	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)
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2-SM-1	0	CK	1	SA	A/3	0	C	2	A	AC	CF-1 SLT-1	CF-2 SLT-2	R R	YES, NOTE 3 YES, NOTE 2
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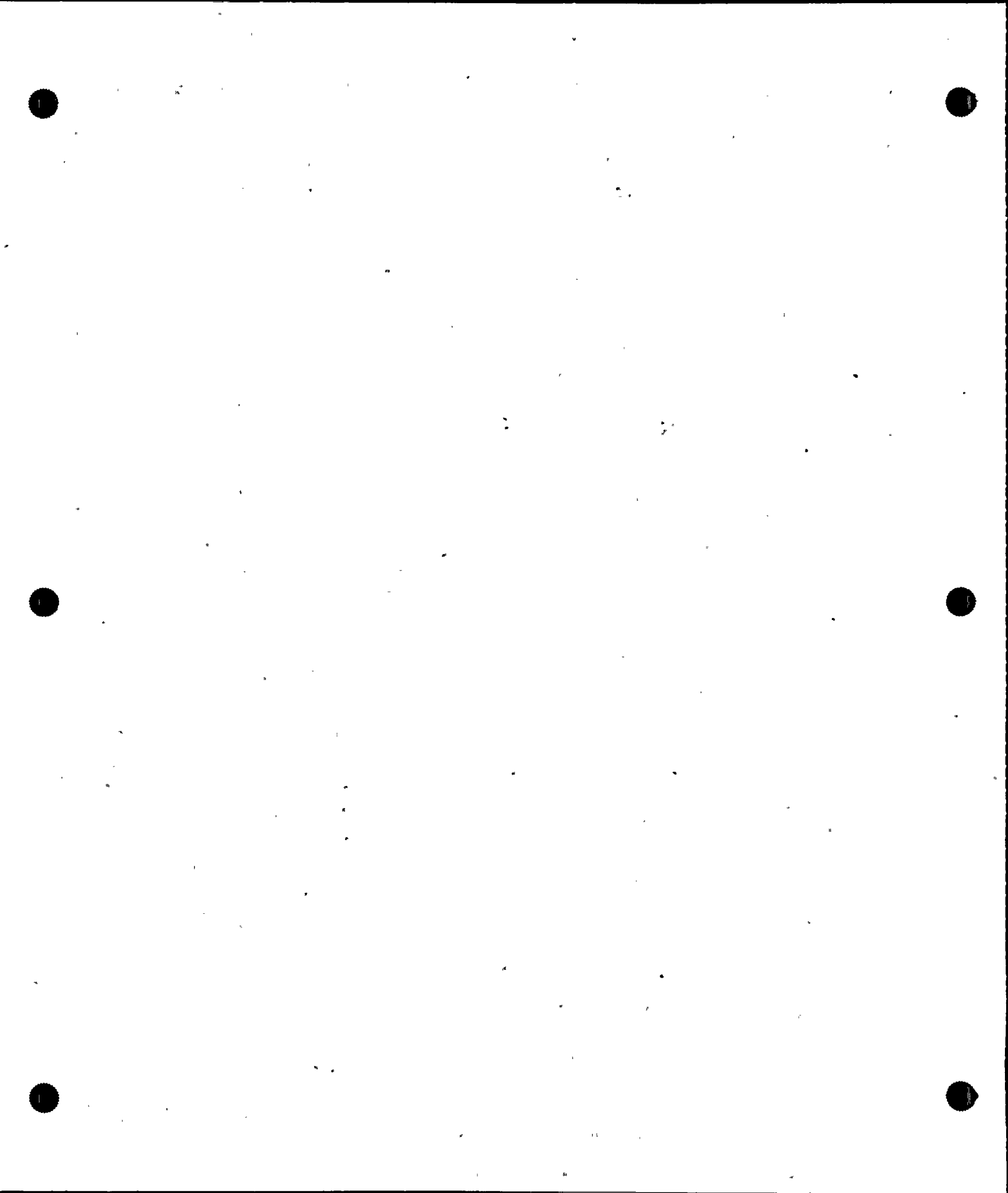


DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLOW DIAGRAM: 12-5141F-6

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM NAME: PAL SAMPLING & INST. PANELS - U-2

VALVE				VALVE POSITION				ASME SECTION XI							
NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD A/P CL	CAT	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)		
2-ECR-31	2	GL	1	A	B/3	O	C	2	A	A	EF-1	EF-1	P	NO	
											EF-5	EF-5	-	NO	
											EF-7	EF-7	P	NO	
											ET-005	ET-005	P	NO	
											SLT-1	SLT-2	R	YES, NOTE 2	
2-ECR-32	2	GL	1	A	B/5	O	C	2	A	A	EF-1	EF-1	P	NO	
											EF-5	EF-5	-	NO	
											EF-7	EF-7	P	NO	
											ET-005	ET-005	P	NO	
											SLT-1	SLT-2	R	YES, NOTE 2	
2-ECR-33	2	GL	0.75	A	B/2	O	C	2	A	A	EF-1	EF-1	P	NO	
											EF-5	EF-5	-	NO	
											EF-7	EF-7	P	NO	
											ET-004	ET-004	P	NO	
											SLT-1	SLT-2	R	YES, NOTE 2	
2-ECR-35	2	GL	1	A	B/2	O	C	2	A	A	EF-1	EF-1	P	NO	
											EF-5	EF-5	-	NO	
											EF-7	EF-7	P	NO	
											ET-004	ET-004	P	NO	
											SLT-1	SLT-2	R	YES, NOTE 2	
2-ECR-36	2	GL	1	A	B/3	O	C	2	A	A	EF-1	EF-2	C	YES, NOTE 1	
											EF-5	EF-5	-	NO	
											EF-7	EF-8	C	NO, NOTE 1	
											ET-004	ET-004	C	YES, NOTE 1	
											SLT-1	SLT-2	R	YES, NOTE 2	



DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLOW DIAGRAM: 12-5141C-8

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM-NAME: PAS-LIQUID & GAS - UNIT-2

VALVE

VALVE POSITION

ASME SECTION XI

NUMBER	REV	TYPE	SIZE	ACT F.D. TYPE COORD	POWER OPER	SAFETY FUNCT	CD A/P CL	CAT	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)
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2-ECR-536	0	GL	0.5	A L/2	C	C	2 P A		EF-1 EF-5 EF-7 ET-003 SLT-1	EF-1 EF-5 EF-7 ET-003 SLT-2	P - P P R	NO NO NO NO YES, NOTE 1
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DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLOW DIAGRAM: 12-5141C-8

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM-NAME: PAS-LIQUID & GAS - UNIT-2

VALVE				VALVE POSITION				ASME SECTION XI						
NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD A/P CL	CAT	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)	
2-ECR-416	0	GL	0.5	A	M/6	C	C	2	P	A	EF-1 EF-5 EF-7 ETF-002 SLT-1	EF-1 EF-5 EF-7 ETF-002 SLT-2	P - P P R	NO NO NO NO YES, NOTE 1
2-ECR-417	0	GL	0.5	A	M/6	C	C	2	P	A	EF-1 EF-5 EF-7 ETF-002 SLT-1	EF-1 EF-5 EF-7 ETF-002 SLT-2	P - P P R	NO NO NO NO YES, NOTE 1
2-ECR-496	0	GL	0.5	A	M/8	C	C	2	P	A	EF-1 EF-5 EF-7 ETF-002 SLT-1	EF-1 EF-5 EF-7 ETF-002 SLT-2	P - P P R	NO NO NO NO YES, NOTE 1
2-ECR-497	0	GL	0.5	A	M/8	C	C	2	P	A	EF-1 EF-5 EF-7 ETF-002 SLT-1	EF-1 EF-5 EF-7 ETF-002 SLT-2	P - P P R	NO NO NO NO YES, NOTE 1
2-ECR-535	0	GL	0.5	A	M/2	C	C	2	P	A	EF-1 EF-5 EF-7 ET-003 SLT-1	EF-1 EF-5 EF-7 ET-003 SLT-2	P - P P R	NO NO NO NO YES, NOTE 1

DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLOW DIAGRAM: 12-5137A-21

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM-NAME: WDS-VENTS-&-DRAINS

VALVE

VALVE POSITION

ASME SECTION XI

NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD CL	A/P	CAT	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)
2-DCR-621	0	DA	1	A	N/9	0	C	2	A	A	EF-1 EF-5 EF-7 ET-005 SLT-1	EF-1 EF-5 EF-7 ET-005 SLT-2	P - P P R	NO NO NO NO YES, NOTE 1
2-N-160	0	CK	1	SA	F/4	0	C	2	A	AC	CF-1 SLT-1	CF-2 SLT-2	R R	YES, NOTE 2 YES, NOTE 1
2-SF-159	0	DA	3	M	E/5	C	C	2	P	A	SLT-1	SLT-2	R	YES, NOTE 1
2-SF-160	0	DA	3	M	F/5	C	C	2	P	A	SLT-1	SLT-2	R	YES, NOTE 1



DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLOW DIAGRAM: 12-5137A-21

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM-NAME: WDS-VENTS-&-DRAINS

VALVE

VALVE-POSITION

ASME SECTION XI

NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD CL	A/P	CAT	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)
2-DCR-206	0	GL	4	A	E/8	O/C	C	2	A	A	EF-1 EF-5 EF-7 ET-005 SLT-1	EF-1 EF-5 EF-7 ET-005 SLT-2	P - P P R	NO NO NO NO YES, NOTE 1
2-DCR-207	0	DA	1	A	F/4	O	C	2	A	A	EF-1 EF-5 EF-7 ETF-002 SLT-1	EF-1 EF-5 EF-7 ETF-002 SLT-2	P - P P R	NO NO NO NO YES, NOTE 1
2-DCR-610	0	DA	2.5	A	M/9	O	C	2	A	A	EF-1 EF-5 EF-7 ET-003 SLT-1	EF-1 EF-5 EF-7 ET-003 SLT-2	P - P P R	NO NO NO NO YES, NOTE 1
2-DCR-611	0	DA	2.5	A	N/9	O	C	2	A	A	EF-1 EF-5 EF-7 ET-004 SLT-1	EF-1 EF-5 EF-7 ET-004 SLT-2	P - P P R	NO NO NO NO YES, NOTE 1
2-DCR-620	0	DA	1	A	M/9	O	C	2	A	A	EF-1 EF-5 EF-7 ET-004 SLT-1	EF-1 EF-5 EF-7 ET-004 SLT-2	P - P P R	NO NO NO NO YES, NOTE 1

DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUPPLY SHEET - UNIT 2
FLOW DIAGRAM: 12-5137A-21

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM-NAME: WDS-VENTS-&-DRAINS

VALVE

VALVE POSITION

ASME SECTION XI

NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD CL	A/P	CAT	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)
2-DCR-201	0	DA	1	A	E/4	C	C	2	P	A	EF-1 EF-5 EF-7 ETF-002 SLT-1	EF-1 EF-5 EF-7 ETF-002 SLT-2	P - P P R	NO NO NO NO YES, NOTE 1
2-DCR-202	0	DA	0.75	A	E/5	0	C	2	A	A	EF-1 EF-5 EF-7 ETF-002 SLT-1	EF-1 EF-5 EF-7 ETF-002 SLT-2	P - P P R	NO NO NO NO YES, NOTE 1
2-DCR-203	0	DA	1	A	F/4	C	C	2	P	A	EF-1 EF-5 EF-7 ETF-002 SLT-1	EF-1 EF-5 EF-7 ETF-002 SLT-2	P - P P R	NO NO NO NO YES, NOTE 1
2-DCR-204	0	DA	0.75	A	F/5	0	C	2	A	A	EF-1 EF-5 EF-7 ET-003 SLT-1	EF-1 EF-5 EF-7 ET-003 SLT-2	P - P P R	NO NO NO NO YES, NOTE 1
2-DCR-205	0	GL	4	A	E/7	O/C	C	2	A	A	EF-1 EF-5 EF-7 ET-005 SLT-1	EF-1 EF-5 EF-7 ET-005 SLT-2	P - P P R	NO NO NO NO YES, NOTE 1



DONALD C. COOK NUCLEAR PLANT
 SECOND TEN YEAR INTERVAL
 VALVE SUMMARY SHEET - UNIT 2
 FLOW DIAGRAM: 12-5136-25

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM-NAME: SPENT-FUEL-PIT-COOLING & CLEANUP U2

VALVE

VALVE POSITION

ASME SECTION XI

NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD CL	A/P	CAT	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)
2-SF-152	0	DA	2.5	M	K/9	C	C	2	P	A	SLT-1	SLT-2	R	YES, NOTE 1
2-SF-154	0	GL	2.5	M	K/9	C	C	2	P	A	SLT-1	SLT-2	R	YES, NOTE 1



DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLOW DIAGRAM: 12-5131-19

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM-NAME: CVCS - BORON MAKE-UP - UNITS 1 & 2

VALVE		VALVE POSITION		ASME SECTION XI										
NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD	A/P	CAT	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)
2-CS-415-3	1	CK	2	SA	K/6	O/C	0	3	A	C	CF-1	CF-1	P	NO
2-CS-415-4	1	CK	2	SA	L/6	O/C	0	3	A	C	CF-1	CF-1	P	NO
2-CS-426-S	1	CK	1	SA	M/6	O/C	0	3	A	C	CF-1	CF-1	P	NO
2-CS-427-S	1	CK	2	SA	M/5	C	0	3	A	C	CF-1	CF-2	C	YES, NOTE 1
2-QMO-420	1	GA	2	MO	L/5	C	0	3	A	B	EF-1	EF-1	P	NO
											EF-5	EF-5	-	NO
											ET-012	ET-012	P	NO
2-QRV-421	1	GL	2	A	M/6	O/C	0	3	A	B	EF-1	EF-1	P	NO
											EF-5	EF-5	-	NO
											EF-7	EF-7	P	NO
											ET-006	ET-006	P	NO
2-QRV-422	1	GL	2	A	M/7	O	C	3	A	B	EF-1	EF-1	P	NO
											EF-5	EF-5	-	NO
											EF-7	EF-7	P	NO
											ET-005	ET-005	P	NO

DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLOW DIAGRAM: 12-5120B-22

RUN DATE AND TIME: 14SEP87:13:28

SYSTEM NAME: COMPRESSED AIR SYSTEM UNIT 2

VALVE				VALVE POSITION				ASME SECTION XI						
NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD	A/P	CAT	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)
2-PA-342	2	CK	2	SA	K/7	O/C	C	2	A	AC	CF-1 SLT-1	CF-2 SLT-2	R R	YES, NOTE 1 YES, NOTE 2
2-PCR-40	2	GA	2	A	H/7	O/C	C	2	A	A	EF-1 EF-5 EF-7 ET-003 SLT-1	EF-1 EF-5 EF-7 ET-003 SLT-2	P - P P R	NO NO NO NO YES, NOTE 2
2-XCR-100	0	GL	1	A	L/3	0	C	2	A	A	EF-1 EF-5 EF-7 ET-004 SLT-1	EF-2 EF-5 EF-8 ET-004 SLT-2	C - C C R	YES, NOTE 3 NO NO, NOTE 3 NO YES, NOTE 2
2-XCR-101	0	GL	1	A	L/3	0	C	2	A	A	EF-1 EF-5 EF-7 ET-005 SLT-1	EF-2 EF-5 EF-8 ET-005 SLT-2	C - C C R	YES, NOTE 3 NO NO, NOTE 3 NO YES, NOTE 2
2-XCR-102	0	GL	1	A	L/2	0	C	2	A	A	EF-1 EF-5 EF-7 ET-004 SLT-1	EF-2 EF-5 EF-8 ET-004 SLT-2	C - C C R	YES, NOTE 3 NO NO, NOTE 3 NO YES, NOTE 2
2-XCR-103	0	GL	1	A	L/2	0	C	2	A	A	EF-1 EF-5 EF-7 ET-004 SLT-1	EF-2 EF-5 EF-8 ET-004 SLT-2	C - C C R	YES, NOTE 3 NO NO, NOTE 3 NO YES, NOTE 2

DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLOW DIAGRAM: 12-5115A-41

RUN DATE AND TIME: 14SEP87:13:28

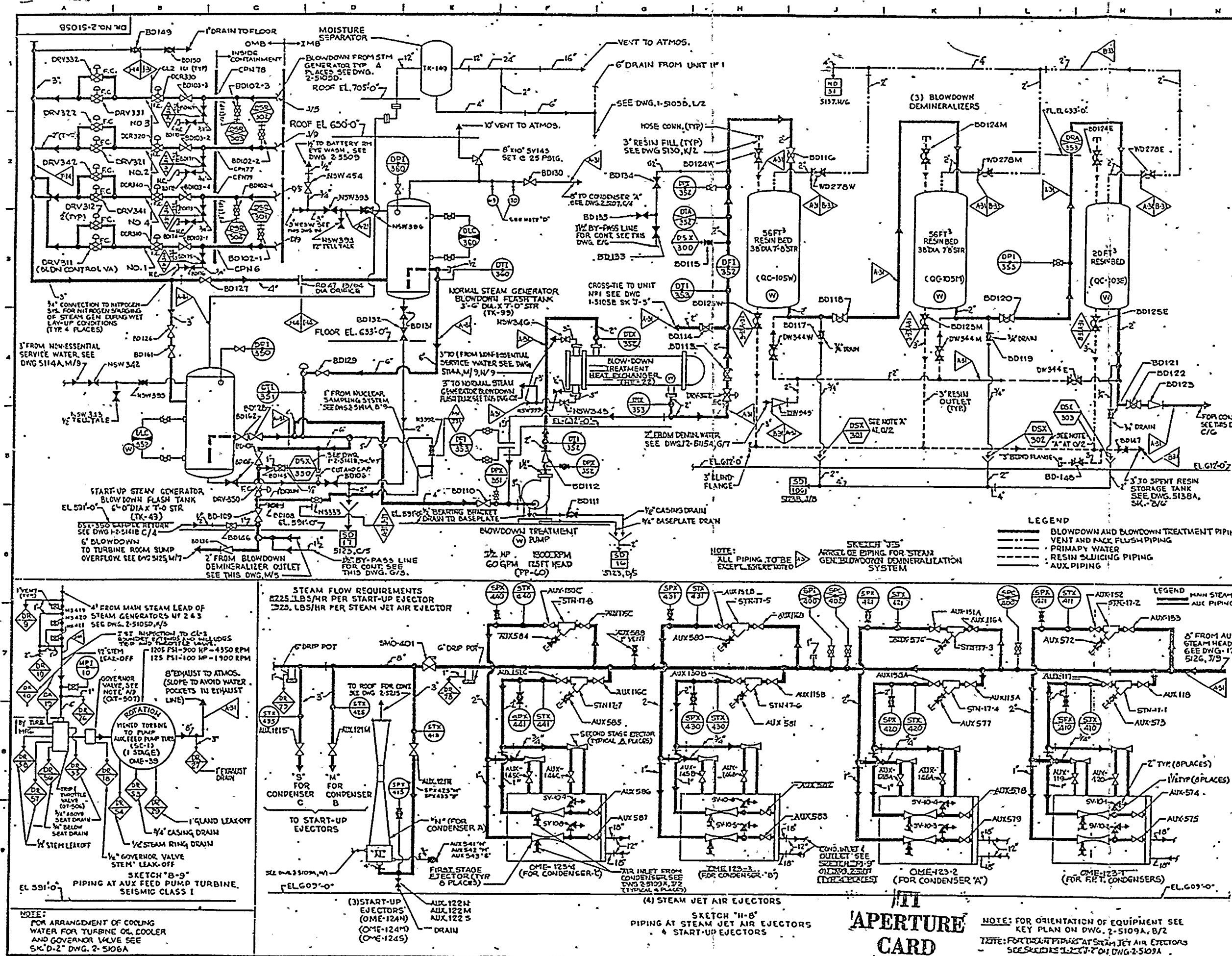
SYSTEM NAME: MAKE-UP & PRIMARY WATER UNIT 2

VALVE

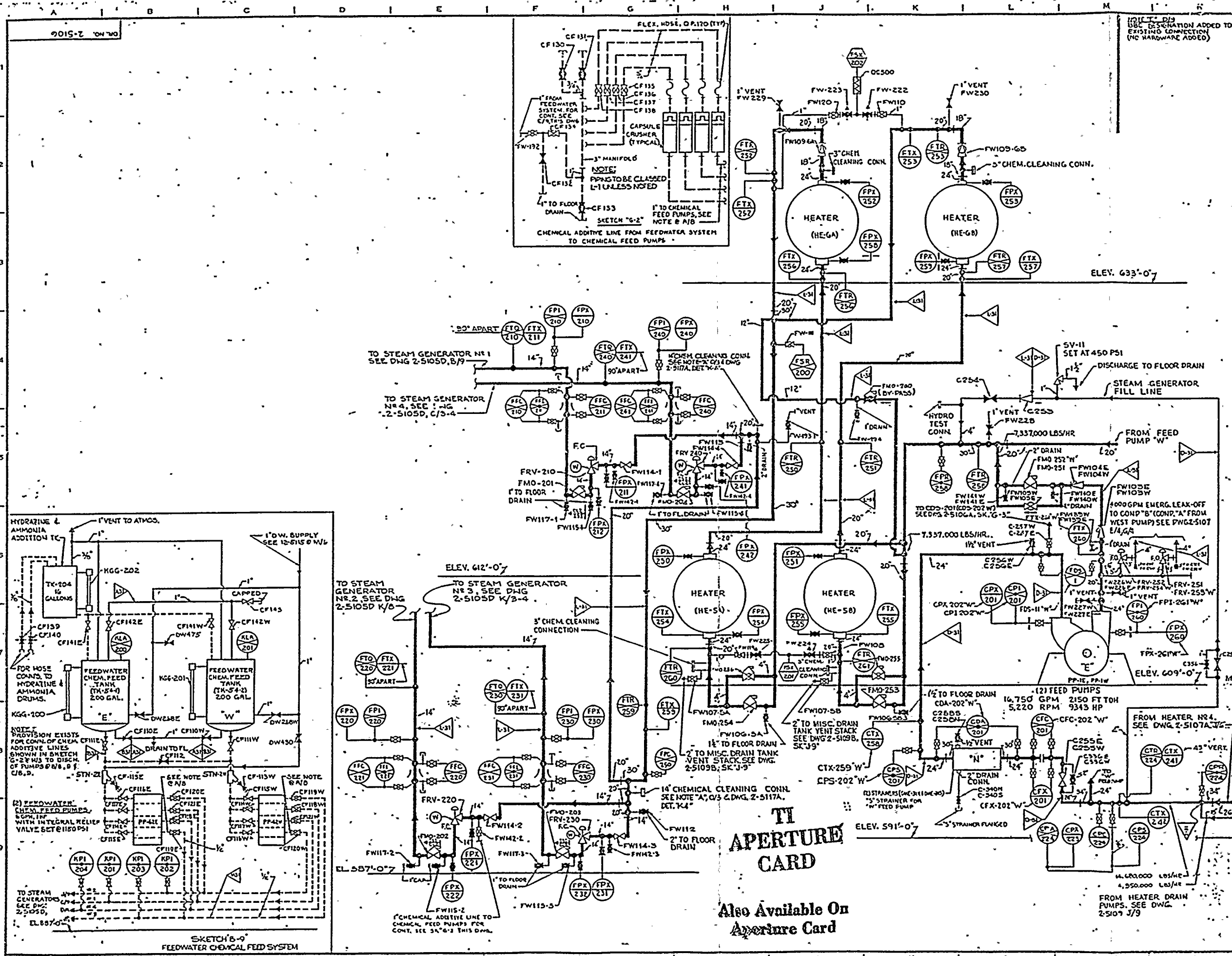
VALVE POSITION

ASME SECTION XI

NUMBER	REV	TYPE	SIZE	ACT TYPE	F.D. COORD	POWER OPER	SAFETY FUNCT	CD CL	A/P	CAT	PRIM TEST REQUIRED	TEST PERFORMED	TEST MODE	RELIEF REQUEST(S)
2-QCR-919	0	DA	2	A	D/7	O/C	C	2	A	A	EF-1 EF-5 EF-7 ET-004 SLT-1	EF-1 EF-5 EF-7 ET-004 SLT-2	P - P P R	NO NO NO NO YES, NOTE 1
2-QCR-920	0	DA	2	A	D/7	O/C	C	2	A	A	EF-1 EF-5 EF-7 ET-004 SLT-1	EF-2 EF-5 EF-7 ET-004 SLT-2	P - P P R	NO NO NO NO YES, NOTE 1



