



Idaho National Engineering Laboratory

PDR

March 19, 1987

Mr. M. Carrington, Project Manager
Office of Nuclear Reactor Regulation
U. S. Nuclear Regulatory Commission
Washington, D.C. 20555

TRANSMITTAL OF FINAL REPORT, "CONFORMANCE TO GENERIC LETTER 83-28,
ITEMS 2.1 (PART 1) EQUIPMENT CLASSIFICATION, COOK UNITS 1 AND 2"
EGG-NTA-7609, MARCH 1987, (FIN D6001) EWR-55-87

Ref: NRC Form 