<p>GENERAL NOTES</p> <p>LEGEND AS NOTED</p> <p>FOR VALVE, INSTRUMENT, SAMPLING, PIPE MATERIAL AND OTHER SYMBOLS NOT EXPLAINED ON THIS DWG. AND FOR MARK NUMBER CODES, SEE DWGS. 12-5103 & 12-5104.</p> <p>(W) BY WESTINGHOUSE (IR) BY INGERSOLL RAND</p> <p>NOTE "A" SUFFICIENT ROOM PROVIDED AT DSX-301, 302, & 303 FOR GRAB SAMPLES</p> <p>NOTE "B" ALL EQUIPMENT SC II EXCEPT AS NOTED</p> <p>NOTE "C" FOR CLASS 2 INSTRUMENT LINES TO SEE EQUIPMENT EXTENDING TO AND INCLUDING THE FIRST STOP VALVE FOR CLASS 2 VENT & DRAIN LINES, THE SEE BOUNDARY EXTENDS TO AND INCLUDING THE FIRST NORMALLY CLOSED VALVE.</p> <p>NOTE "D" THIS SYMBOL INDICATES A TRACER INJECT ON TAP. THE NUMBER WITHIN THE CIRCLE INDICATES THE VALVE ID FOR THE CONNECTION. (E.G., NO. 13 SHOWN ON TAP VALVE LIST.)</p> <p>NOTE 1 THE UNIT PREFIX DESIGNATIONS FOR EACH COMPONENT IDENTIFICATION NUMBER IS "2" UNLESS OTHERWISE NOTED.</p>	
<p>MANUALLY OPERATED VALVE IDENTIFICATION NUMBERS</p> <p>1. ONLY "UNIQUE VALVE NUMBERS" APPEAR ON THIS DRAWING. SEE SEPARATE VALVE IDENTIFICATION LIST FOR EQUIVALENT DESIGN (MCR) NUMBERS.</p> <p>2. "TAG" NUMBERS MODIFIED FOR "DRAWING USE" AS FOLLOWS: TAG NO. 2-NSW-VIS-W APPEARS AS: NSW-VIS-W</p> <p>3. INSTRUMENT ROOT VALVE MARK "YS" NOT SHOWN ON DRAWING (SEE VALVE IDENTIFICATION LIST) DERIVED BY ADDING TO INSTRUMENT NUMBER FOR SINGLE PULSED VALVE FOR DOUBLE PULSED VALVE (STREAM-0, VALVE-1251030)</p>	
<p>DATE: 7-11-71 BY: JLC</p> <p>DATE: 10-1-71 BY: JLC</p> <p>FOR REVISION DESCRIPTION SEE SEPARATE REVISION RECORD FOR THIS DRAWING</p>	
<p>THIS DRAWING IS THE PROPERTY OF THE AMERICAN ELECTRIC POWER SERVICE CORP. AND IS LOANED TO YOU ON THE CONDITION THAT IT IS NOT TO BE REPRODUCED OR COPIED, IN WHOLE OR IN PART, OR USED FOR ANY PURPOSE OTHER THAN THAT FOR WHICH IT WAS ORIGINALLY PREPARED. THE WRITER ORIGINATOR OF THE AEP SERVICE CORP. DOES NOT ASSUME RESPONSIBILITY FOR THE USE OF THIS DRAWING, AND IT IS TO BE RETURNED TO THE ORIGINATOR.</p>	
<p>INDIANA & MICHIGAN ELECTRIC CO. DONALD C. COOK NUCLEAR PLANT.</p>	
<p>"FLOW DIAGRAM" MAIN STEAM UNIT NO. 2 SHEET 3 OF 3</p>	
<p>DR. NO. 2-51058-42</p>	



GENERAL NOTES

LEGEND

— FEEDWATER
 --- AUX. PIPING
 --- CHEM. FEED PIPING

FOR VALVE, INSTRUMENT, SAMPLING, PIPE MATERIAL, AND OTHER SYMBOLS NOT EXPLAINED ON THIS DWG., AND FOR MARK NUMBER CODES, SEE DWG. 11503412501

NOTE:
 ALL EQUIPMENT SEISMIC CLASS III EXCEPT AS NOTED

QUANTITIES PER G.E. CO. HEAT BALANCE 371H1316 V.W.O. REVISED 3-25-68 & B.B. CO. HEAT BALANCE D10240/D DATED 12-11-68 AT MAX. REACTOR POWER

NOTE "A": H/S, G/S CHEMICAL CLEANING CONNECTIONS PROVIDED FOR CONDENSATE FEEDWATER SYSTEM CHEMICAL CLEANING. SEE DWG. 2-51078 FOR MISC. COND. MATERIALS (9 VENTS, DRAIN, SYS)

NOTE "A": FOR CODE CLASS "Z" INSTRUMENT CONNECTIONS, THE 151 BOUNDARY EXTENDS TO AND INCLUDES THE FIRST ROOT VALVE.

NOTE "C": FOR CODE CLASS "Z" VENTS AND DRAINS THE 151 BOUNDARY EXTENDS TO AND INCLUDES THE FIRST NORMALLY CLOSED VALVE.

THE ENT. PREFIX DESIGNATION FOR EACH COMPONENT IDENTIFICATION NO. IS "Z" UNLESS OTHERWISE NOTED.

HAND OPERATED VALVE IDENTIFICATION NUMBERS

ONLY "UNIQUE VALVE NUMBERS" APPEAR ON THIS DRAWING. SEE SEPARATE VALVE IDENTIFICATION LIST FOR EQUIVALENT DESIGN (MCR) NUMBERS.

2. "TAG" NUMBERS MODIFIED FOR DRAWING USE AS FOLLOWS:
 TAG NO. 2-NSW-V105-W APPEARS AS: NSW105W

3. INSTRUMENT ROOT VALVE MARK IS NOT SHOWN ON DRAWING (SEE VALVE IDENTIFICATION LIST) DERIVED BY ADDING TO INSTRUMENT NUMBER:
 FOR SINGLE IMPULSE: V
 FOR DOUBLE IMPULSE: V2
 FOR DOUBLE IMPULSE: V2

DATE	NO.	APPROVED
2-25-67	34	12-50-68

FOR REVISION DESCRIPTION SEE SEPARATE REVISION RECORD FOR THIS DRAWING

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INDIANA & MICHIGAN ELECTRIC CO.
 DONALD C. COOK
 NUCLEAR PLANT
 BRIDGEMAN

FLOW DIAGRAM - FEEDWATER

UNIT NO. 2 - SHEET 1 OF 2

DWG. NO. - 2-5106-34

DATE: 2-25-67
 BY: J.B.
 CHECKED: J.B.
 APPROVED: J.B.

AMERICAN ELECTRIC POWER SERVICE CORP.
 2 BROADWAY
 NEW YORK

TI APERTURE CARD

Also Available On Aperture Card

"NOT FOR D. C. COOK OPERATIONAL USE"

Y9015-2
DR. NO. 2-5106A

THE UNIT PREFIX DESIGNATION
FOR EACH COMPONENT IDENTIFI-
CATION NUMBER IS "2" UNLESS
OTHERWISE NOTED.

GENERAL NOTES

- AUX. FEED WATER
- EMERGENCY LEAKOFF
- COOLING WATER
- AUX. PIPING
- TEST LINE

THIS DWG. MADE UNIQUE
FOR UNIT 2 FROM DWG.
12-5106A REV. 25.

FOR VALVE, INSTRUMENT, SAM-
PLING, PIPE MATERIAL & OTHER
SYMBOLS NOT EXPLAINED ON
THIS DWG. FOR MARK NUMBER
CODES, SEE DWG. 12-5103 (12-5104)

LINE IDENTIFICATION SYMBOLS

NOTE "A"
VALVES ALSO SHOWN (AND
NUMBERED) ON DWG. 5113 NOT
TO BE DUPLICATED, AND ARE
NORMALLY CLOSED.

NOTE "B"
6" FROM MISC. DRAIN TANK
USED FOR HEATING SYSTEM
OPERATION DURING CONSTRUCTION
AND ON OCCASIONS WHEN
BOTH UNITS ARE IN OPERATION

NOTE "C" EQUIPMENT SEISMIC CLASSIFI- CATION AS NOTED.

NOTE "D"
1. FOR CODE CLASS 2 & 3 INSTRUMENT
CONNECTIONS, THE 151
BOUNDARY EXTENDS TO AND
INCLUDES THE FIRST NORMALLY CLOSED
VALVE.

NOTE "E"
CL-110 LOCATED IN TADFP ROOM.

NOTE "F"
FOR TADFP STEAM SUPPLY
SEE DWG. 2-5105B, A/3.

NOTE "G"
FOR COOLING WATER SUPPLY TO
AUX. FEED PUMP BEARING
SEE DWG. 2-5114

HAND OPERATED VALVE IDENTIFICATION NUMBERS

1. EARLY UNIQUE VALVE NUMBERS
APPEAR ON THIS DWG. SEE
SEPARATE VALVE IDENTIFICATION
LIST FOR EQUIVALENT DESIGN
(MCR) NUMBERS.

2. TAG NUMBERS MODIFIED FOR
DWG. 12-5106A AS FOLLOWS:
TAG NR. 2-NSW-V105W
APPEARS AS NSW105W

3. INSTRUMENT ROOT VALVE
MARK NPS NOT SHOWN ON DWG.
(SEE VALVE IDENTIFICATION
LIST) DERIVED BY ADDING TO
INSTRUMENT NUMBER
FOR SINGLE IMPULSE: VI
FOR DOUBLE IMPULSE: V2
(UPSTREAM)
V2 (DOWNSTREAM)

4-5-37-41

DATE: 12-5106A
FOR REVISION DESCRIPTION SEE
SEPARATE REVISION RECORD
FOR THIS DRAWING

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INDIANA & MICHIGAN ELECTRIC CO.
DONALD C. COOK
NUCLEAR PLANT

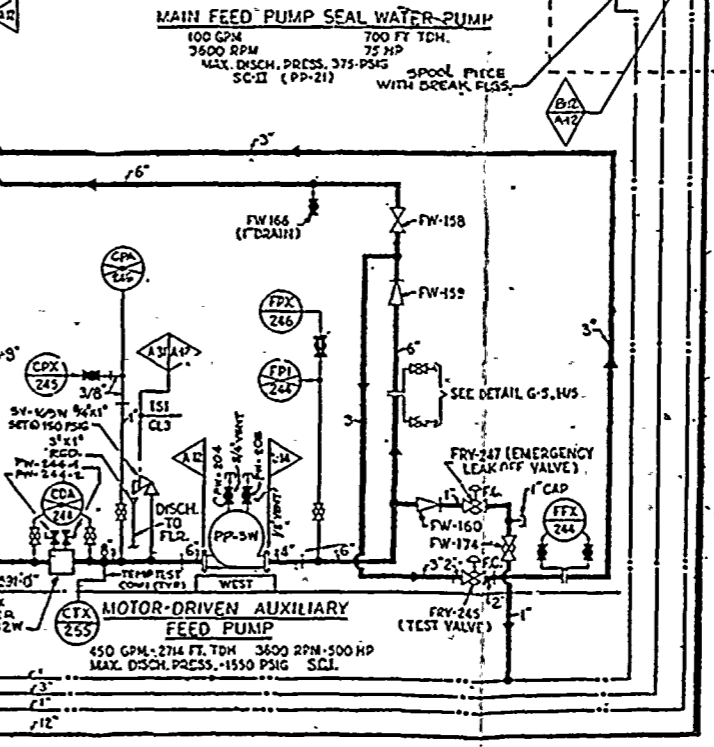
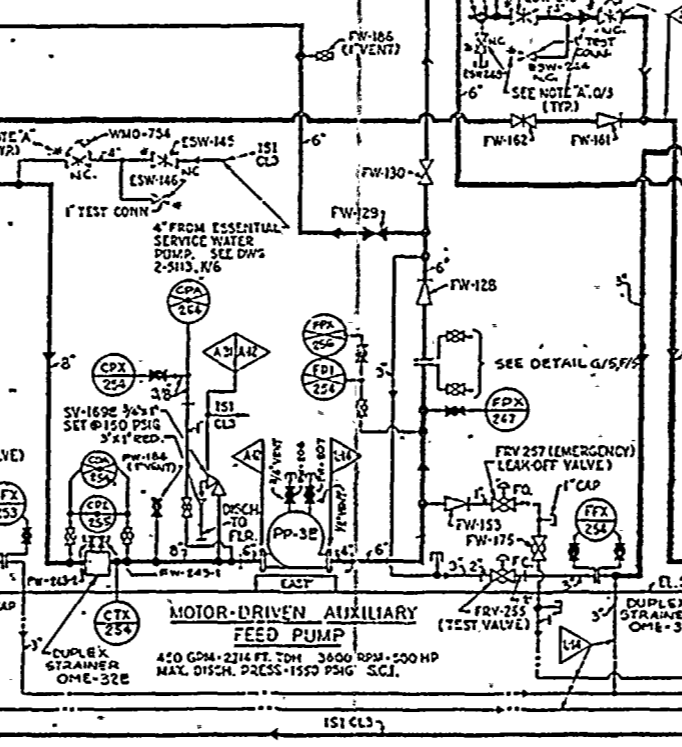
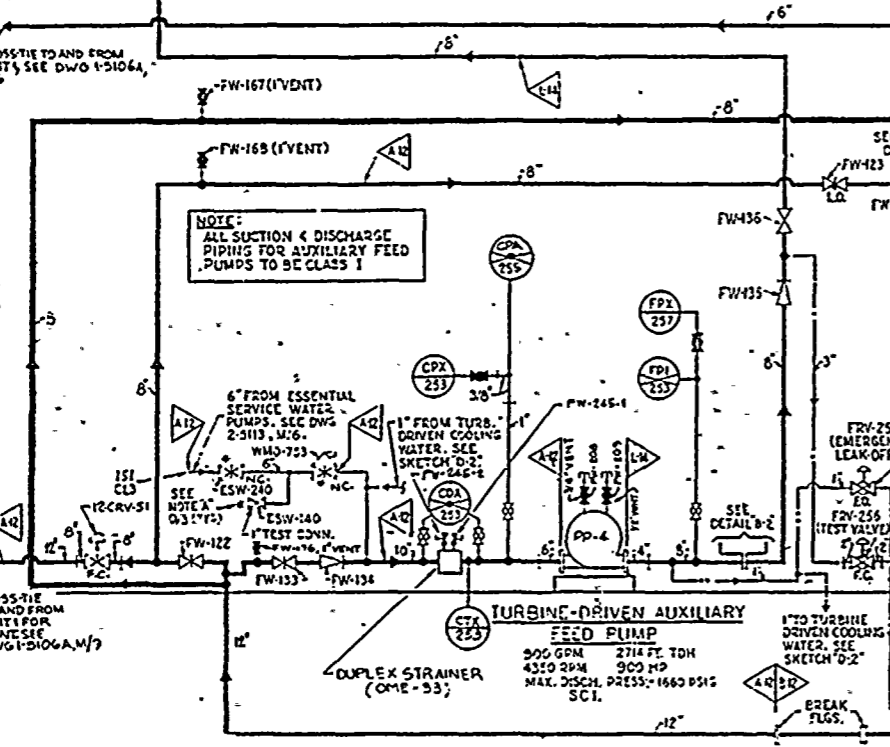
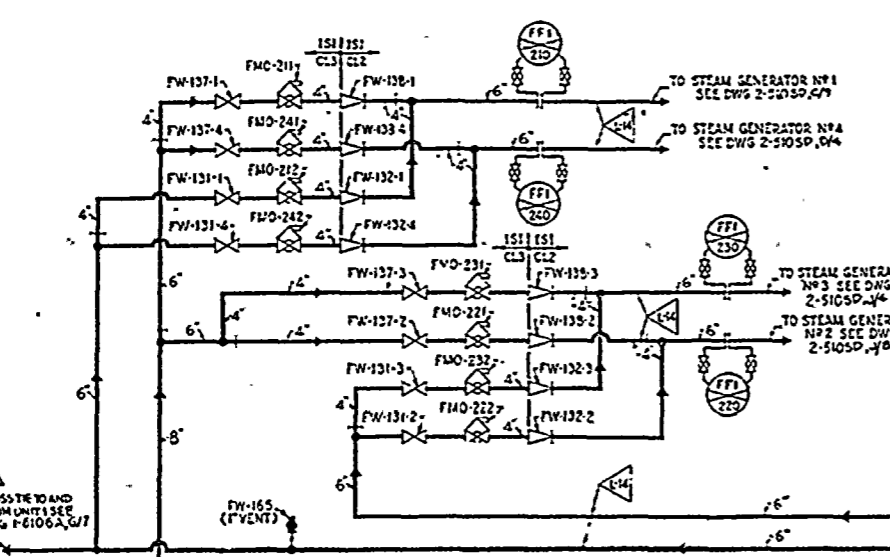
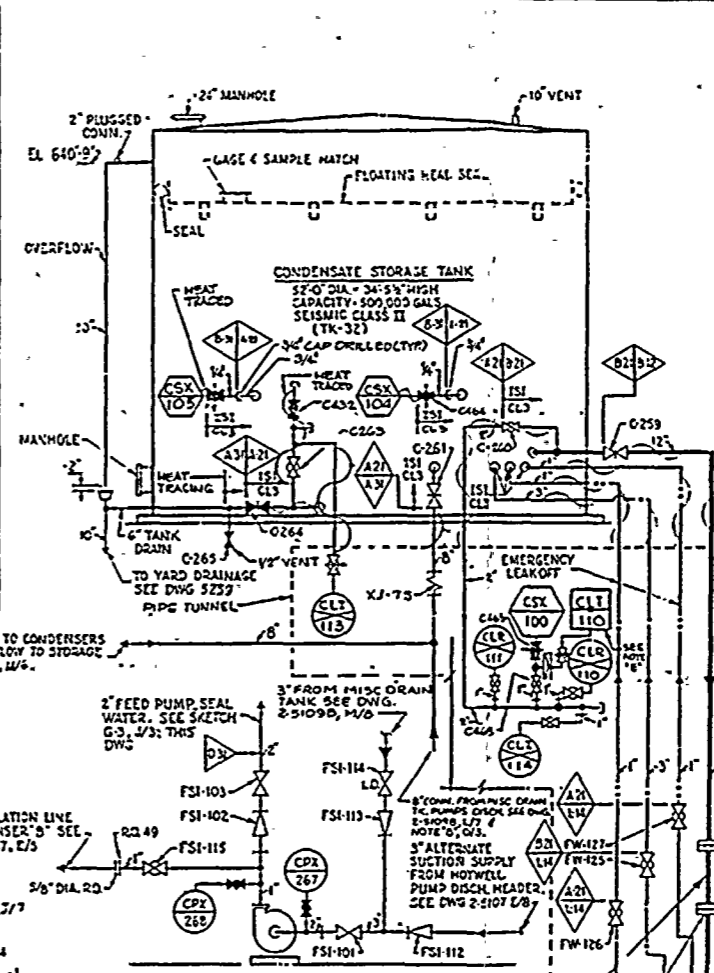
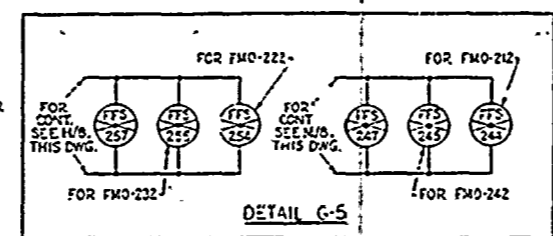
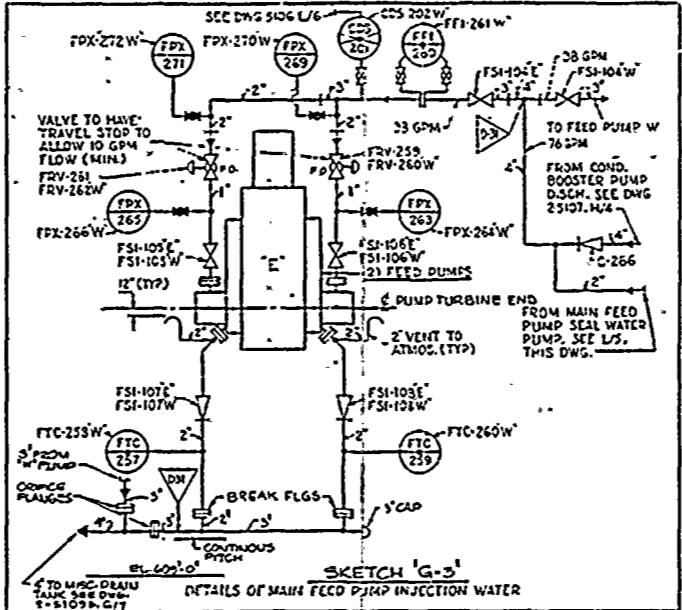
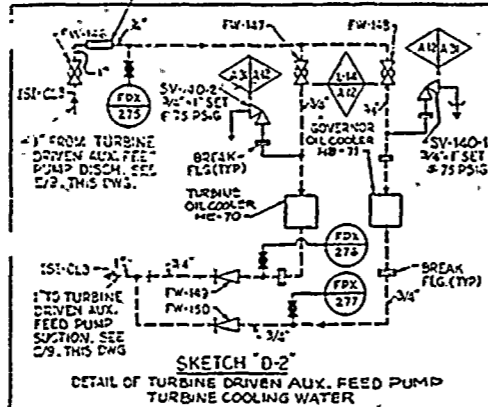
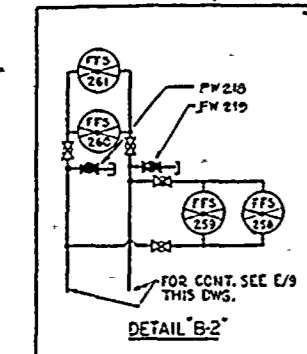
BROOKMAN MICHIGAN

FLOW DIAGRAM AUX. FEEDWATER

UNIT 2
DR. NO. 2-5106A-41

SCALE: 1" = 10'-0"
DATE: 12-5106A
BY: [Signature]
CHECKED BY: [Signature]
APPROVED BY: [Signature]

AMERICAN ELECTRIC POWER SERVICE CORP.

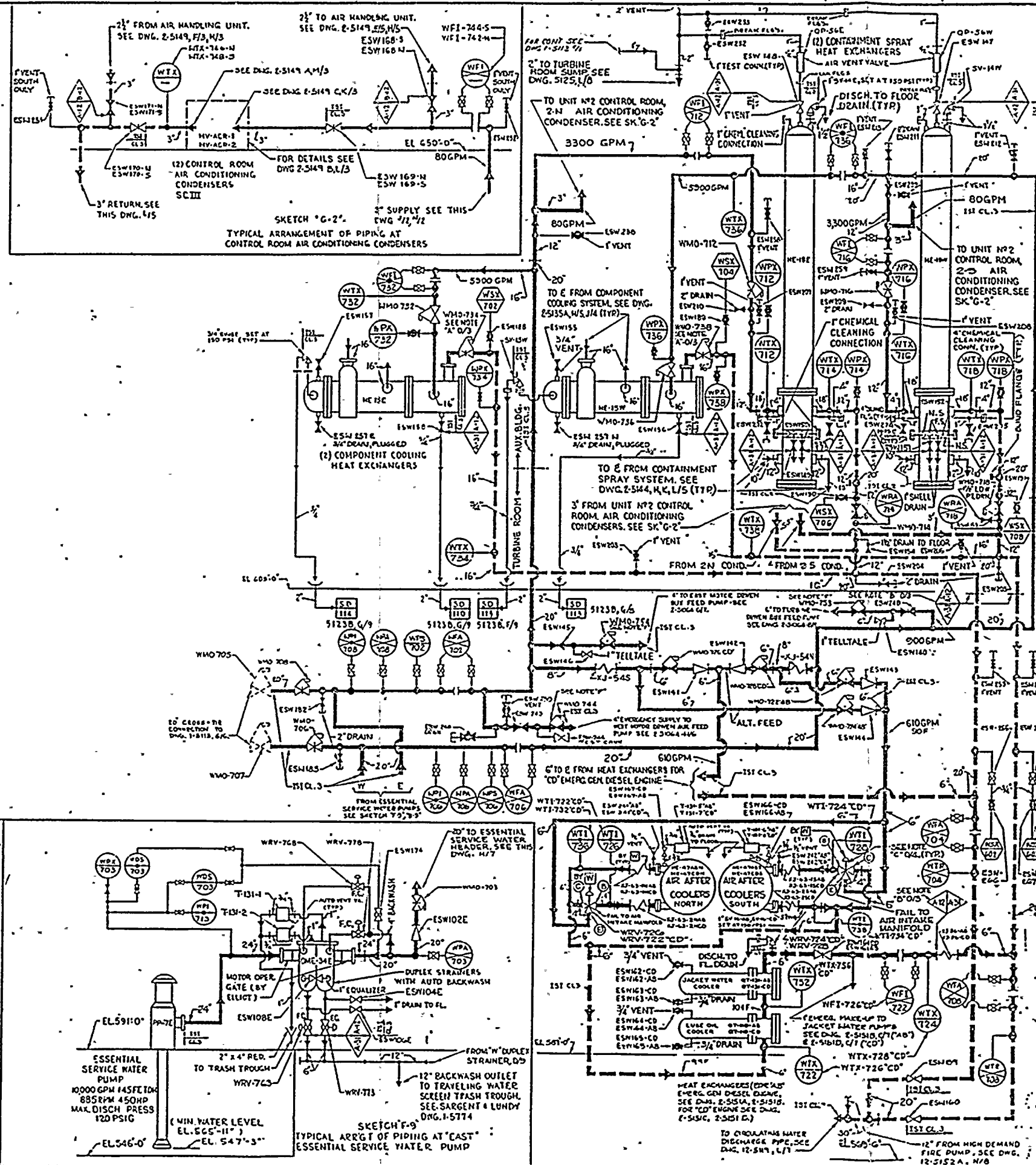


TI
APERTURE
CARD

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8710130278-04

**Also Available On
Aperture Card**



- SUPPLY PIPING
- RETURN PIPING
- AUX. PIPING

ALL PIPING CLASS A-12
UNLESS NOTED
ALL EQUIPMENT SEISMIC CLASS I.
EXCEPT AS NOTED.
FOR VALVE, INSTRUMENT,
SAMPLING, PIPE MATERIAL AND
OTHER SYMBOLS NOT EXPLAINED
ON THIS DWG., AND FOR MARK
NUMBER CODES, SEE DWG. 12-5036
12-5104.

12-3164.
SYMBOL
[W] BY WORTHINGTON

NOTE A: N/A. 1/5
WMO-734, 738 TO
HAVE INTERMEDIATE LIMIT
SWITCH TO LIMIT FLOW ON
SAFETY INJECTION SIGNAL

NOTE B: M/S. .
RETURN PIPING CHANGES
FROM CLASS I (AUX. BLDG)
TO CLASS III (TURB. ROOM)

NOTE 'c': J/A, N/A,
ENCIRCLED LETTERS ARE
SHOWN FOR ORIENTATION OF
VALVE IN PIPING. THESE
LETTERS REFLECT SIMILAR
MARKINGS ON VALVE BODY

MARKINGS ON VALVE BODY
NOTE "D"
THIS WAS MADE UNDER FOR UNIT 2
AND SUPERSEDED 32-311221
NOTE "E"
FOR LOSS CLASS 2 & 3 INTER-
CONNECTIONS, THE 1ST BOUNDARY
EXTENDS TO AND INCLUDES THE
FIRST ROOT VALVE.

NOTE F
FOR COOL CLASS 225 VENTS AND W/IN
THE 151 BOUNDARY EXTENDS TO AND
INCLUDES THE FIRST NORMALLY CLOSED
VALVE.

NOTE 6 -
VALVES ALSO SHOWN (AND
NUMBERED) ON Dwg. 2-S102A.
THE UNIT PREFIX DESIGNATION
FOR EACH COMPONENT IDENTIFICATION
NUMBER IS "2" UNLESS
OTHERWISE SPECIFIED.

HAND OPERATED VALVE
IDENTIFICATION NUMBERS
1. ONLY "UNIQUE VALVE NUMBERS"
APPEAR ON THIS DRAWING. SEE
SEPARATE VALVE IDENTIFICATION
LIST FOR EQUIVALENT DESIGN
(MCR) NUMBERS.

2. TAG NUMBERS MODIFIED FOR DRAWING USE AS FOLLOWS:
TAG N: 2-NSW-VIOS-W
APPEARS AS: NSWVIOSW

3. INSTRUMENT ROOT VALVE MARK N'S NOT SHOWN ON DRAWING (SEE VALVE IDENTIFICATION LIST) DERIVED BY ADDING TO INSTRUMENT NUMBER:
• FOR SINGLE IMPULSE: VI
• FOR DOUBLE IMPULSE: VVUPSTREAM
VZCOWSTH

THE UNIVERSITY OF MICHIGAN
LIBRARY

2-2-87	36	<i>[Signature]</i> RA
DATE	NO	APPROVED

FOR REVISION DESCRIPTION SEE
SEPARATE REVISION RECORD
FOR THIS DRAWING

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INDIANA & MICHIGAN ELECTRIC CO.
DONALD C. COOK
NUCLEAR PLANT

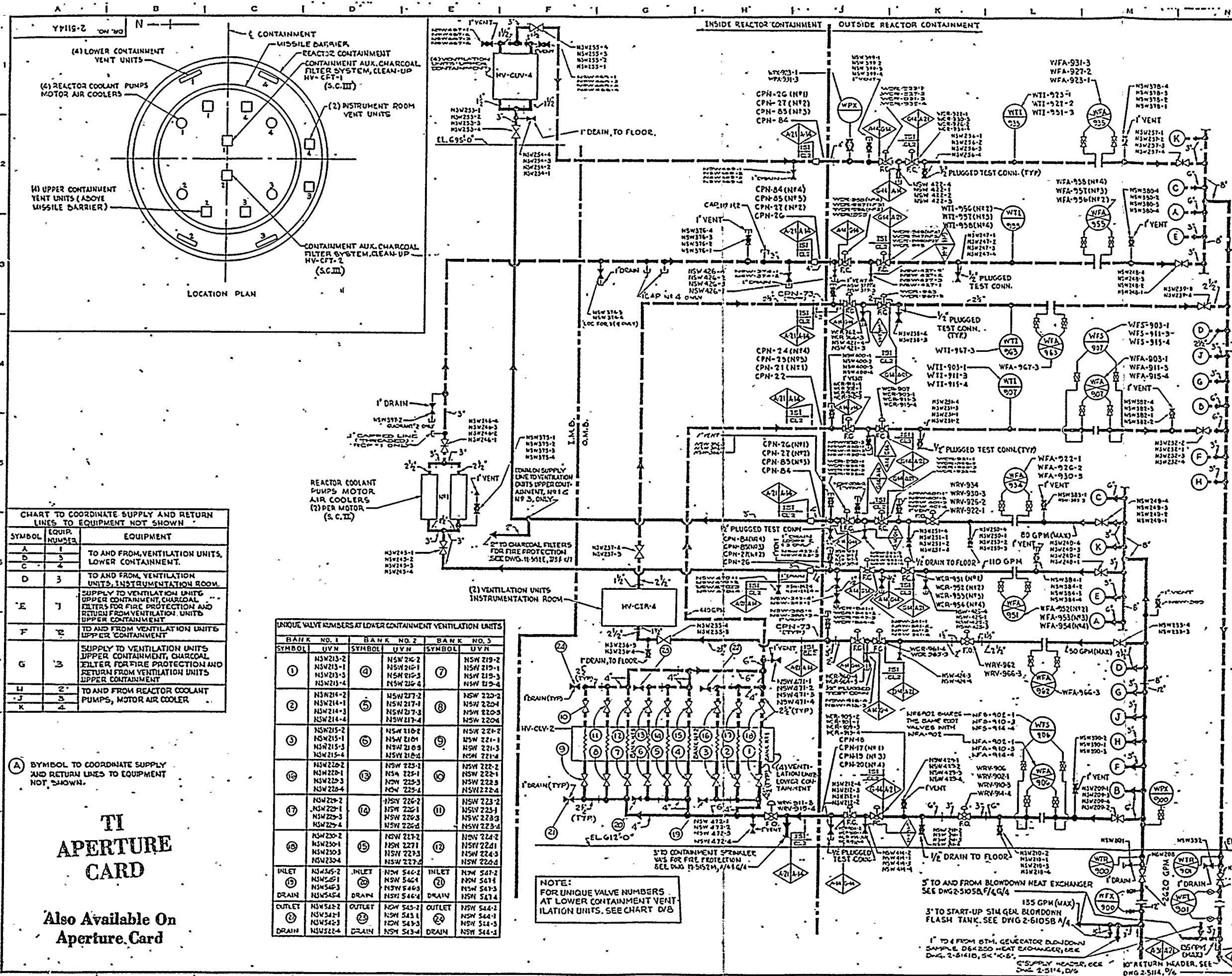
BRIDGMAN - MORGAN
FLOW DIAGRAM
ESSENTIAL
SERVICE WATER
UNITS No 2

OPR. NO. 2-5113-36 *

DATE	10/10/1964
TIME	10:10
LOCATION	10:10
REMARKS	10:10

DATE	12/1/68	TIME	10:00
BY	J. J. [illegible]	INITIALS	[illegible]
REMARKS	[illegible]		

8710130278-05



GENERAL NOTES

LEGEND

SUPPLY PIPING
RETURN PIPING
AUXILIARY PIPING

FOR VALVE, INSTRUMENT, SAMPLING, PIPE MATERIAL AND OTHER SYMBOLS NOT EXPLAINED ON THIS DWG., AND FOR MARK NUMBER CODES, SEE DWG. 2-5114-204.

NOTES:

A. ALL EQUIPMENT SEISMIC CLASS II EXCEPT AS NOTED.

B. FOR CODE CLASS 2 INST. COUPLERS, THE 181 BOUNDARY EXTENDS TO & INCLUDES THE FIRST ROOT VALVE.

C. FOR CODE CLASS 2 VENTS & DRAINS, THE 181 BOUNDARY EXTENDS TO & INCLUDES THE FIRST NORMALLY CLOSED VALVE.

NOTE:

1. USE UNIT PREFIX DESIGNATION FOR EACH COMPONENT IDENTIFICATION NUMBER IS "2" UNLESS OTHERWISE NOTED.

HAND OPERATED VALVE IDENTIFICATION NUMBERS

1. ONLY "UNIQUE VALVE NUMBERS" APPEAR ON THIS DRAWING. SEE SEPARATE VALVE IDENTIFICATION LIST FOR EQUIVALENT DESIGN (MCR) NUMBERS.

2. "TAG" NUMBERS MODIFIED FOR DRAWING USE AS FOLLOWS: TAG NO. 2-5114-204-W APPEARS AS: 2-5114-W

3. INSTRUMENT ROOT VALVE MARK IS NOT SHOWN ON DRAWING (SEE VALVE IDENTIFICATION LIST) DERIVED BY ADDING TO INSTRUMENT NUMBER "1" FOR SINGLE PULSE, "V" FOR DOUBLE PULSE, "V" FOR DOUBLE PULSE, "V" FOR DOUBLE PULSE, "V" FOR DOUBLE PULSE.

FOR REVISION DESCRIPTION SEE SEPARATE REVISION RECORD FOR THIS DRAWING

DATE: 10-3-66
NO. 27
APPROVED: CUL

FOR REVISION DESCRIPTION SEE SEPARATE REVISION RECORD FOR THIS DRAWING

INDIANA & MICHIGAN ELECTRIC CO.
DONALD C. COOK
NUCLEAR PLANT

FLOW DIAGRAM NON-ESSENTIAL SERVICE WATER UNIT NO. 27

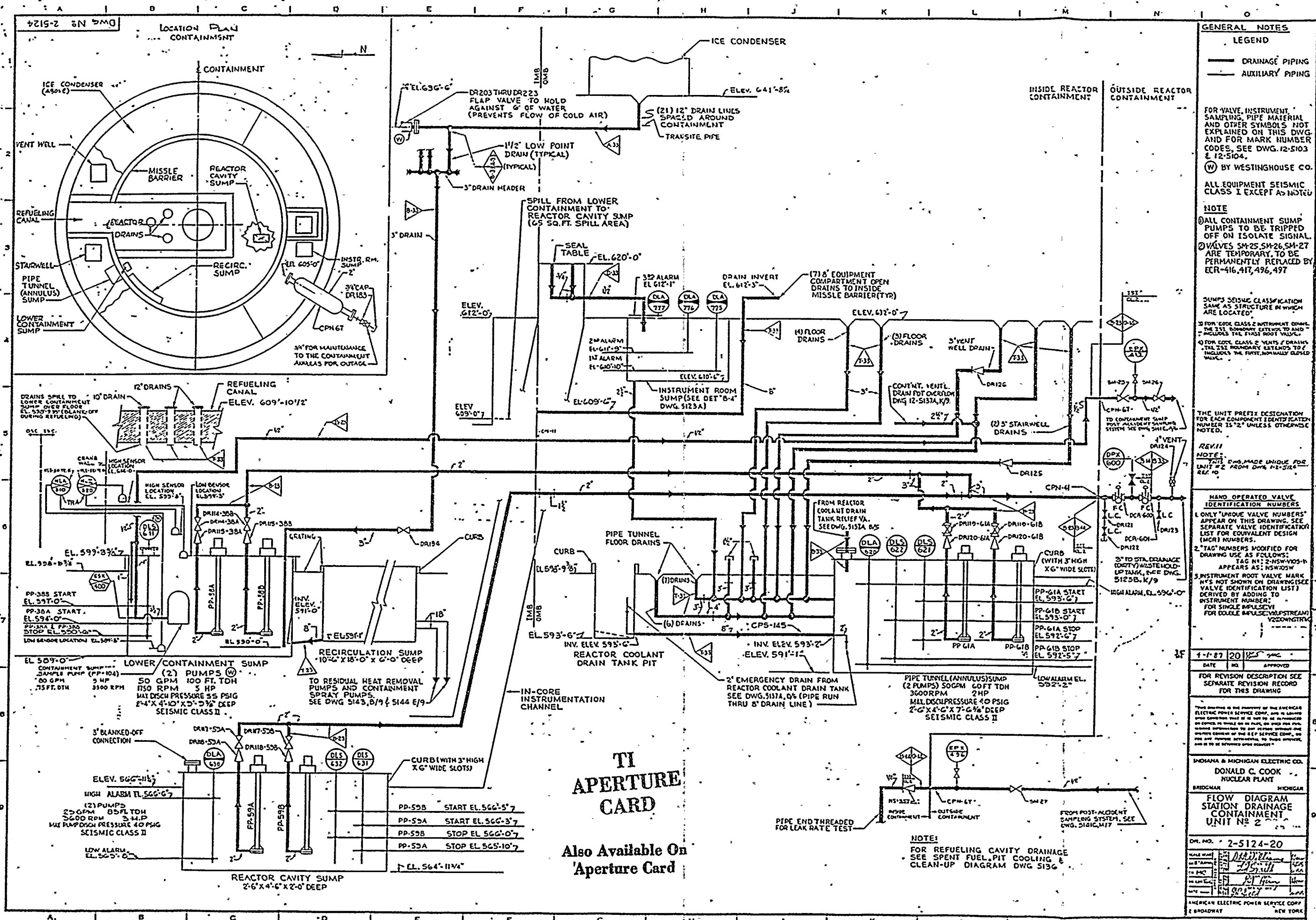
DR. NO. 2-5114 A-27

AMERICAN ELECTRIC POWER SERVICE CORP.
2 BROADWAY, NEW YORK

CHART TO COORDINATE SUPPLY AND RETURN LINES TO EQUIPMENT NOT SHOWN

SYMBOL	EQUIP. NUMBER	EQUIPMENT
A	1	TO AND FROM VENTILATION UNITS, LOWER CONTAINMENT.
B	2	TO AND FROM VENTILATION UNITS, INSTRUMENTATION ROOM.
C	3	SUPPLY TO VENTILATION UNITS UPPER CONTAINMENT, CHARCOAL FILTERS FOR FIRE PROTECTION AND RETURN FROM VENTILATION UNITS UPPER CONTAINMENT.
D	4	TO AND FROM VENTILATION UNITS UPPER CONTAINMENT.
E	5	SUPPLY TO VENTILATION UNITS UPPER CONTAINMENT, CHARCOAL FILTER FOR FIRE PROTECTION AND RETURN FROM VENTILATION UNITS UPPER CONTAINMENT.
F	6	TO AND FROM REACTOR COOLANT PUMPS, MOTOR AIR COOLER.
G	7	TO AND FROM REACTOR COOLANT PUMPS, MOTOR AIR COOLER.
H	8	TO AND FROM REACTOR COOLANT PUMPS, MOTOR AIR COOLER.
I	9	TO AND FROM REACTOR COOLANT PUMPS, MOTOR AIR COOLER.
J	10	TO AND FROM REACTOR COOLANT PUMPS, MOTOR AIR COOLER.
K	11	TO AND FROM REACTOR COOLANT PUMPS, MOTOR AIR COOLER.

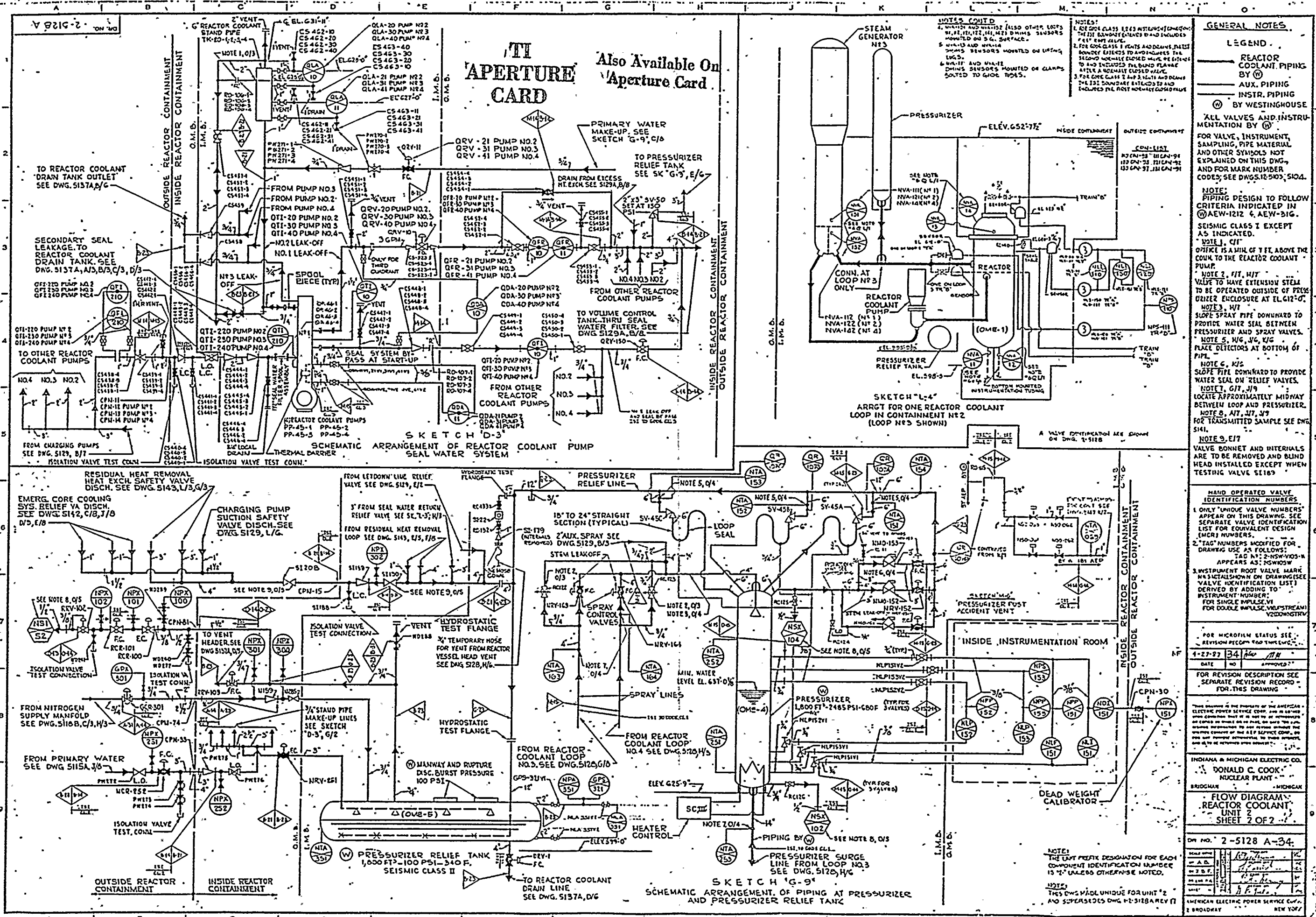
UNIQUE VALVE NUMBERS AT LOWER CONTAINMENT VENTILATION UNITS					
BANK NO. 1		BANK NO. 2		BANK NO. 3	
SYMBOL	UVN	SYMBOL	UVN	SYMBOL	UVN
①	NSW 213-2	④	NSW 216-2	⑦	NSW 219-2
	NSW 213-1		NSW 216-1		NSW 219-1
	NSW 213-3		NSW 216-3		NSW 219-3
	NSW 213-4		NSW 216-4		NSW 219-4
②	NSW 214-2	⑤	NSW 217-2	⑧	NSW 220-2
	NSW 214-1		NSW 217-1		NSW 220-1
	NSW 214-3		NSW 217-3		NSW 220-3
	NSW 214-4		NSW 217-4		NSW 220-4
③	NSW 215-2	⑥	NSW 218-2	⑨	NSW 221-2
	NSW 215-1		NSW 218-1		NSW 221-1
	NSW 215-3		NSW 218-3		NSW 221-3
	NSW 215-4		NSW 218-4		NSW 221-4
⑬	NSW 226-2	⑭	NSW 229-2	⑯	NSW 232-2
	NSW 226-1		NSW 229-1		NSW 232-1
	NSW 226-3		NSW 229-3		NSW 232-3
	NSW 226-4		NSW 229-4		NSW 232-4
⑰	NSW 229-2	⑱	NSW 232-2	㉑	NSW 235-2
	NSW 229-1		NSW 232-1		NSW 235-1
	NSW 229-3		NSW 232-3		NSW 235-3
	NSW 229-4		NSW 232-4		NSW 235-4
㉓	NSW 230-2	㉔	NSW 233-2	㉖	NSW 236-2
	NSW 230-1		NSW 233-1		NSW 236-1
	NSW 230-3		NSW 233-3		NSW 236-3
	NSW 230-4		NSW 233-4		NSW 236-4
㉙	NSW 346-2	㉚	NSW 349-2	㉜	NSW 352-2
	NSW 346-1		NSW 349-1		NSW 352-1
	NSW 346-3		NSW 349-3		NSW 352-3
	NSW 346-4		NSW 349-4		NSW 352-4
㉞	NSW 348-2	㉟	NSW 351-2	㊱	NSW 354-2
	NSW 348-1		NSW 351-1		NSW 354-1
	NSW 348-3		NSW 351-3		NSW 354-3
	NSW 348-4		NSW 351-4		NSW 354-4
㊲	NSW 349-2	㊳	NSW 352-2	㊵	NSW 355-2
	NSW 349-1		NSW 352-1		NSW 355-1
	NSW 349-3		NSW 352-3		NSW 355-3
	NSW 349-4		NSW 352-4		NSW 355-4



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GENERAL NOTES

LEGEND

REACTOR COOLANT PIPING BY (R)
AUX. PIPING BY (A)
INSTR. PIPING BY (I)
BY WESTINGHOUSE

ALL VALVES AND INSTRUMENTATION BY (W)

FOR VALVE, INSTRUMENT, SAMPLING, PIPE MATERIAL AND OTHER SYMBOLS NOT EXPLAINED ON THIS DWG., AND FOR MARK NUMBER CODES, SEE DWG. 12-5103, 5104.

NOTE:
PIPING DESIGN TO FOLLOW CRITERIA INDICATED IN (W) AEW-1212 & AEW-316.

SEISMIC CLASS I EXCEPT AS INDICATED.

NOTE 1, C11:
ORIFICE IS A MIN. OF 1 FT. ABOVE THE COU. TO THE REACTOR COOLANT PUMP.

NOTE 2, H17, H17:
VALVE TO HAVE EXTENSION STEM TO BE OPERATED OUTSIDE OF PRESSURIZER ENCLOSURE AT EL. 612'-0".

NOTE 3, H17:
SLOPE SPRAY PIPE DOWNWARD TO PROVIDE WATER SEAL BETWEEN PRESSURIZER AND SPRAY VALVES.

NOTE 5, W16, W16:
PLACE DETECTORS AT BOTTOM OF PIPE.

NOTE 6, K12:
SLOPE PIPE DOWNWARD TO PROVIDE WATER SEAL ON RELIEF VALVES.

NOTE 7, G17, J19:
LOCATE APPROXIMATELY MIDWAY BETWEEN LOOP AND PRESSURIZER.

NOTE 8, A17, J17, J17:
FOR TRANSMITTED SAMPLE SEE DWG. 5141.

NOTE 9, E17:
VALVE BONNET AND INTERNALS ARE TO BE REMOVED AND BLIND HEAD INSTALLED EXCEPT WHEN TESTING VALVE 51189.

HAND OPERATED VALVE IDENTIFICATION NUMBERS

1. ONLY "UNIQUE VALVE NUMBERS" APPEAR ON THIS DRAWING. SEE SEPARATE VALVE IDENTIFICATION LIST FOR EQUIVALENT DESIGN (HICR) NUMBERS.

2. TAG NUMBERS MODIFIED FOR DRAWING USE AS FOLLOWS:
TAG NO. 2-NSW-V100-11 APPEARS AS: 2-NSW-V100-11

3. INSTRUMENT ROOT VALVE MARK NO. 31211 SHOWN ON DRAWING (SEE VALVE IDENTIFICATION LIST) DERIVED BY ADDING TO INSTRUMENT NUMBER:
FOR SINGLE IMPLUSE, VI
FOR DOUBLE IMPLUSE, VI/STREAM

FOR MICROFILM STATUS SEE:
REVISION RECORD FOR THIS DWG.

4-27-97 34 *AW* *MR*

DATE NO. APPROVED

FOR REVISION DESCRIPTION SEE SEPARATE REVISION RECORD FOR THIS DRAWING

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INDIANA & MICHIGAN ELECTRIC CO.
DONALD C. COOK
NUCLEAR PLANT
BIRMINGHAM
MEMPHIS

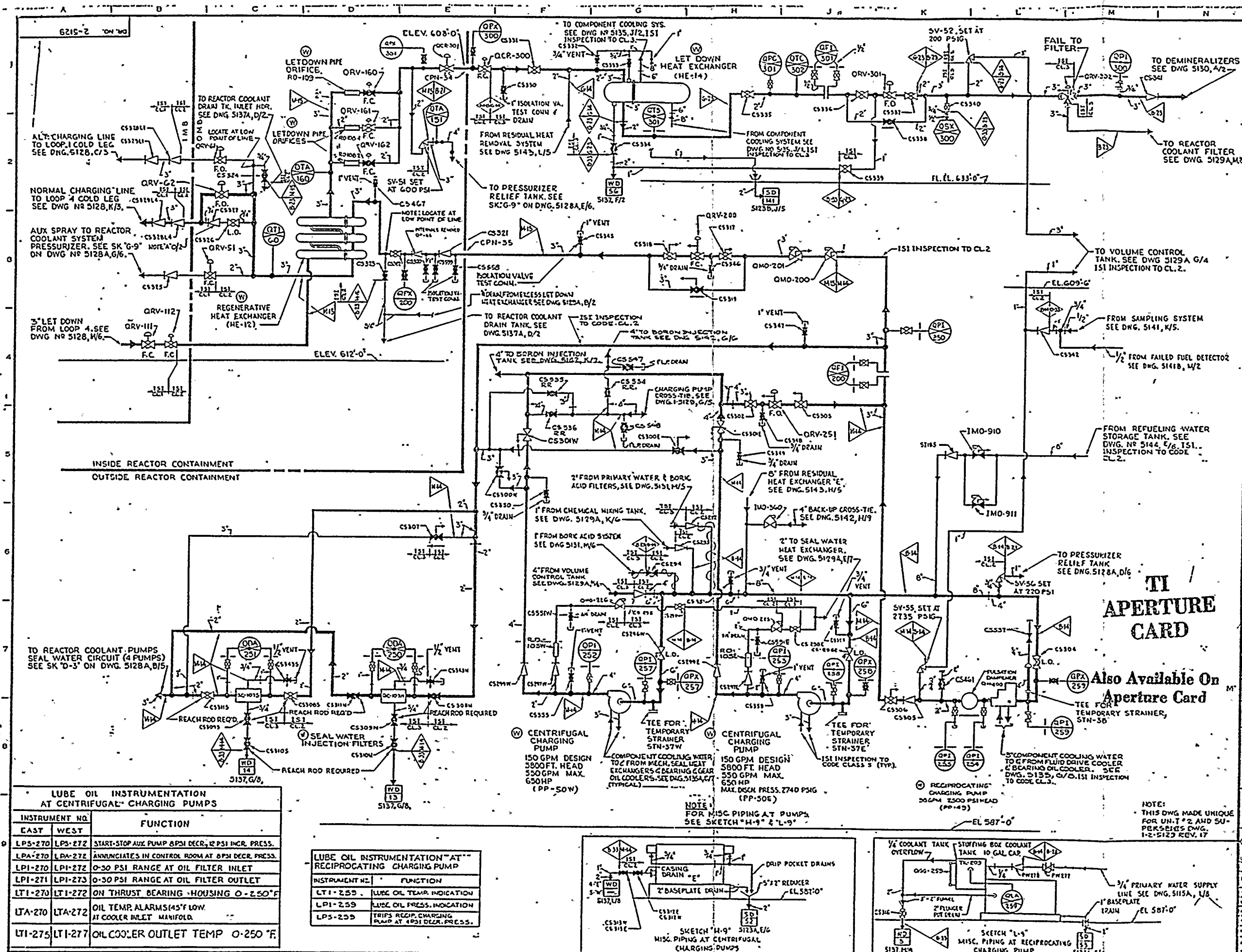
FLOW DIAGRAM:
REACTOR COOLANT UNIT 2
SHEET 2 OF 2

DW. NO. 2-5128 A-34

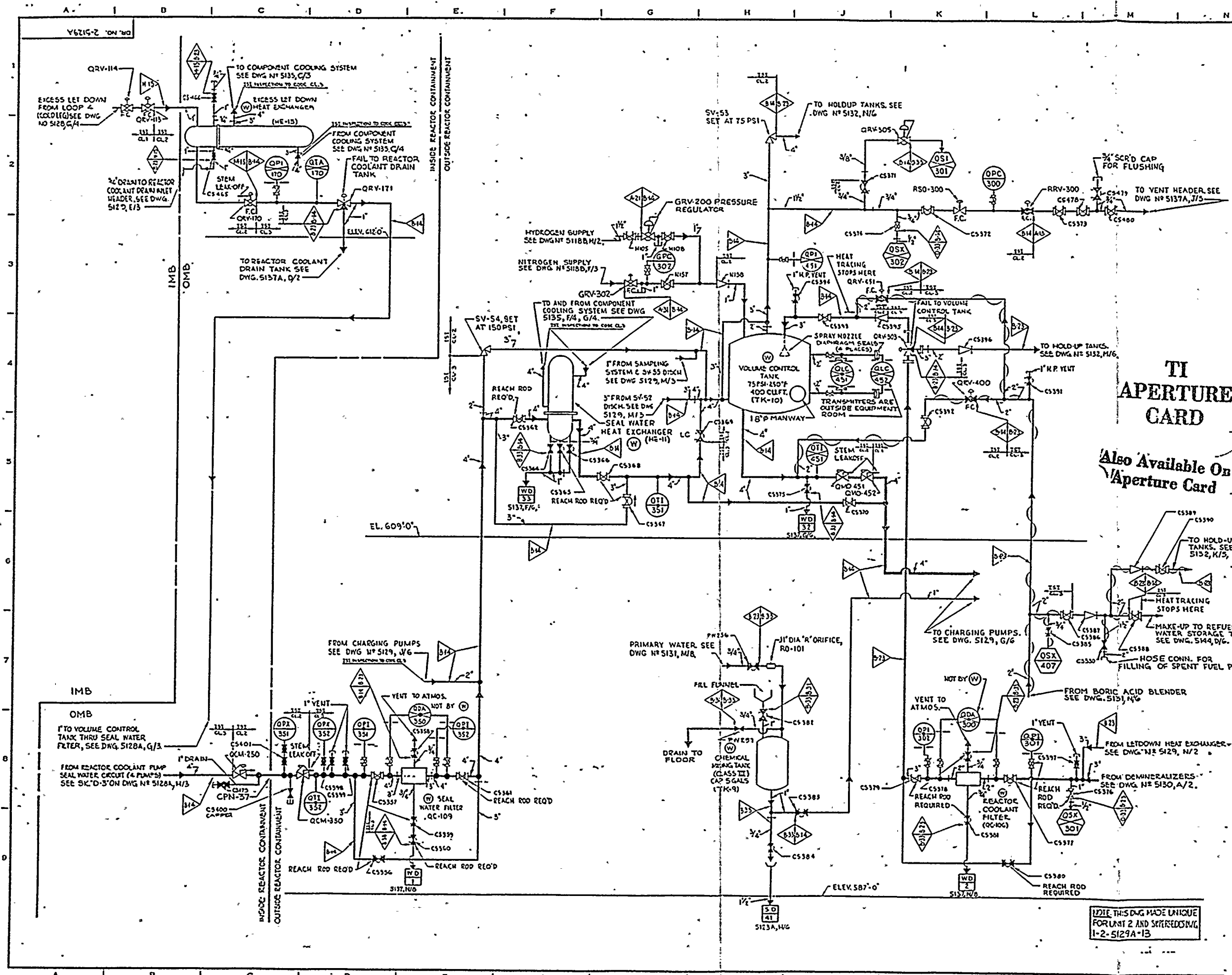
NOTES:
THE UNIT PREFIX DESIGNATION FOR EACH COMPONENT IDENTIFICATION NUMBER IS "2" UNLESS OTHERWISE NOTED.

NOTE:
THIS DWG. HAS UNIQUE FOR UNIT 2 AND SUPERSEDES DWG. 12-5128 REV. 1

AMERICAN ELECTRIC POWER SERVICE CORP.
2 BROADWAY
NEW YORK



GENERAL NOTES	
LEGEND <div style="display: flex; justify-content: space-around; margin-top: 5px;"> <div style="border-bottom: 2px solid black; width: 50px; display: inline-block;"></div> MAIN FLOW <div style="border-bottom: 2px dashed black; width: 50px; display: inline-block;"></div> AUX. FLOW </div>	
FOR VALVE, INSTRUMENT, SAMPLING, PIPE MATERIAL AND OTHER SYMBOLS NOT EXPLAINED ON THIS DWG AND FOR MARK NUMBER CODES SEE DWG 12-5103412-5004	
SEISMIC CLASS. VALVE NOTED "A" B/S VALVE OPENS AT 500 PSID	
(W) BY WESTINGHOUSE EXCEPT AS NOTED ALL VALVES & INSTRUMENTATION SUPPLIED BY (W)	
EQUIPMENT SUPPLIED BY (W) AS NOTED	
1. FOR CODE CLASS 2 & 3 INSTRUMENT CONNECTIONS, THE ISI BOUNDARY EXTENDS TO AND INCLUDES THE FIRST ROOT VALVE.	
2. FOR CODE CLASS 2 & 3 VENTS & DRAINS, THE ISI BOUNDARY EXTENDS TO & INCLUDES THE FIRST NORMALLY CLOSED VALVE.	
3. R.R. INDICATES REACH ROD REQUIRED	
THE UNIT PREFIX DESIGNATION FOR EACH COMPONENT IDENTIFICATION NUMBER IS "2" UNLESS OTHERWISE NOTED.	
<u>HAND OPERATED VALVE IDENTIFICATION NUMBERS</u> 1. ONLY "UNIQUE VALVE NUMBERS" APPEAR ON THIS DRAWING. SEE SEPARATE VALVE IDENTIFICATION LIST FOR EQUIVALENT DESIGN (MCR) NUMBERS.	
2. "TAG" NUMBERS MODIFIED FOR DRAWING USE AS FOLLOWS: <div style="margin-left: 20px;">TAG N°: 2-NSW-VIOS-W</div> <div style="margin-left: 20px;">APPEARS AS: NSWIOOW </div>	
3. INSTRUMENT ROOT VALVE MARK N°'S NOT SHOWN ON DRAWING (SEE VALVE IDENTIFICATION LIST), DERIVED BY ADDING TO INSTRUMENT NUMBER: <div style="margin-left: 20px;">FOR SINGLE PULSE USE: V2204MSTRM</div> <div style="margin-left: 20px;">FOR DOUBLE PULSE: V2204MSTRM </div>	
4. FOR MICROFILM STATUS SEE REVISION RECORD FOR THIS DWG.	
<div style="display: flex; justify-content: space-between;"> <div>3-11-77</div> <div>32</div> <div>REV 1</div> </div> <div style="display: flex; justify-content: space-between; margin-top: 5px;"> <div>DATE</div> <div>NO</div> <div>APPROVED</div> </div>	
FOR REVISION DESCRIPTION SEE SEPARATE REVISION RECORD FOR THIS DRAWING	
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INDIANA & MICHIGAN ELECTRIC CO. DONALD C. COOK NUCLEAR PLANT	
BROCKMAN	MICHIGAN
FLOW DIAGRAM CVCS - REACTOR LEADOWN & CHARGING	
SHEET NO. 2	
SHEET 1 OF 2	
ORL NO. 2-5129-32	
<div style="display: flex; justify-content: space-between;"> <div> AMERICAN ELECTRIC POWER SERVICE CORP. BROADWAY NEW YORK </div> <div> </div> </div>	



GENERAL NOTES

LEGEND

— MAIN FLOW
 - - - AUX. FLOW

FOR VALVE, INSTRUMENT, SAMPLING, PIPE MATERIAL AND OTHER SYMBOLS NOT EXPLAINED ON THIS DWG AND FOR MARK NUMBER CODES SEE DWGS 12 5103 & 12 5104

SEISMIC CLASS I EXCEPT AS NOTED

BY WESTINGHOUSE

ALL VALVES & INSTRUMENTATION SUPPLIED BY (W)

EQUIPMENT SUPPLIED BY (W) AS NOTED

NOTES:

1. FOR COOL. CL. 2/3 INSTRUMENT CODES THE 1ST BOUNDARY EXTENDS TO AND INCLUDES THE FIRST ROOT VALVE.

2. FOR COOL. CL. 2/3 VENTS/DRAINS THE 1ST BOUNDARY EXTENDS TO AND INCLUDES THE FIRST NORMALLY CLOSED VALVE.

THE UNIT PREFIX DESIGNATION FOR EACH COMPONENT IDENTIFICATION NUMBER IS 2-5129A, UNLESS OTHERWISE NOTED.

HAND OPERATED VALVE IDENTIFICATION NUMBERS

1. ONLY "UNIQUE VALVE NUMBERS" APPEAR ON THIS DRAWING. SEE SEPARATE VALVE IDENTIFICATION LIST FOR EQUIVALENT DESIGN (HCR) NUMBERS.

2. TAG NUMBERS MODIFIED FOR DRAWING USE AS FOLLOWS:
 TAG NO. 2-NSW-VIOS-W APPEARS AS: NSWVIOSW

3. INSTRUMENT ROOT VALVE MARK HTS NOT SHOWN ON DRAWING (SEE VALVE IDENTIFICATION LIST) DERIVED BY ADDING TO INSTRUMENT NUMBER:
 FOR SINGLE IMPULSE: V
 FOR DOUBLE IMPULSE: VIMP
 FOR DOUBLE IMPULSE/STREAM: V2DOUSTRM

FOR MICROFILM STATUS SEE REVISION RECORD FOR THIS DWG.

DATE	NO.	APPROVED
4-10-87	2013	RZ

FOR REVISION DESCRIPTION SEE SEPARATE REVISION RECORD FOR THIS DRAWING

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INDIANA & MICHIGAN ELECTRIC CO.
 DONALD C. COOK
 NUCLEAR PLANT

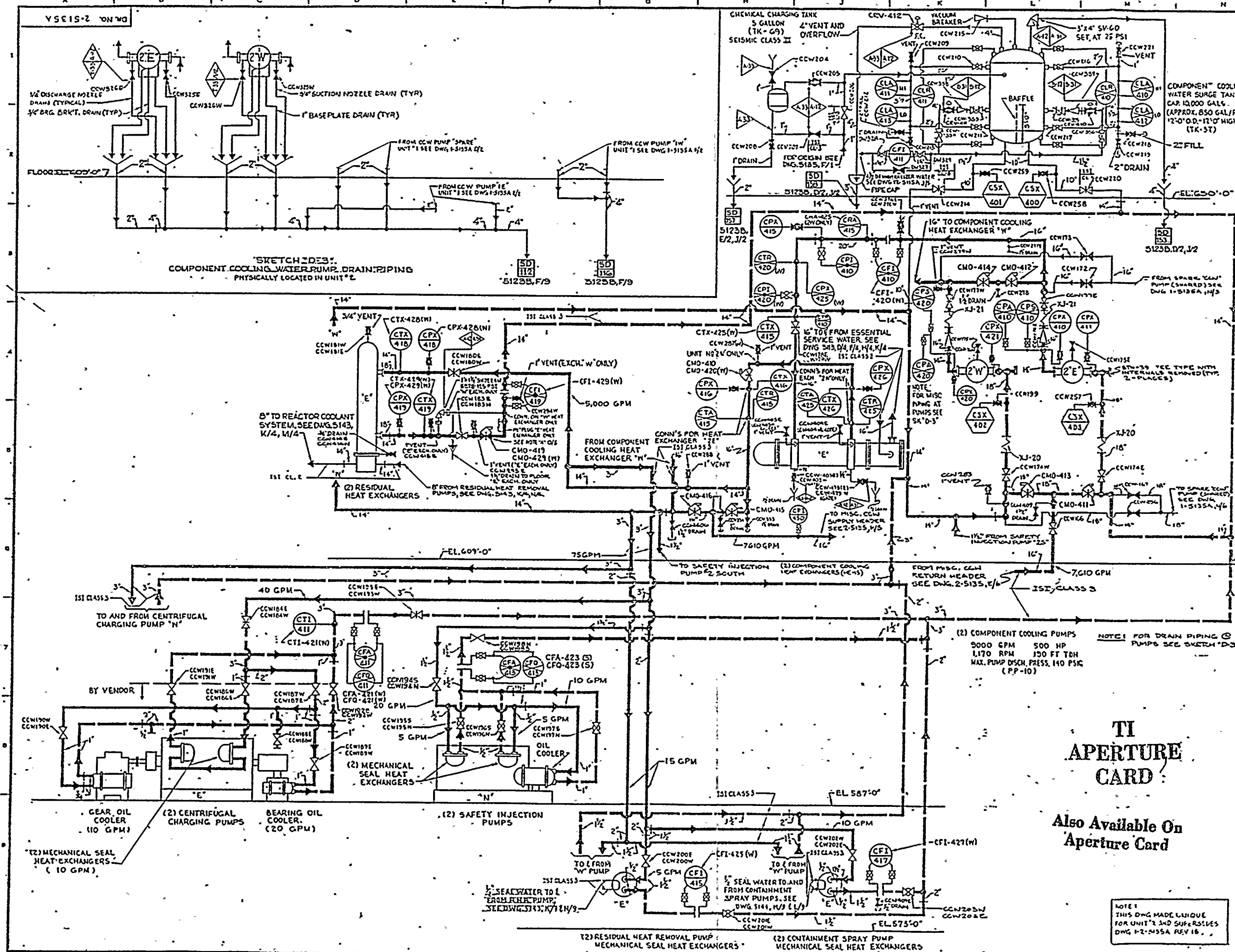
BROCKMAN MORGAN

**FLOW DIAGRAM
 CVCS-REACTOR LETDOWN
 & CHARGING**

UNIT NO. 2
 SHEET 2 OF 2

DWG. NO. 2-5129A-20

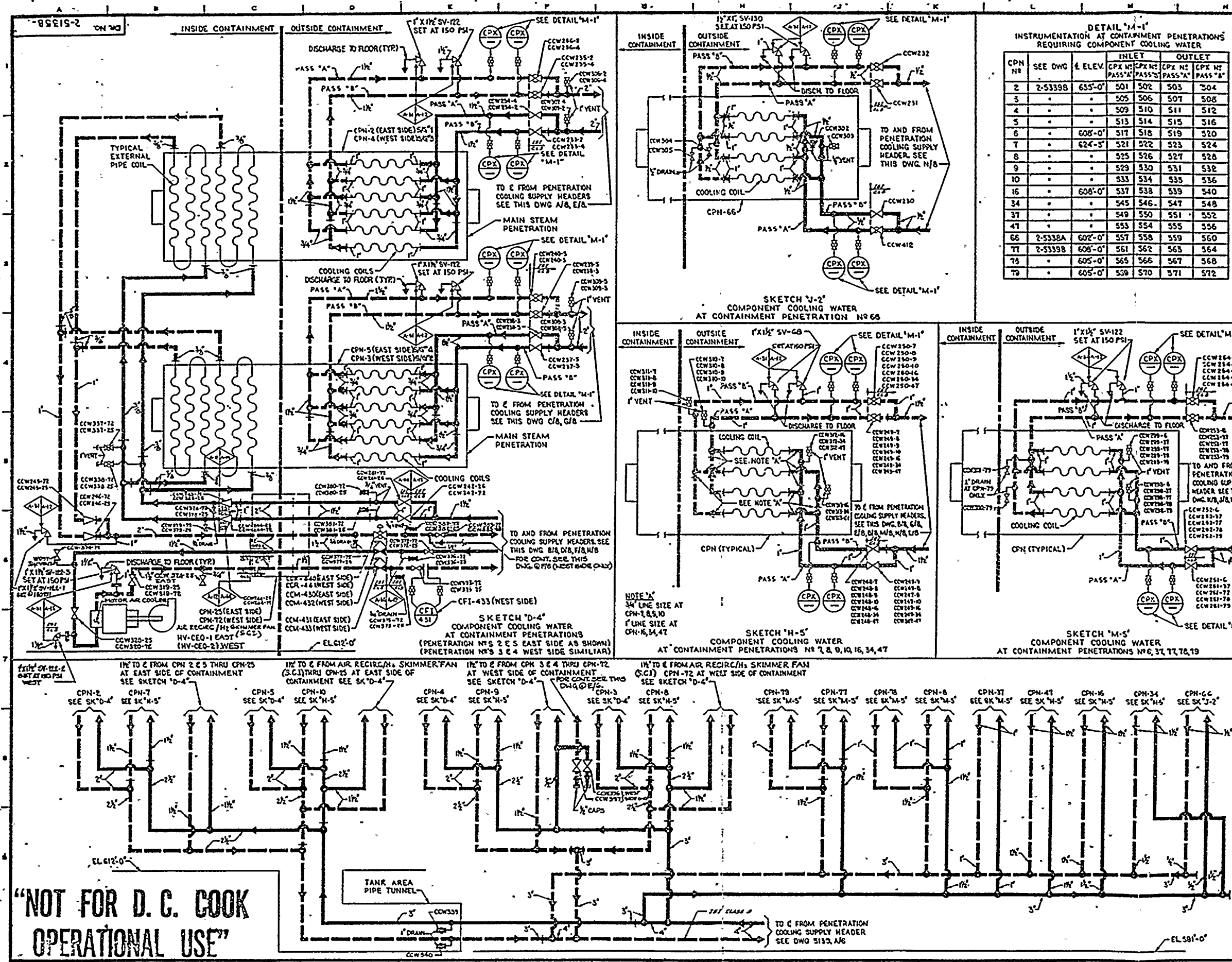
NOTE: THIS DWG MADE UNIQUE FOR UNIT 2 AND SUPERSEDES DWG. 1-2-5129A-13



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87-10130278-13



DETAIL "M-1"
INSTRUMENTATION AT CONTAINMENT PENETRATIONS
REQUIRING COMPONENT COOLING WATER

CPN NO.	SEE DWG.	E. ELEV.	INLET	OUTLET
			CPX NO. PASS "A"	CPX NO. PASS "B"
2	2-5339B	635'-0"	501	502
3	"	"	505	506
4	"	"	509	510
5	"	"	513	514
6	"	608'-0"	517	518
7	"	624'-5"	521	522
8	"	"	525	526
9	"	"	529	530
10	"	"	533	534
16	"	608'-0"	537	538
34	"	"	541	542
37	"	"	545	546
47	"	"	549	550
66	2-5338A	602'-0"	553	554
77	2-5339B	608'-0"	557	558
78	"	605'-0"	561	562
79	"	605'-0"	565	566

GENERAL NOTES

LEGEND

- COMPONENT COOLING SUPPLY
- COMPONENT COOLING RETURN
- AUXILIARY PIPING

FOR VALVE, INSTRUMENT SAMPLING PIPE MATERIAL AND OTHER SYMBOLS NOT EXPLAINED ON THIS DWG., AND FOR MARK NUMBER CODES, SEE DWG. 5104

PENETRATION COOLING COILS ARE SEISMIC CLASS I

NOTE:
1. COOLING COILS ARE 1/2" NPT
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99. COOLING COILS ARE 1/2" NPT
100. COOLING COILS ARE 1/2" NPT

THE UNIT PREFIX DESIGNATION FOR EACH COMPONENT I.D. NO. IS "2" UNLESS OTHERWISE NOTED.

NOTE
THIS DWG. MADE UNIQUE FOR UNIT "2" AND SUPERSEDES DWG. 2-5135 B REV. 7

HAND OPERATED VALVE IDENTIFICATION NUMBERS

- ONLY "UNIQUE VALVE NUMBERS" APPEAR ON THIS DRAWING. SEE SEPARATE VALVE IDENTIFICATION LIST FOR EQUIVALENT DESIGN (WCR) NUMBERS.
- "TAG" NUMBERS MODIFIED FOR DRAWING USE AS FOLLOWS:
TAG NO. 2-5135-B-1000-W
APPEARS AS: 100000W
- INSTRUMENT ROOT VALVE MARK IS NOT SHOWN ON DRAWING (SEE VALVE IDENTIFICATION LIST) DERIVED BY ADDING TO INSTRUMENT NUMBER:
FOR SINGLE BRANCH
FOR DOUBLE BRANCH

FOR APPROVAL STATUS SEE REVISION RECORD ON "2" DWG.

DATE: 11/14/77 BY: [Signature]

FOR REVISION DESCRIPTION SEE SEPARATE REVISION RECORD FOR THIS DRAWING

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INDIANA & MICHIGAN ELECTRIC CO.
DONALD C. COOK
NUCLEAR PLANT

FLOW DIAGRAM
COMPONENT COOLING
UNIT NO. 2
SHEET 3 OF 3

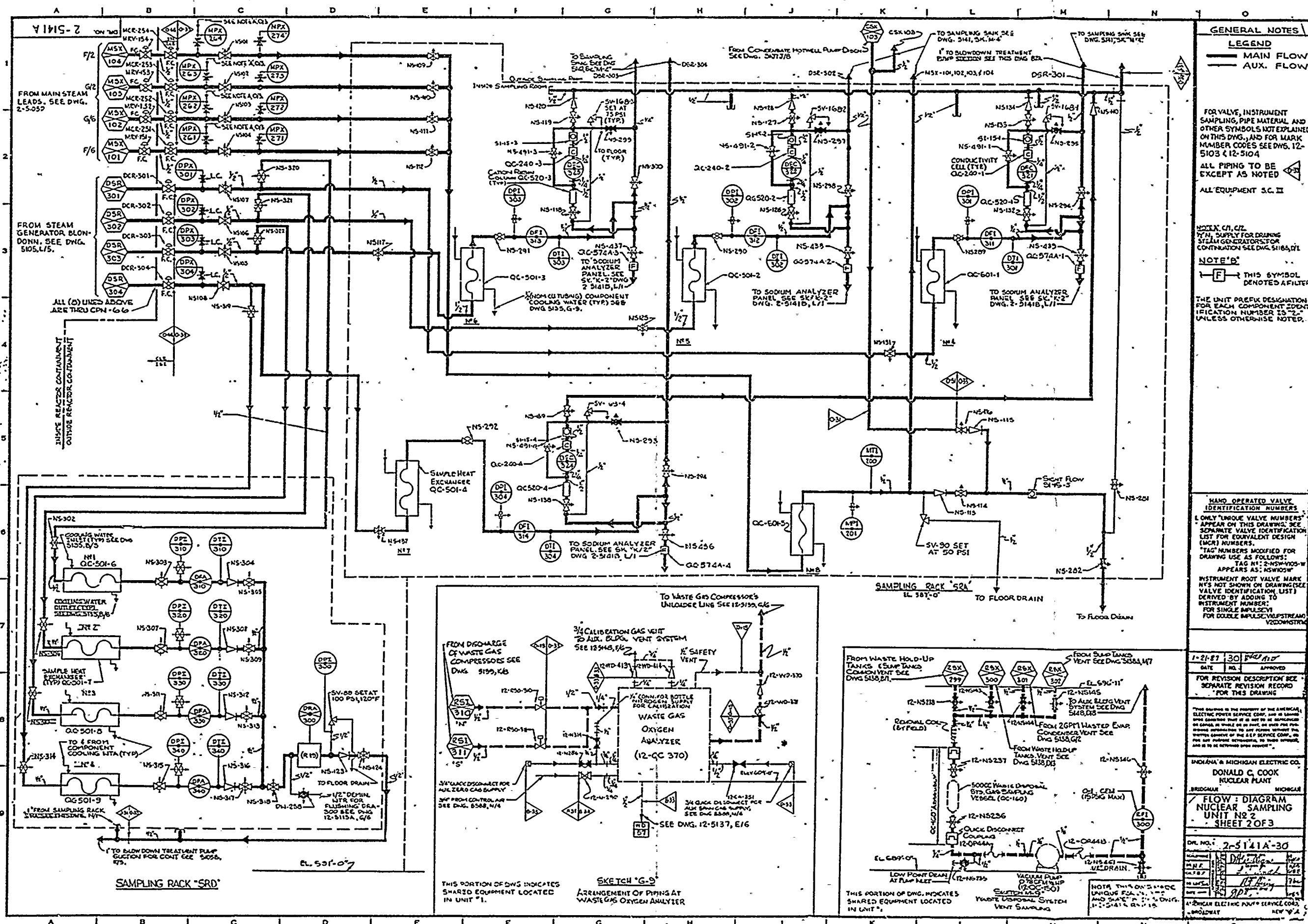
DWG. NO. 2-5135B-14

AMERICAN ELECTRIC POWER SERVICE CORP.
1 BROADWAY
NEW YORK

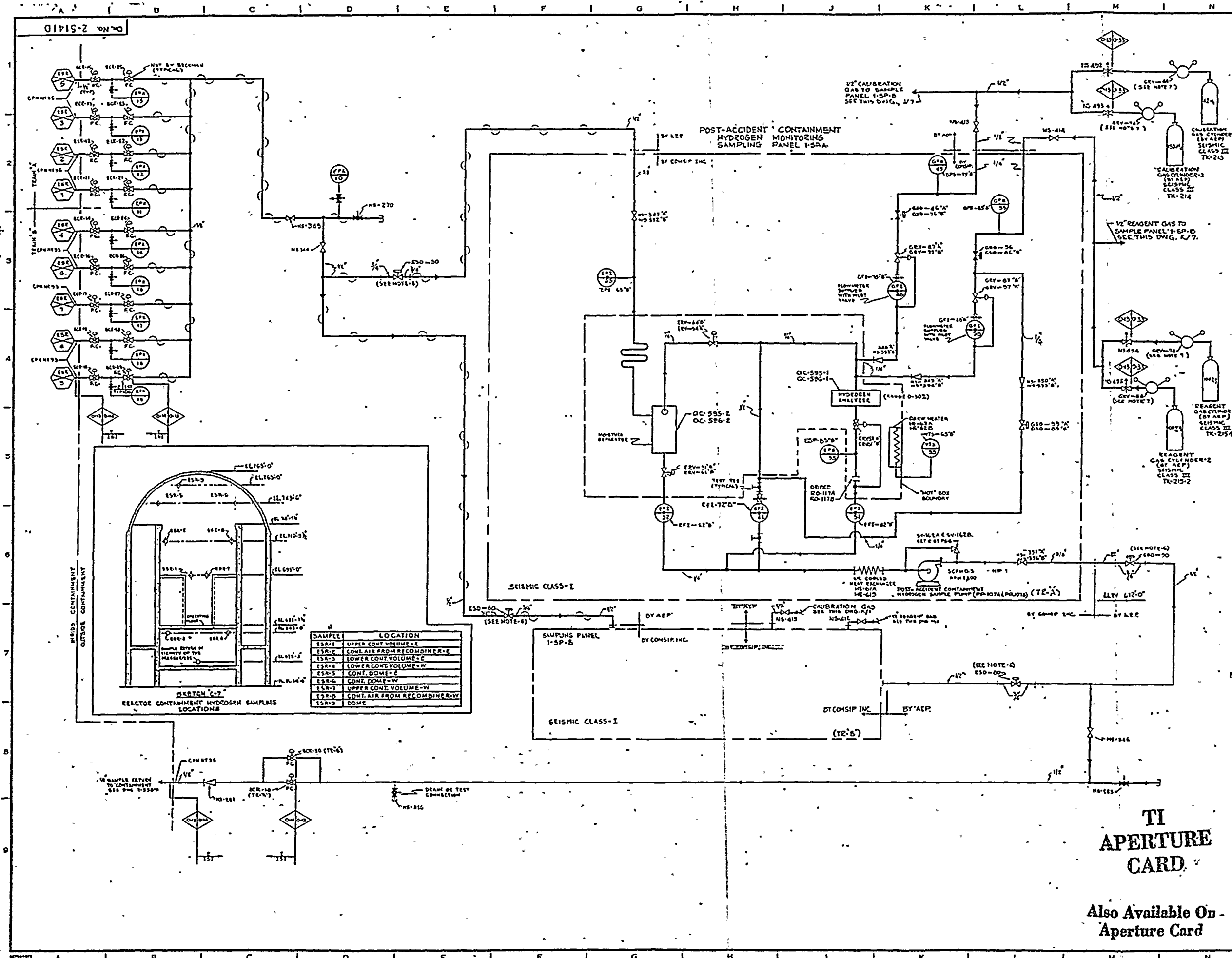
TE
APERTURE
CARD

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8710130278-16



GENERAL NOTES

LEGEND

MAIN FLOW
ASSEMBLY

NOTES

1. LOCATION OF NEW POST-ACCIDENT CONTAINMENT HYDROGEN MONITORING SYSTEMS AS PER INCH-050 AND INCH-051, 1965.

2. ALL EQUIPMENT IS SEISMIC CLASS II, EXCEPT AS NOTED.

3. CONTROL SWITCH LOCATED ON REACTOR CONTAINMENT CONTROL PANEL REGULATING VALVE WITH UP AND DOWN STEADY-PRESSURE GAUGES AND SAFETY RELIEF.

4. ALL PIPING IS 0.15 EXCEPT AS NOTED.

5. THE UNIT PREFIX DESIGNATION FOR EACH COMPONENT IDENTIFICATION NUMBER IS "2" UNLESS OTHERWISE NOTED.

HAND OPERATED VALVE IDENTIFICATION NUMBERS

1. ONLY "UNIQUE VALVE NUMBERS" APPEAR ON THIS DRAWING. SEE SEPARATE VALVE IDENTIFICATION LIST FOR EQUIVALENT DESIGN (UNC) NUMBERS.

2. TAG NUMBERS MODIFIED FOR DRAWING USE AS FOLLOWS:
TAG NO. 2-NSW-100-1
APPEARS AS: NSW100-1

3. INSTRUMENT ROOT VALVE MARK THIS NOT SHOWN ON DRAWING SEE VALVE IDENTIFICATION LIST. DERIVED BY ADDING TO INSTRUMENT NUMBER:
FOR SINGLE PULSE VALVE
FOR DOUBLE PULSE VALVE
FOR DOUBLE PULSE VALVE

DATE: 11-17-66
NO. 2-5141D-8
APPROVED: [Signature]

FOR REVISION DESCRIPTION SEE SEPARATE REVISION RECORD FOR THIS DRAWING

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INCHMAN & MICHIGAN ELECTRIC CO.
DONALD C. COOK
NUCLEAR PLANT

BRIDGMAN MICHIGAN

FLOW DIAGRAM
POST-ACCIDENT SAMPLING
CONTAINMENT HYDROGEN
UNIT #2

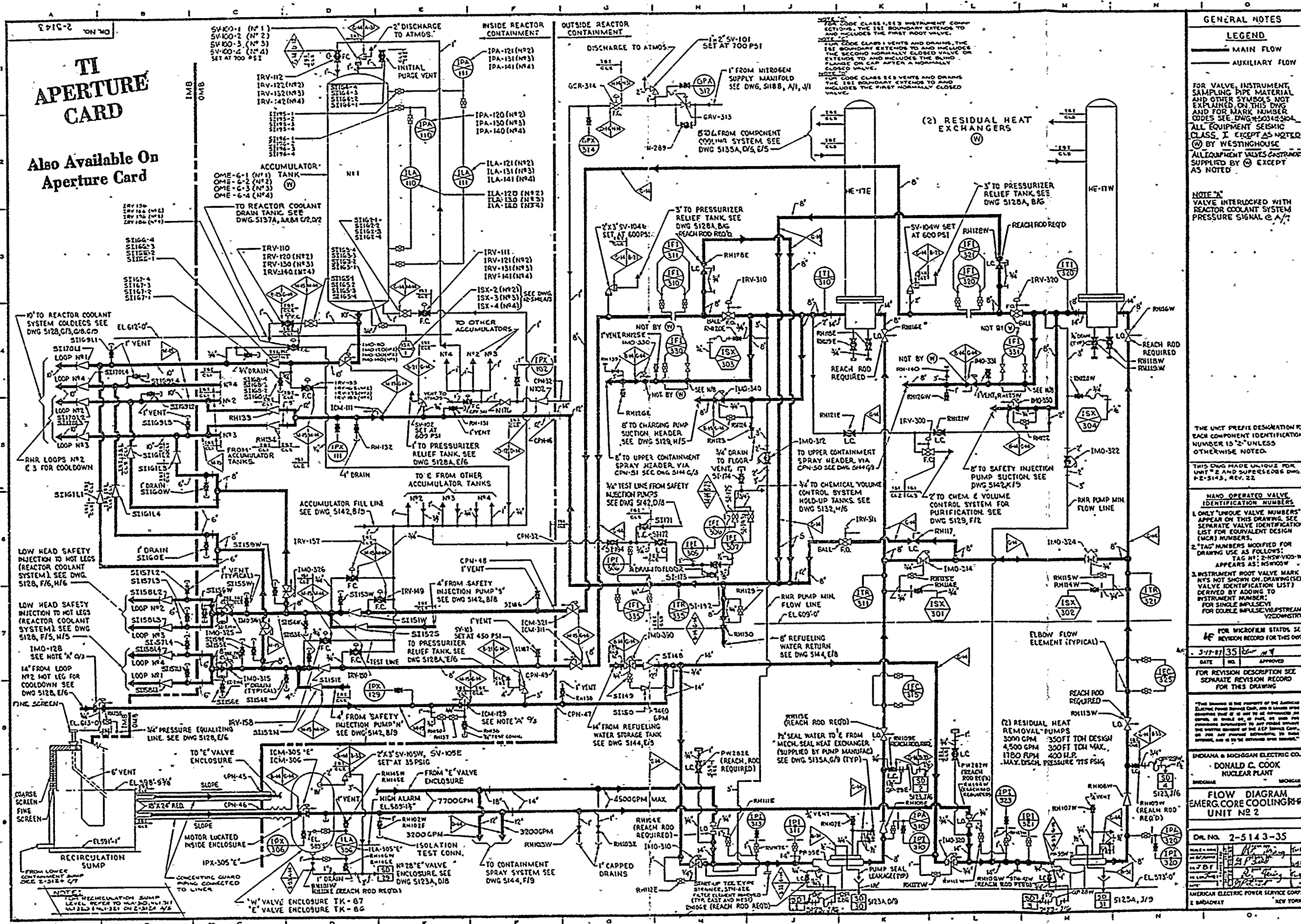
DR. NO. 2-5141D-8

AMERICAN ELECTRIC POWER SERVICE CORP.

**TI
APERTURE
CARD**

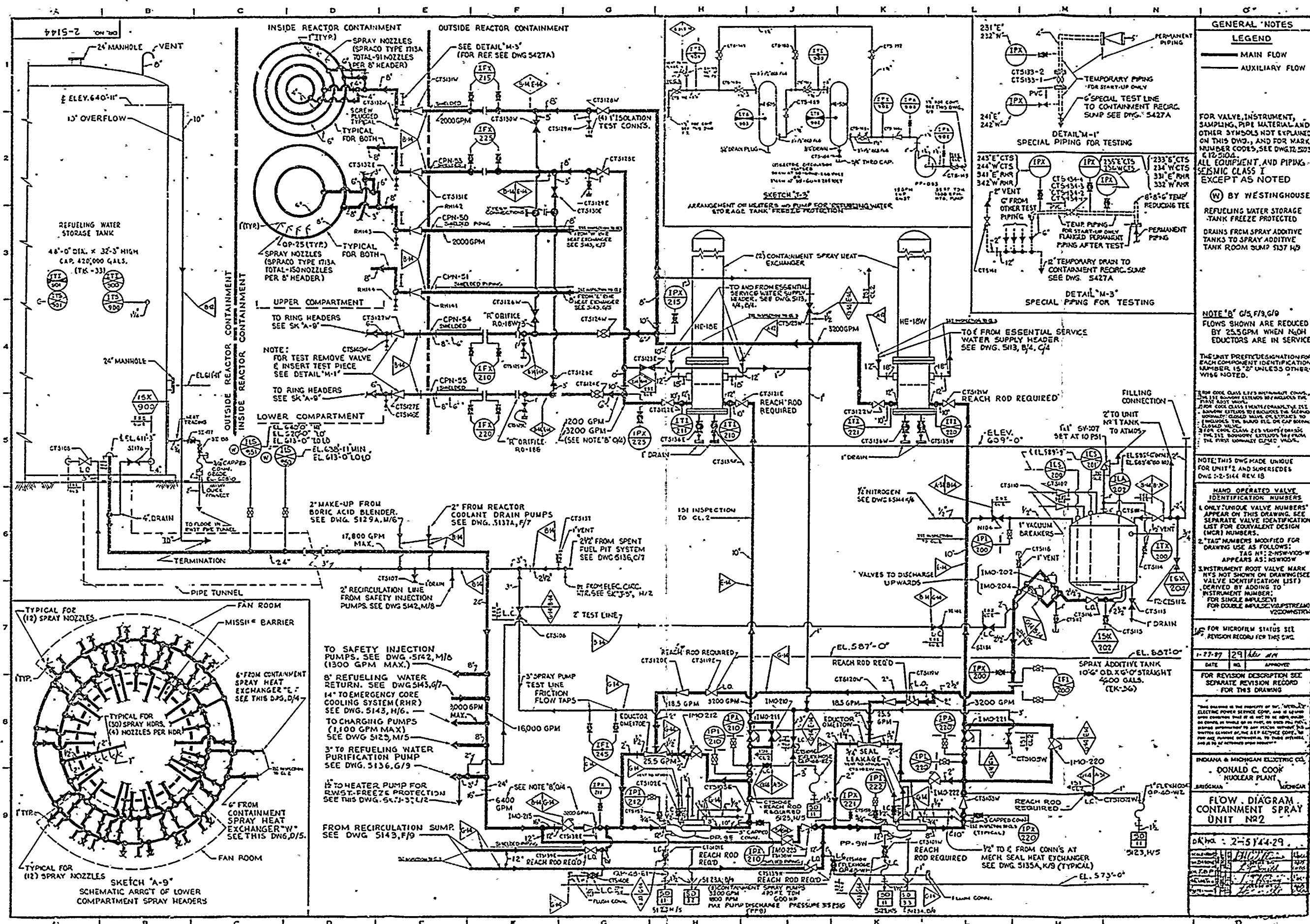
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OPERATIONAL USE



8710130278-19

"NOT FOR D.C. COOK"
OPERATIONAL USE

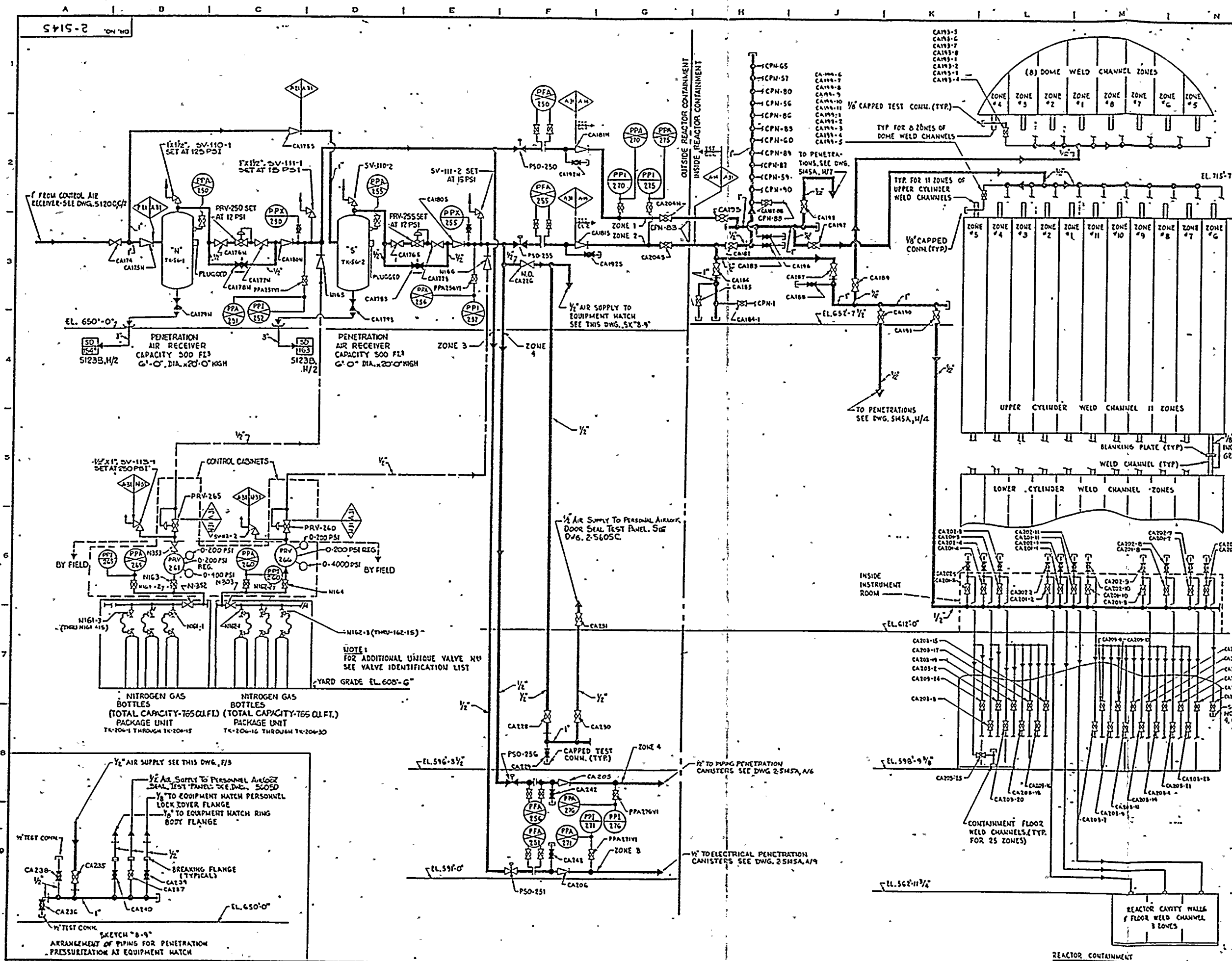


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8710130278-20

100



GENERAL NOTES

LEGEND

--- NITROGEN
--- CONTROL AIR
--- AUX. PIPING

FOR VALVE, INSTRUMENT, SAMPLING, PIPE MATERIAL AND OTHER SYMBOLS NOT EXPLAINED ON THIS DWG., AND FOR MARK NUMBER CODES, SEE DWGS. 12-503 AND 12-5104. ALL EQUIPMENT SEISMIC CLASS II.

NOTE 1. REFER TO DWGS. 5336 & 5337 FOR LIST OF PIPING PENETRATIONS. ALL PIPING CLASS A-31.

NOTE 2. UNIQUE VALVE NUMBERS ARE DERIVED FOR CONTAINMENT PENETRATIONS BY USING THE HEADER VALVE NUMBER PLUS THE CPH NUMBER.

NOTE 3. FOR ADDITIONAL UNIQUE VALVE NUMBERS SEE VALVE IDENTIFICATION LIST.

NOTE 4. FOR COC CLASS 2 INSTRUMENT CONNECTIONS, THE 1ST BOUNDARY EXTENDS TO AND INCLUDES THE FIRST ROOT VALVE.

NOTE 5. THE UNIT PREFIX DESIGNATION FOR EACH COMPONENT IDENTIFICATION NUMBER IS "2" UNLESS OTHERWISE NOTED.

NOTE: THIS DWG. MADE UNKNOWN FOR UNIT 2 AND SUPERSEDED DWG. 2-5145 REV. 10

HAND OPERATED VALVE IDENTIFICATION NUMBERS

1. ONLY "UNIQUE VALVE NUMBERS" APPEAR ON THIS DRAWING. SEE SEPARATE VALVE IDENTIFICATION LIST FOR EQUIVALENT DESIGN (MCR) NUMBERS.

2. "TAG" NUMBERS MODIFIED FOR DRAWING USE AS FOLLOWS:
TAG NO: 2-NSW-VIOS-W
APPEARS AS: 2-NSW-VIOS-W

3. INSTRUMENT ROOT VALVE MARK NOT SHOWN ON DRAWING (SEE VALVE IDENTIFICATION LIST) DERIVED BY ADDING TO INSTRUMENT NUMBER:
FOR SINGLE INFLUENT: V2DOWNSTREAM
FOR DOUBLE INFLUENT: V2DOWNSTREAM

FOR MICROFILM STATUS SEE REVISION RECORD FOR THIS DWG.

DATE: 12-26-20
BY: [Signature]
APPROVED: [Signature]

FOR REVISION DESCRIPTION SEE SEPARATE REVISION RECORD FOR THIS DRAWING

THIS DRAWING IS THE PROPERTY OF THE AMERICAN ELECTRIC POWER SERVICE CORP. AND IS LOANED TO YOU ON THE CONDITION THAT IT IS NOT TO BE REPRODUCED OR COPIED IN ANY MANNER OR FOR ANY PURPOSE WITHOUT THE WRITTEN CONSENT OF THE AEP SERVICE CORP., AND IS TO BE RETURNED TO THE AEP SERVICE CORP. IMMEDIATELY UPON REQUEST.

INDIANA & MICHIGAN ELECTRIC CO.
DONALD C. COOK
NUCLEAR PLANT

BRIDGMAN MICHIGAN

FLOW DIAGRAM
CONTAINMENT PENETRATION & WELD CHANNEL PRESSURIZATION
UNIT #2
SHEET 1 OF 2

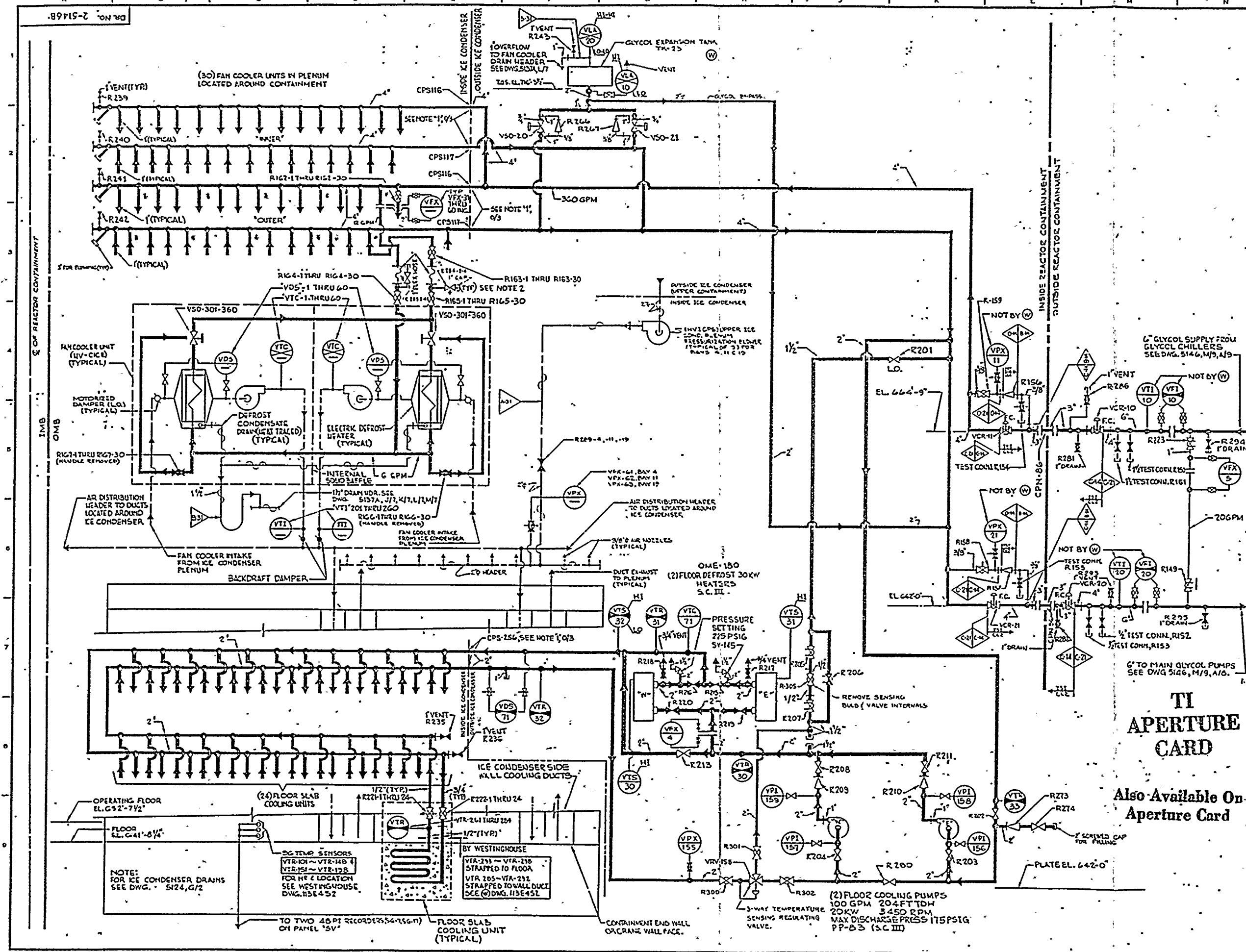
ORL NO. 2-5145-20

REACTOR CONTAINMENT

TI
APERTURE
CARD

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Aperture Card

8710130278-21



GENERAL NOTES

LEGEND

- ETHYLENE GLYCOL
- AIR
- AUXILIARY PIPING

FOR VALVE, INSTRUMENT SAMPLING, PIPE MATERIAL AND OTHER SYMBOLS NOT EXPLAINED ON THIS DWG AND FOR MARK NUMBER CODES, SEE DWG 504.

ALL EQUIPMENT VALVES AND INSTRUMENTATION, BY WESTINGHOUSE UNLESS OTHERWISE NOTED

ALL PIPING C-21 EXCEPT AS NOTED

ALL EQUIPMENT SEISMIC CLASS II EXCEPT AS NOTED

NOTE: 1" E/2, E/7 GLYCOL PIPING PENETRATES 125" END WALL ONLY.

NOTE: 2" B/3, C/3, D/3 GLYCOL CONNECTIONS FOR ICE BASKET WATER ADDITION EQUIPMENT SEE PH. 507, AND 1-4, 2-4, 3-4, 4-4

NOTE: 3" FOR CODE CLASS 1, 2, 3 INSTRUMENT CONNECTIONS, THE 125" BOUNDARY EXTENDS TO AND INCLUDES THE FIRST ROOT VALVE.

NOTE: 4" FOR CODE CLASS 1 VALVES AND DRAINS, THE 125" BOUNDARY EXTENDS TO AND INCLUDES THE SECOND ROOT VALVE. CLOSED VALVE OR EXTENDS TO AND INCLUDES THE BOUNDARY OF CAP AFTER A MANUALLY CLOSED VALVE.

NOTE: 5" FOR CODE CLASS 2, 3 VALVES AND DRAINS, THE 125" BOUNDARY EXTENDS TO AND INCLUDES THE FIRST UNUSUALLY CLOSED VALVE.

NOTE: THIS DWG MADE UNIQUE FOR UNIT 2 AND SUPERSEDES DWG 1-2-51468, REV 12

HAND OPERATED VALVE IDENTIFICATION NUMBERS

ONLY "UNIQUE" VALVE NUMBERS APPEAR ON THIS DRAWING. SEE SEPARATE VALVE IDENTIFICATION LIST FOR EQUIVALENT DESIGN (MCR) NUMBERS.

2. "TAG" NUMBERS MODIFIED FOR DRAWING USE AS FOLLOWS:

- TAG #11-2-NEW VOS-W APPEARS AS: NEW VOS-W

INSTRUMENT ROOT-VALVE MARK NOT SHOWN ON DRAWING (SEE VALVE IDENTIFICATION LIST) DERIVED BY ADDING TO INSTRUMENT NUMBER:

- FOR SINGLE PULSE: VI
- FOR DOUBLE PULSE: VDP
- FOR STREAM: VSTR
- FOR DOWNSTREAM: VDOWN

THE UNIT PREFIX DESIGNATION FOR EACH COMPONENT IDENTIFICATION NUMBER IS "2" UNLESS OTHERWISE NOTED.

DATE	BY	APPROVED
9-12-81	23	21

FOR REVISION DESCRIPTION SEE SEPARATE REVISION RECORD FOR THIS DRAWING

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INDIANA & MICHIGAN ELECTRIC CO.
DONALD C. COOK
NUCLEAR PLANT

FLOW DIAGRAM ICE CONDENSER REFRIGERATION

UNIT #2 SHEET 3 OF 3

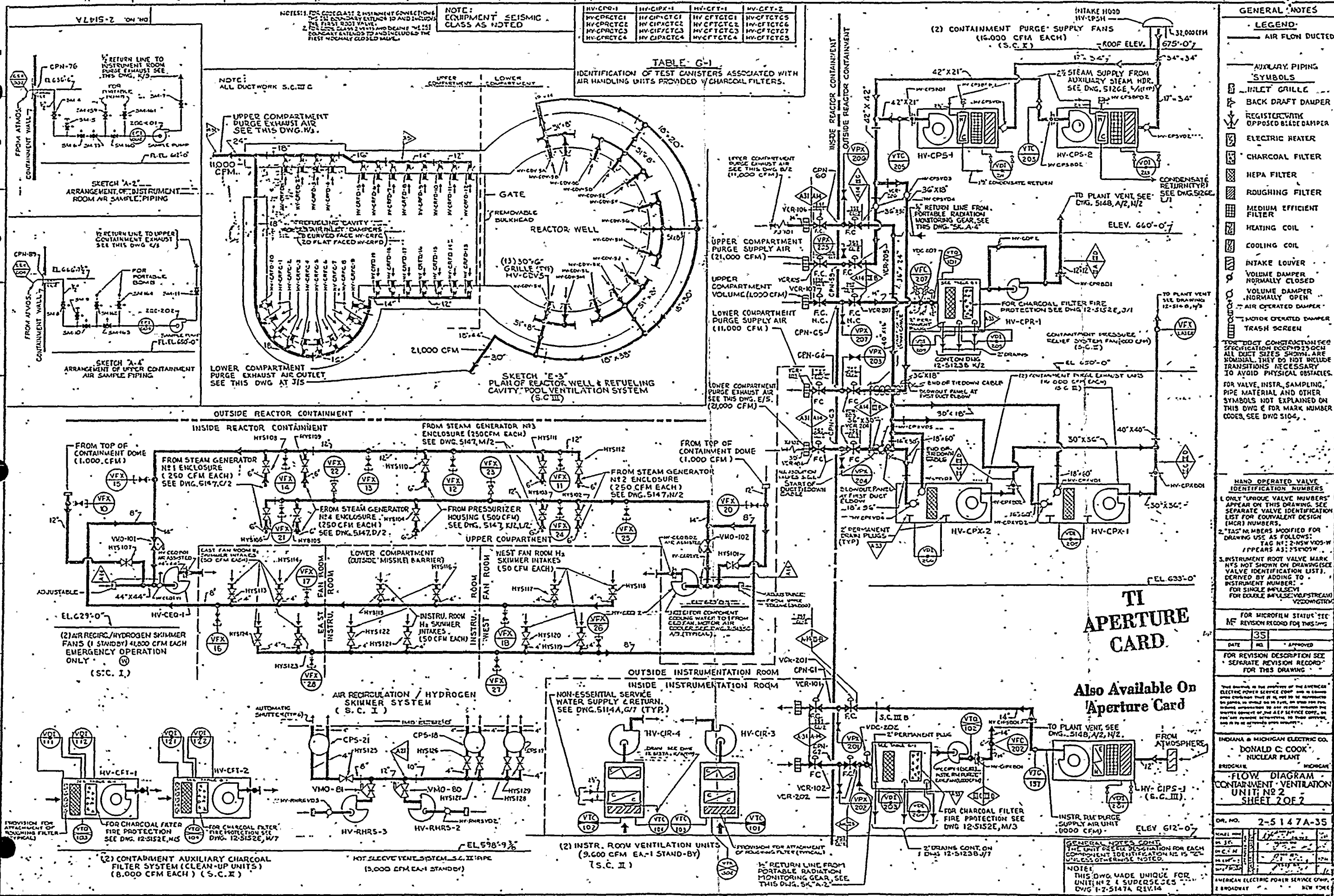
DR. NO. 2-51468-23

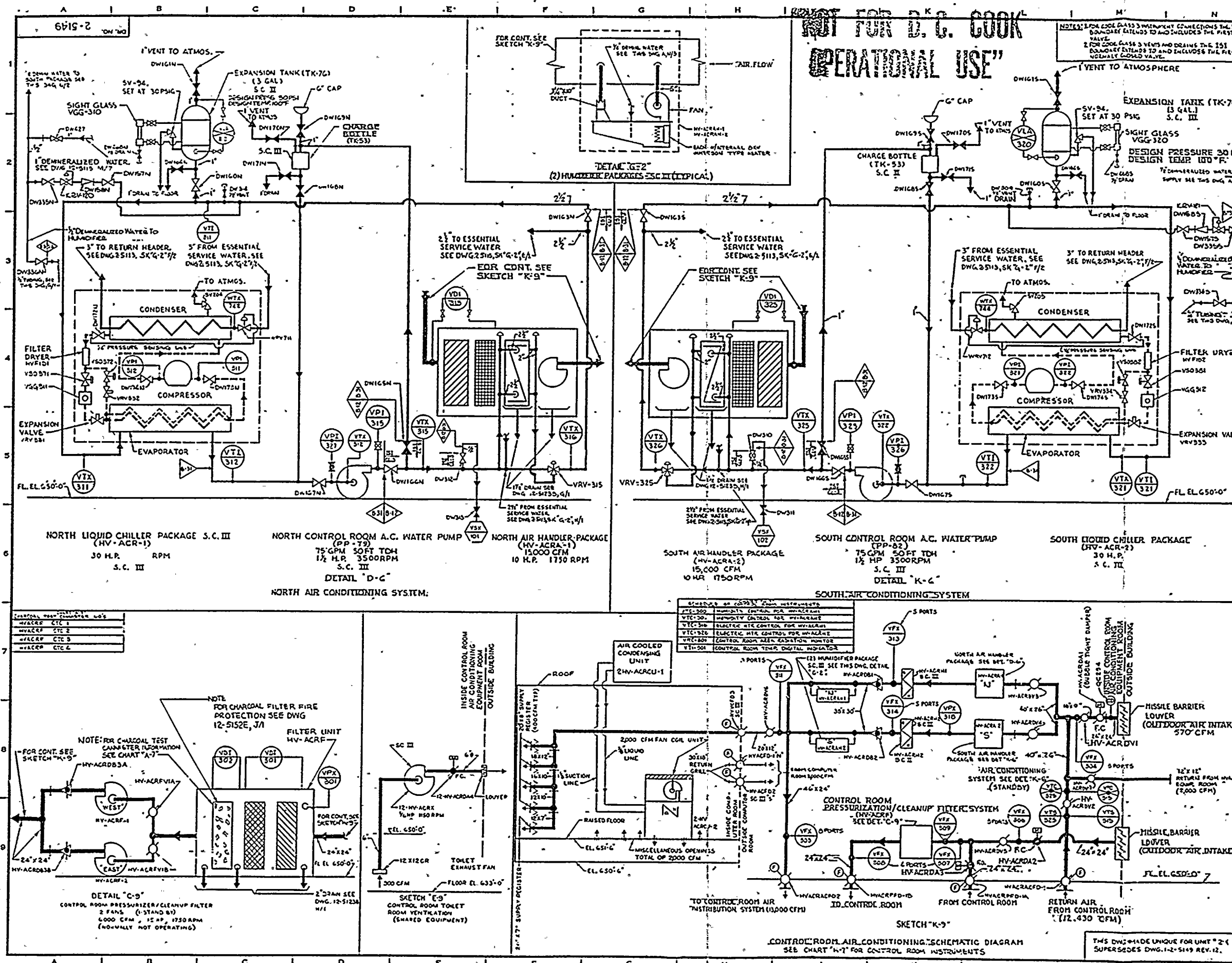
NO.	DATE	BY	APPROVED
1	9-12-81	23	21

AMERICAN ELECTRIC POWER SERVICE CORP.
2 BROADWAY NEW YORK

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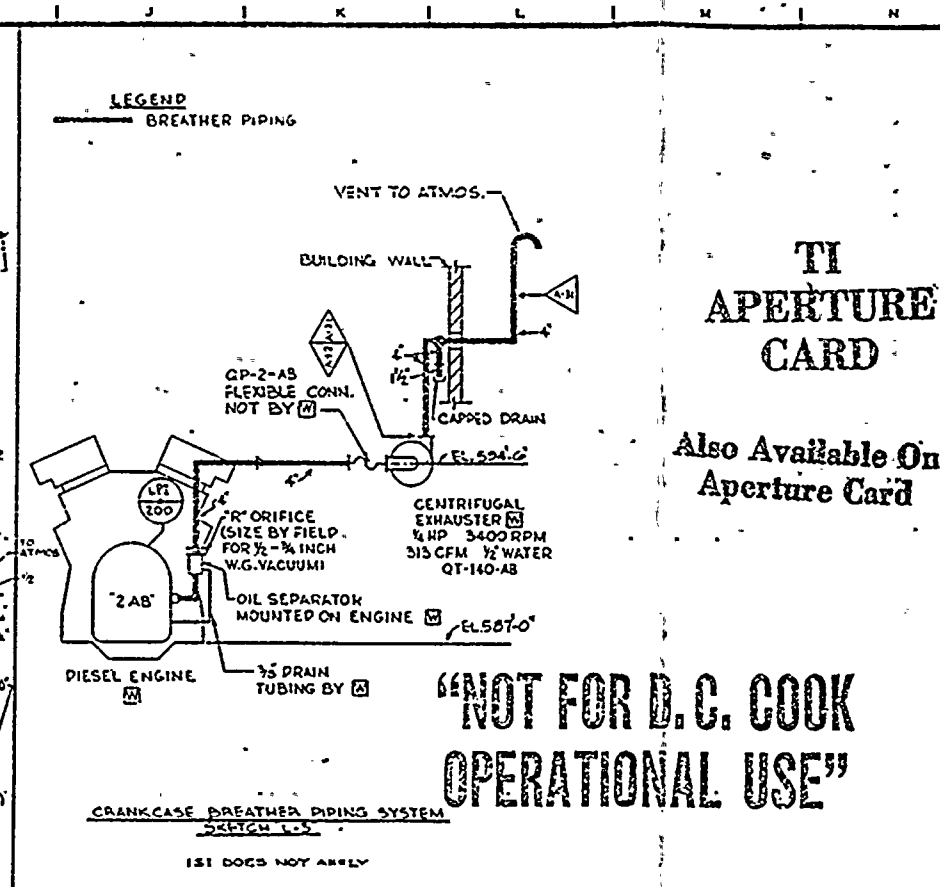
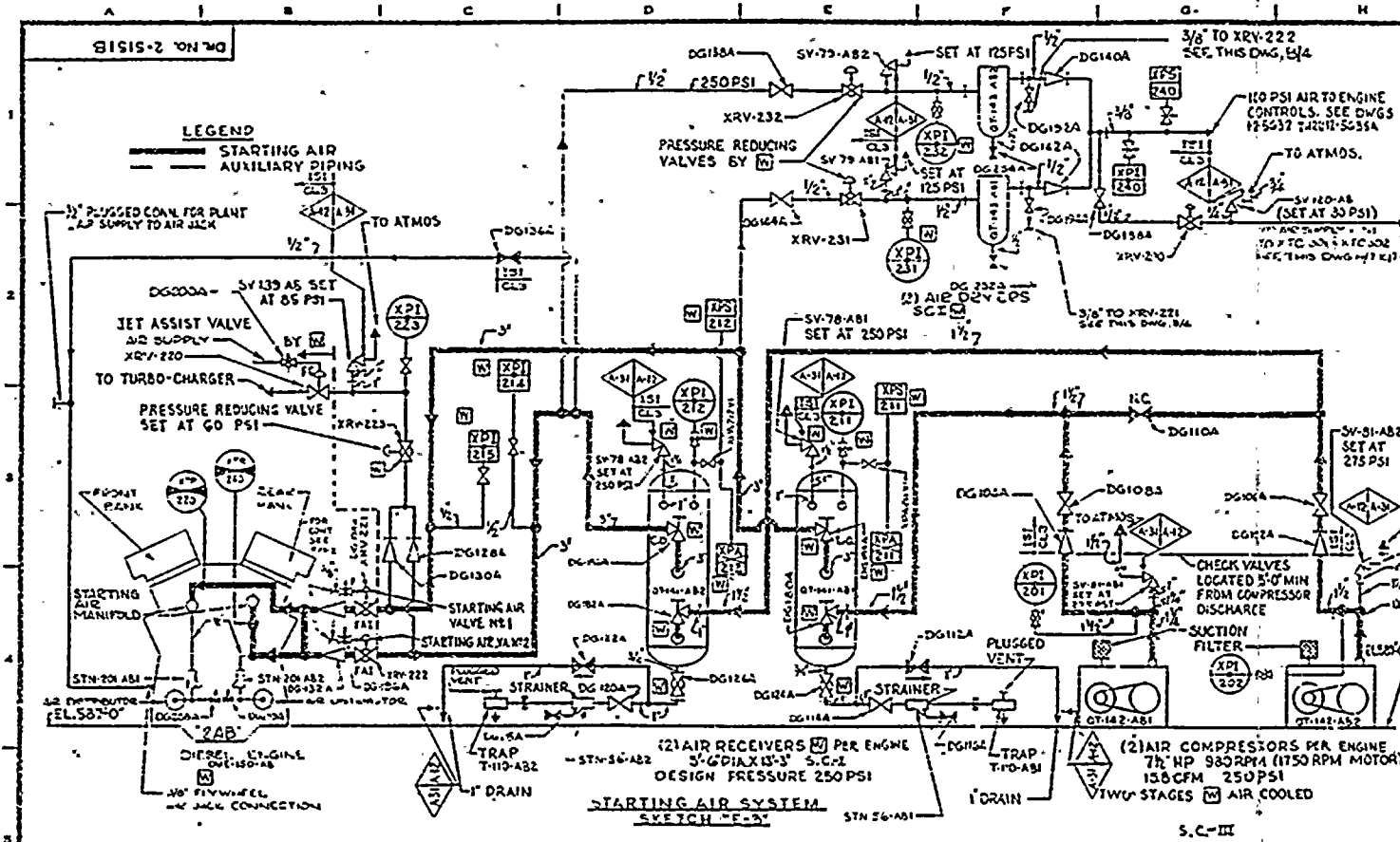




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8710130278-24



GENERAL NOTES:

LEGEND
AS NOTED

SYMBOLS
BY WORTHINGTON

* PIPING AND VALVES FURNISHED BY [] ARE NOTED.

FOR VALVE, INSTRUMENT, SAMPLING, PIPE MATERIAL AND OTHER SYMBOLS NOT EXPLAINED ON THIS DWG., AND FOR MARK-NUMBER CODES SEE DWG. 12-5103. & 12-5104.

THE UNIT PREFIX DESIGNATION FOR EACH COMPONENT IDENTIFICATION NUMBER IS "2" UNLESS OTHERWISE NOTED.

NOTE: E/S
ENCIRCLED LETTERS ARE SHOWN FOR ORIENTATION OF VALVE IN PIPING. THESE LETTERS REFLECT SIMILAR MARKINGS ON VALVE BODY.

NOTE: "F.A.I."
INDICATES (FAILS AS IS).

ALL DIESEL GENERATORS INCLUDING THEIR AUXILIARIES, STORAGE TANKS & PIPING ARE SEISMIC CLASS I EXCEPT AS NOTED.

ALL PIPING TO BE CLASS [] OR [] FOR EMBEDDED. EXCEPT AS NOTED.

CLASS NOTES:
ALL PIPING IS EQUIPPED TO BE 100% COOK CLASS I EXCEPT AS NOTED. FOR COOK CLASS I, THE PIPING SHALL BE 100% COOK CLASS I. THE PIPING SHALL BE 100% COOK CLASS I. THE PIPING SHALL BE 100% COOK CLASS I.

HAND OPERATED VALVE IDENTIFICATION NUMBERS:
1. ONLY "UNIQUE VALVE NUMBERS" APPEAR ON THIS DRAWING. SEE SEPARATE VALVE IDENTIFICATION LIST FOR EQUIVALENT DESIGN (MCR) NUMBERS.
2. "TAG" NUMBERS MODIFIED FOR DRAWING USE AS FOLLOWS:
TAG NO. 2-NSV-100-W APPEARS AS: NSV100W
3. INSTRUMENT ROOT VALVE MARK-INGS NOT SHOWN ON DRAWING (SEE VALVE IDENTIFICATION LIST) DERIVED BY ADDING TO INSTRUMENT NUMBER:
FOR SINGLE IMPLUSEV FOR DOUBLE IMPLUSEV FOR DOUBLE IMPLUSEV

FOR MICROFILM STATUS SET REVISION RECORD FOR THIS DWG.

DATE	NO.	APPROVED
11-3-62	27	[Signature]

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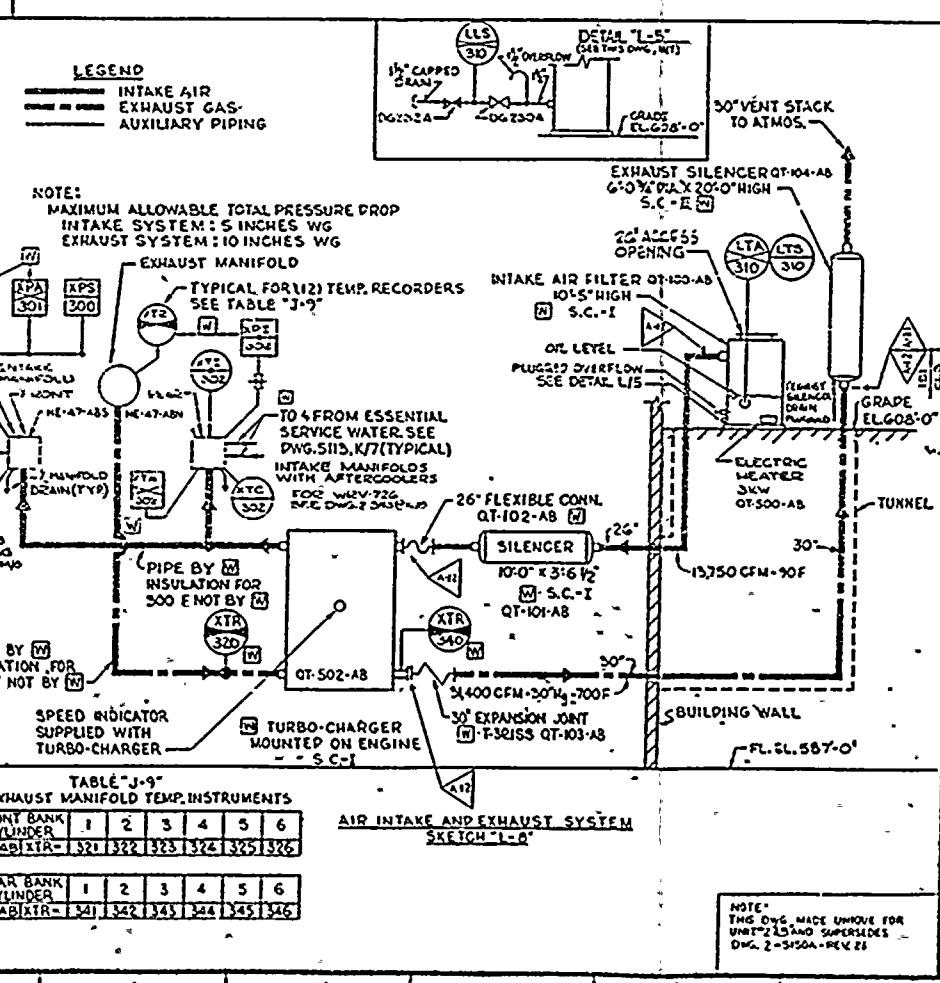
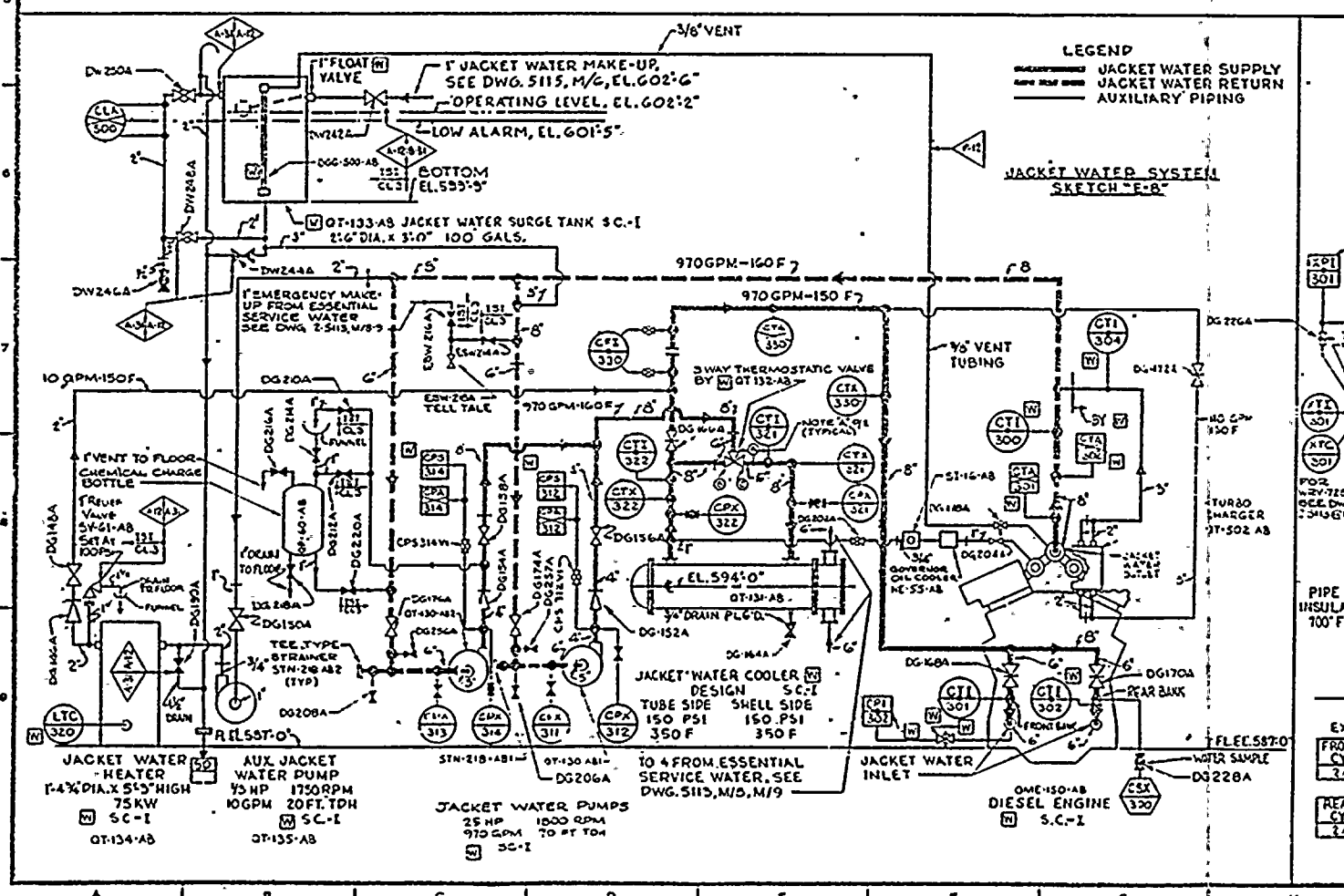
INDIANA & MICHIGAN ELECTRIC CO.
DONALD C. COOK
NUCLEAR PLANT

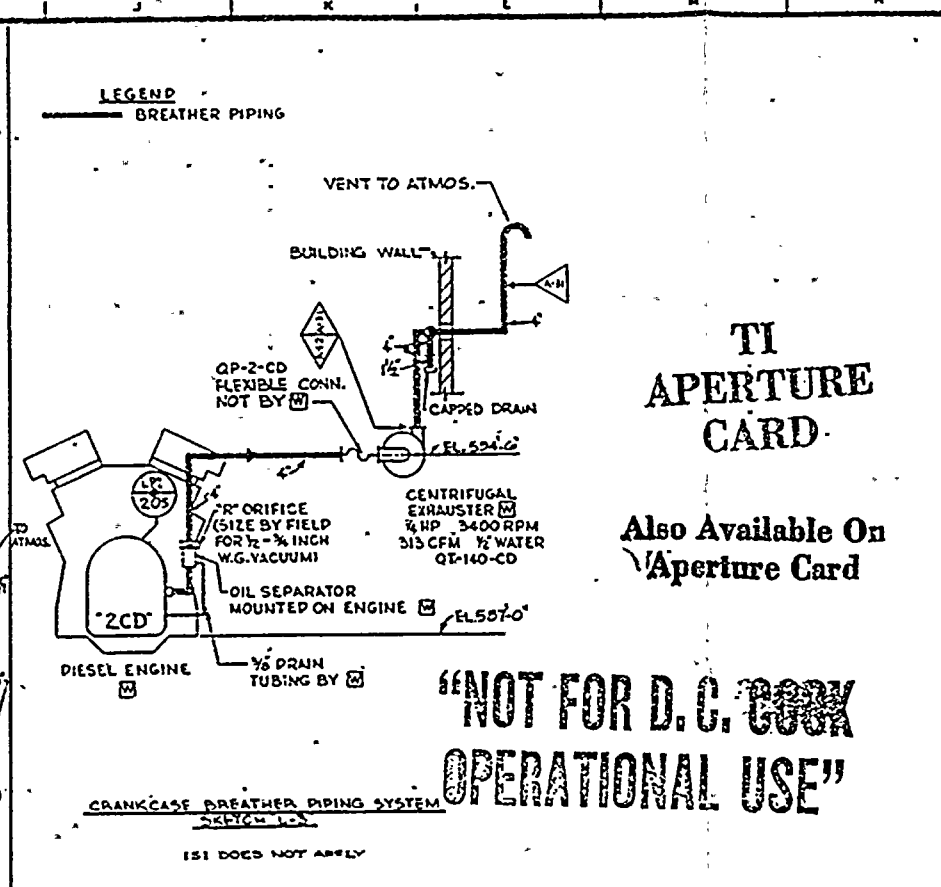
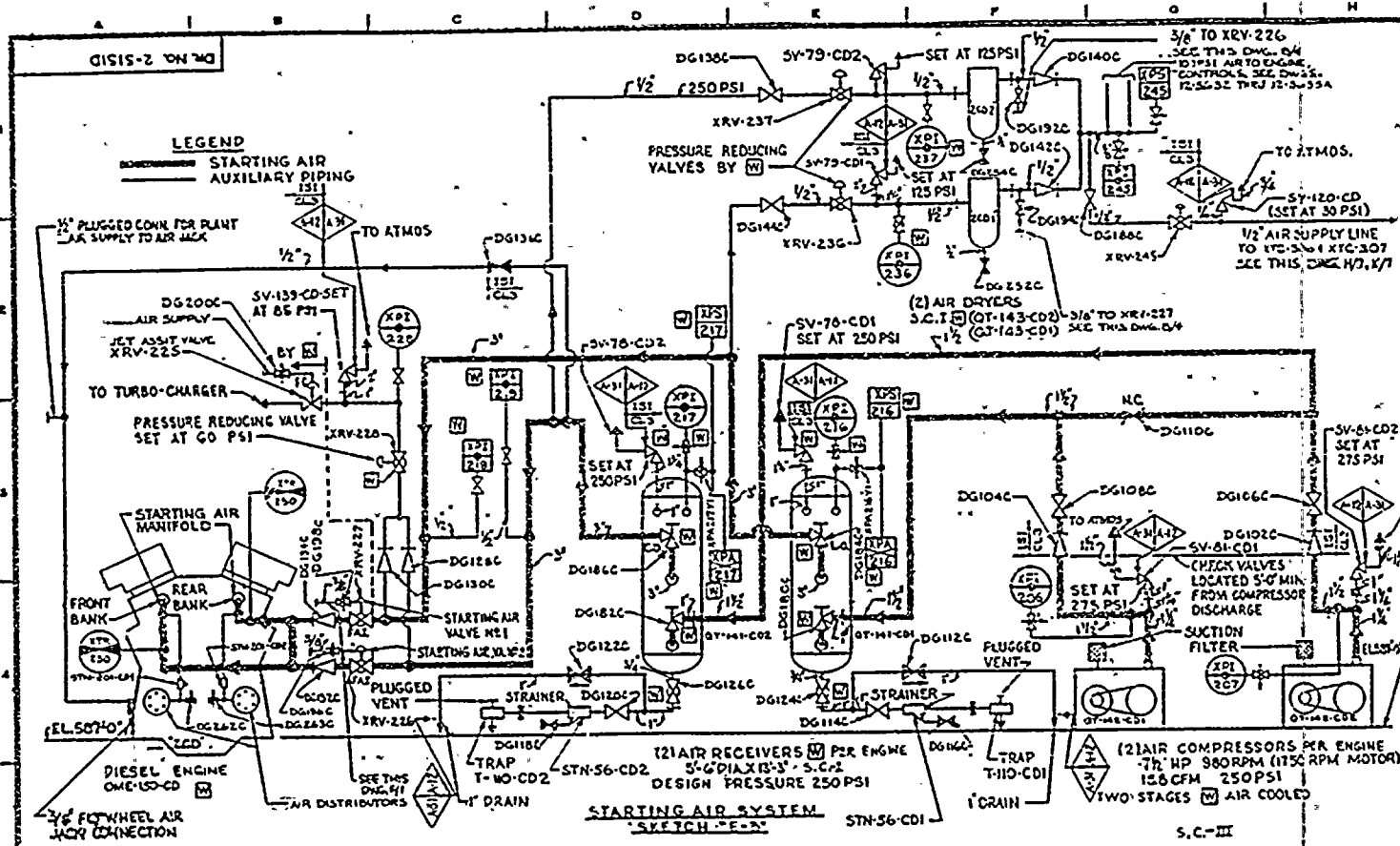
FLOW DIAGRAM EMERGENCY DIESEL GENERATOR "AB"

UNIT 2-5151B-27

DATE	NO.	APPROVED
11-3-62	27	[Signature]

AMERICAN ELECTRIC POWER SERVICE CORP.
2 BROADWAY
NEW YORK





GENERAL NOTES

LEGEND
 AS NOTED

SYMBOLS
 BY WORTHINGTON

PIPING AND VALVES
 FURNISHED BY ARE
 NOTED.

FOR VALVE, INSTRUMENT,
 SAMPLING, PIPE MATERIAL AND
 OTHER SYMBOLS NOT
 EXPLAINED ON THIS DWG., AND
 FOR MARK NUMBER CODES
 SEE DWG. 5104

NOTE A/B
 ENCIRCLED LETTERS ARE
 SHOWN FOR ORIENTATION OF
 VALVE IN PIPING.
 THESE LETTERS REFLECT
 SIMILAR MARKINGS ON
 VALVE BODY

NOTE 1
 THE UNIT PREFIX DESIGNATION
 FOR EACH COMPONENT IDENTIFICATION
 NUMBER, IS "C" UNLESS
 OTHERWISE NOTED.

NOTE 2
 "F.A.I." INDICATES VALVE FAILURE
 POSITION "FAIL AS IS"

ALL DIESEL-GENERATORS
 INCLUDING THEIR AUXILIARIES,
 STORAGE TANKS & PIPING ARE
 SEISMIC CLASS I EXCEPT
 AS NOTED.

ALL PIPING TO BE CLASS
 AND OR, S, FOR EMBEDDED,
 EXCEPT AS NOTED.

CODE CLASS NOTED
 ALL VALVES, INSTRUMENTS, AND
 PIPING TO BE CLASS AND OR, S,
 EXCEPT AS NOTED.

VALVE IDENTIFICATION NUMBERS
 1. ONLY "UNIQUE VALVE NUMBERS"
 APPEAR ON THIS DRAWING. SEE
 SEPARATE VALVE IDENTIFICATION
 LIST FOR EQUIVALENT DESIGN
 (EQR) NUMBERS.

2. "TAG" NUMBERS MODIFIED FOR
 DRAWING USE AS FOLLOWS:
 TAG NO. 100-100-W
 APPEARS AS: 100W

3. INSTRUMENT ROOT VALVE MARK
 IS NOT SHOWN ON DRAWING (SEE
 VALVE IDENTIFICATION LIST)
 SERVED BY ADJACENT TO
 INSTRUMENT NUMBER.
 FOR SINGLE BRANCH,
 FOR DOUBLE BRANCH, VALVE
 IDENTIFICATION NUMBER

FOR MICROFILM STATUS SEE
 REVISION RECORD FOR THIS DWG.

DATE 27 1963
 BY 27 1963

FOR REVISION DESCRIPTION SEE
 SEPARATE REVISION RECORD
 FOR THIS DRAWING

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INDIANA & MICHIGAN ELECTRIC CO.
 DONALD G. COOK
 NUCLEAR PLANT

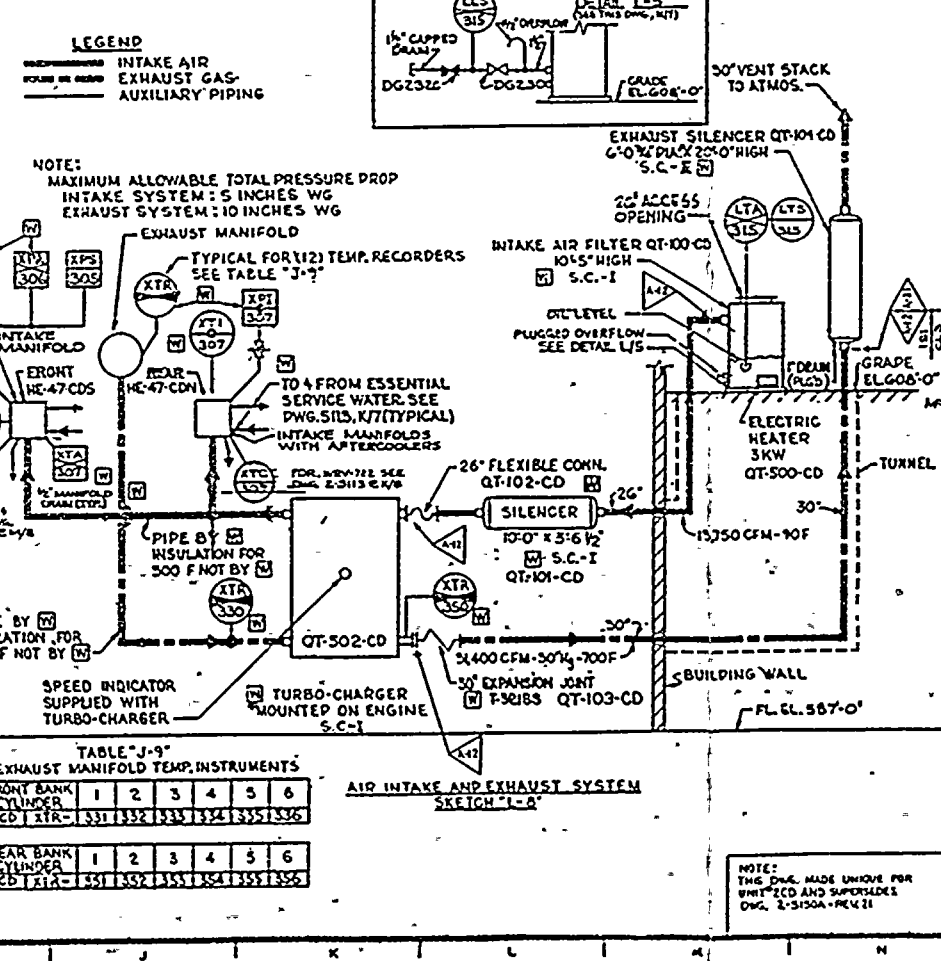
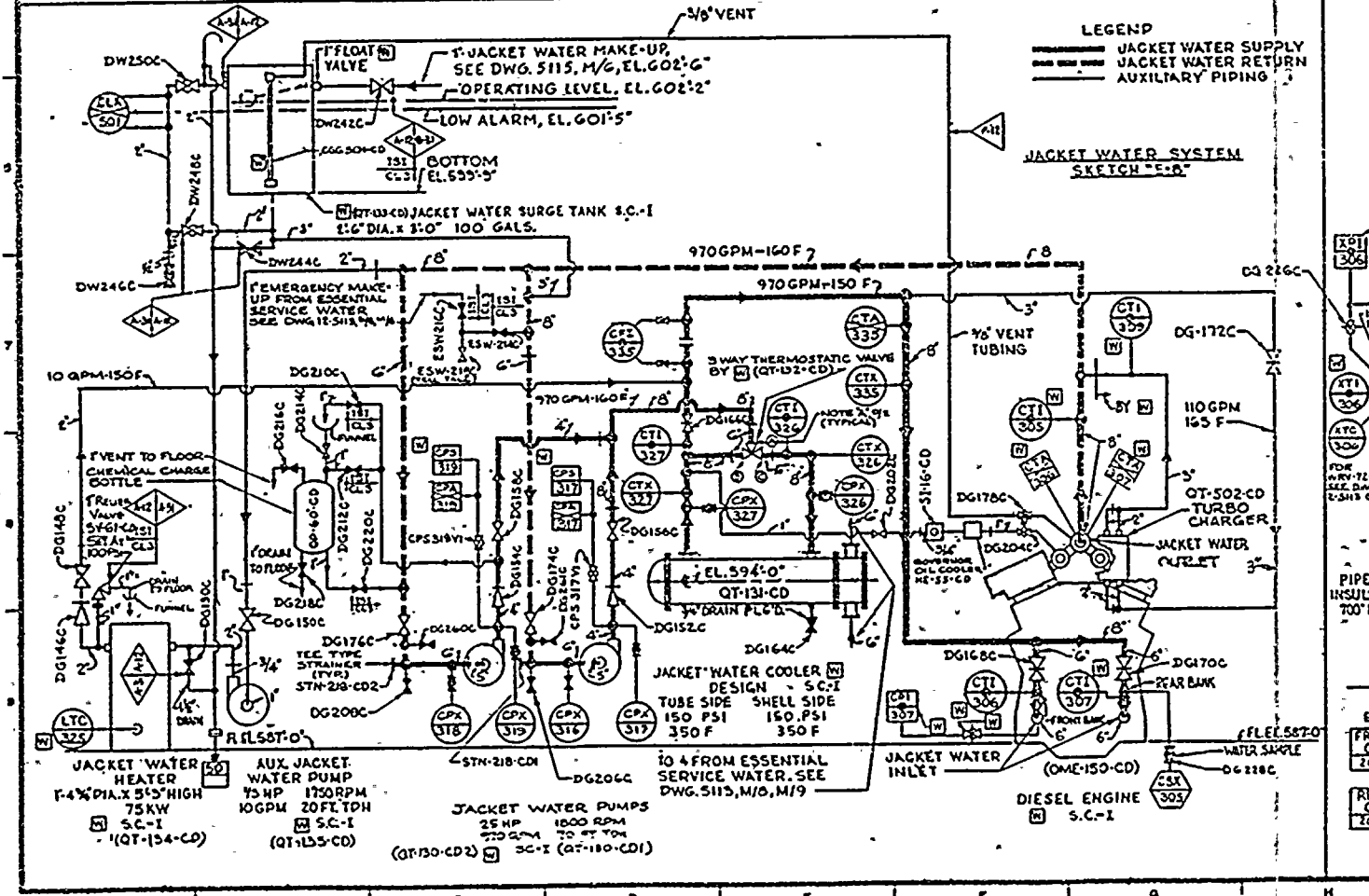
**FLOW DIAGRAM
 EMERGENCY DIESEL
 GENERATOR CD**

UNIT M72
 DWG NO 2-5151D-27

DATE 27 1963
 BY 27 1963

THIS DWG. MADE UNIQUE FOR
 UNIT 2 AND SUPERSEDES
 DWG. 2-5150A-REV 21

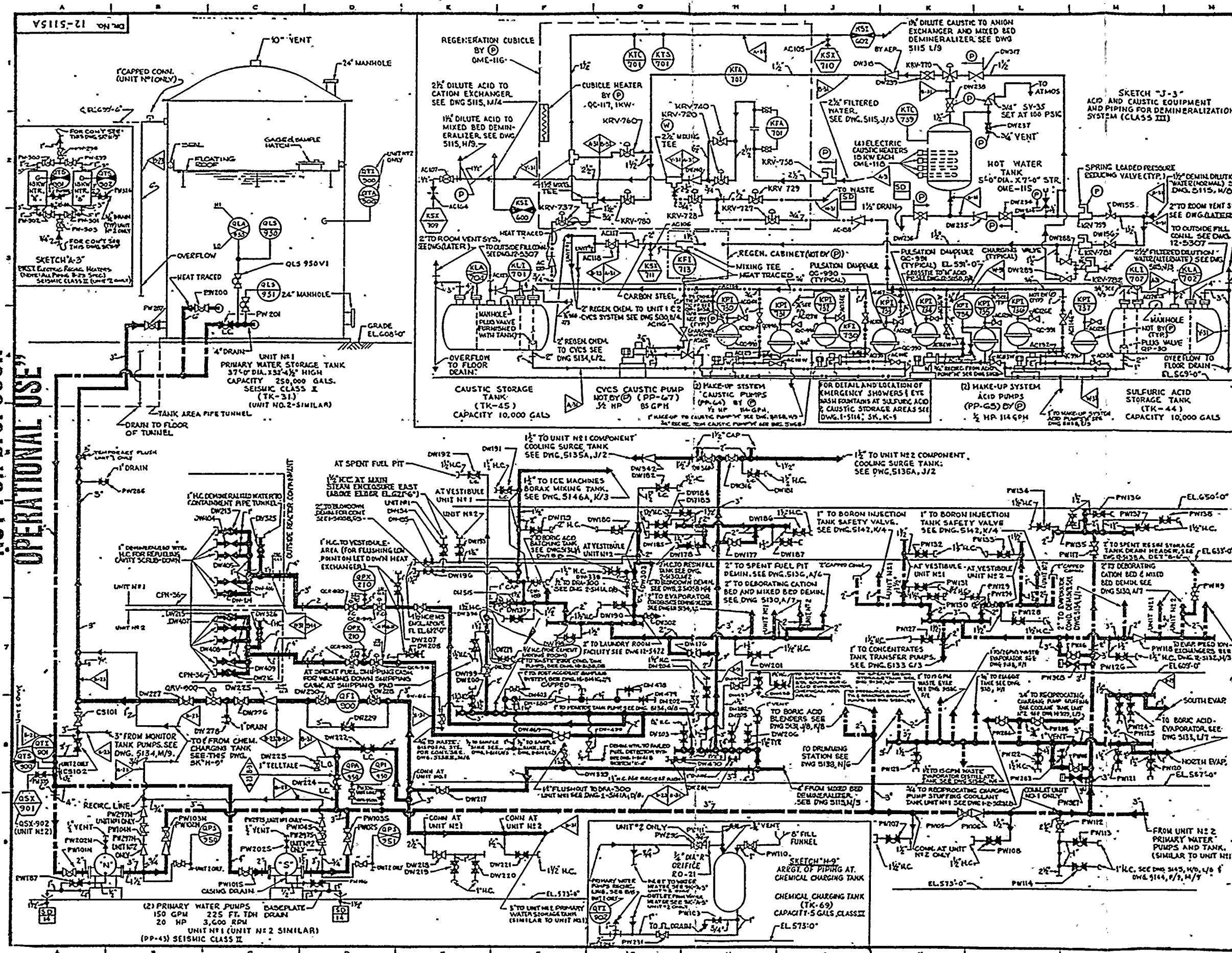
AMERICAN ELECTRIC POWER SERVICE CORP.
 2 BROADWAY
 NEW YORK



NOTE:
 THIS DWG. MADE UNIQUE FOR
 UNIT 2 AND SUPERSEDES
 DWG. 2-5150A-REV 21

AMERICAN ELECTRIC POWER SERVICE CORP.
 2 BROADWAY
 NEW YORK

**NOT FOR D.C. COOK
OPERATIONAL USE**



GENERAL NOTES

LEGEND

— WAKE-UP WATER
 — PRIMARY WATER
 — REGEN. PIPING
 — • — CONC. ACID
 — • • — CONC. CAUSTIC

SYMBOLS

(P) PERMUTIT
 H.C. ROSE CONNECTIONS
 FOR VALVE, INSTRUMENT
 SAMPLING PIPE MATERIAL
 AND OTHER SYMBOLS NOT
 EXPLAINED ON THIS DWG.,
 AND FOR MARK NUMBER
 CODES, SEE DWG. 5104.

"EQUIPMENT SEISMIC CLASS
AS NOTED"

THE UNIT PREFIX DESIGNATION FOR EACH COMPONENT MODIFICATION IS (12) UNLESS OTHERWISE NOTED.

HAND OPERATED VALVE IDENTIFICATION NUMBERS

1. ONLY UNIQUE VALVE NUMBERS APPEAR ON THIS DRAWING. SEE SEPARATE VALVE IDENTIFICATION LIST FOR EQUIVALENT DESIGN. (SICR) NUMBERS.

2. TAG NUMBERS MODIFIED FOR DRAWING USE AS FOLLOWS:
TAG N1 = 2-N5W4YOS-W
APPEARS AS: N5W4YOS

3. INSTRUMENT ROOT VALVE MARK N1'S NOT SHOWN ON DRAWING (SEE VALVE IDENTIFICATION LIST) DERIVED BY ADDING TO INSTRUMENT NUMBER:
FOR 1500 PSI INSTRUMENT FOR DOUBLE IMPULSIVE STREAMS
VCCDNGTSM

12-2-86	41	<i>W. J. A. J.</i>
DATE	NO.	APPROVED

FOR REVISION DESCRIPTION SEE
SEPARATE REVISION RECORD
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INDIANA & MICHIGAN ELECTRIC CO.
DONALD C. COOK
NUCLEAR PLANT

**FLOW DIAGRAM
MAKE-UP WATER &
PRIMARY WATER SYSTEMS**

EXCEPTIONS ARE NOTED
DR. No. 12-5115A-41.

FILE NO.	100	101	102	103	104	105	106	107	108	109	110	111	112	113	114	115	116	117	118	119	120	121	122	123	124	125	126	127	128	129	130	131	132	133	134	135	136	137	138	139	140	141	142	143	144	145	146	147	148	149	150	151	152	153	154	155	156	157	158	159	160	161	162	163	164	165	166	167	168	169	170	171	172	173	174	175	176	177	178	179	180	181	182	183	184	185	186	187	188	189	190	191	192	193	194	195	196	197	198	199	200	201	202	203	204	205	206	207	208	209	210	211	212	213	214	215	216	217	218	219	220	221	222	223	224	225	226	227	228	229	230	231	232	233	234	235	236	237	238	239	240	241	242	243	244	245	246	247	248	249	250	251	252	253	254	255	256	257	258	259	260	261	262	263	264	265	266	267	268	269	270	271	272	273	274	275	276	277	278	279	280	281	282	283	284	285	286	287	288	289	290	291	292	293	294	295	296	297	298	299	300	301	302	303	304	305	306	307	308	309	310	311	312	313	314	315	316	317	318	319	320	321	322	323	324	325	326	327	328	329	330	331	332	333	334	335	336	337	338	339	340	341	342	343	344	345	346	347	348	349	350	351	352	353	354	355	356	357	358	359	360	361	362	363	364	365	366	367	368	369	370	371	372	373	374	375	376	377	378	379	380	381	382	383	384	385	386	387	388	389	390	391	392	393	394	395	396	397	398	399	400	401	402	403	404	405	406	407	408	409	410	411	412	413	414	415	416	417	418	419	420	421	422	423	424	425	426	427	428	429	430	431	432	433	434	435	436	437	438	439	440	441	442	443	444	445	446	447	448	449	450	451	452	453	454	455	456	457	458	459	460	461	462	463	464	465	466	467	468	469	470	471	472	473	474	475	476	477	478	479	480	481	482	483	484	485	486	487	488	489	490	491	492	493	494	495	496	497	498	499	500	501	502	503	504	505	506	507	508	509	510	511	512	513	514	515	516	517	518	519	520	521	522	523	524	525	526	527	528	529	530	531	532	533	534	535	536	537	538	539	540	541	542	543	544	545	546	547	548	549	550	551	552	
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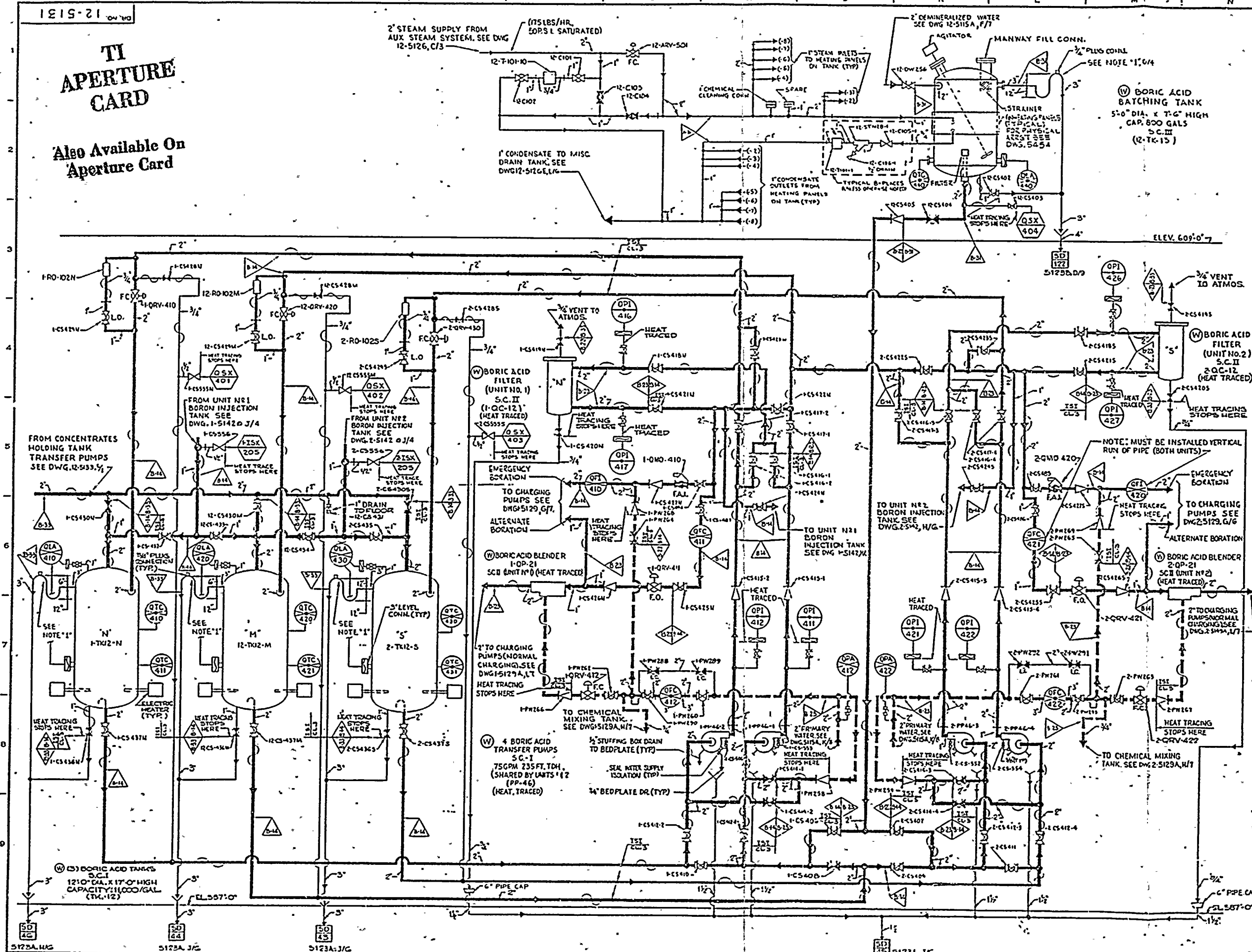
**Also Available On
Aperture Card**

87 10 130278-29

1215-21

TI APERTURE CARD

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GENERAL NOTES

- LEGEND
- BORATED WATER
 - PRIMARY WATER
 - AUXILIARY PIPING
 - SYMBOLS
 - DIAPHRAGM SEAL

FOR VALVE, INSTRUMENT SAMPLING, PIPE MATERIAL AND OTHER SYMBOLS NOT EXPLAINED ON THIS DWG., AND FOR MARK NUMBER, CODES, SEE DWGS. 12-5103 & 5104.

BY WESTINGHOUSE

ALL VALVES AND INSTRUMENTATION SUPPLIED BY (W) EXCEPT AS NOTED.

EQUIPMENT SUPPLIED BY (W) AS NOTED.

NOTE "1": A/C, C/G, D/C, L/I OVERFLOW LOOP SEALS TO BE FILLED WITH DEMINERALIZED WATER. WHENEVER TANKS ARE OVERFLOWED, LOOP SEALS ARE TO BE FLUSHED TO PREVENT BORIC ACID CRYSTALLIZATION.

SEISMIC CLASSIFICATION OF EQUIPMENT AS NOTED

NOTES
FOR CLASS "3" INSTRUMENT CONNECTIONS, THE 1ST BOUNDARY EXTENDS TO AND INCLUDES THE FIRST ROOT VALVE.

THE UNIT IDENTIFICATION FOR EACH COMPONENT IDENTIFICATION NUMBER IS "12" UNLESS OTHERWISE NOTED

HAND OPERATED VALVE IDENTIFICATION NUMBERS
1. ONLY "UNIQUE VALVE NUMBERS" APPEAR ON THIS DRAWING. SEE SEPARATE VALVE IDENTIFICATION LIST FOR EQUIVALENT DESIGN (MER) NUMBERS.
2. "TAG" NUMBERS MODIFIED FOR DRAWING USE AS FOLLOWS:
TAG NO.: 2-NSW-VI05-W
APPEARS AS: NSW05W
3. INSTRUMENT ROOT VALVE MARK NOT SHOWN ON DRAWING (SEE VALVE IDENTIFICATION LIST) DERIVED BY ADDING TO INSTRUMENT NUMBER:
FOR SINGLE IMPLUSE, VI
FOR DOUBLE IMPLUSE, VI05W (SEE INSTRUMENT)

FOR MICROFILM STATUS SEE REVISION RECORD FOR THIS DWG.

DATE 12-27-79 BY FAW/AD

FOR REVISION DESCRIPTION SEE SEPARATE REVISION RECORD FOR THIS DRAWING

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INDIANA & MICHIGAN ELECTRIC CO.
DONALD C. COOK
NUCLEAR PLANT 1

FLOW DIAGRAM
CVCS-BORON
MAKE-UP
UNITS 1 & 2

DWG. NO. 12-5131-195

AMERICAN ELECTRIC POWER SERVICE CO.
2 BROADWAY

8710130278-31

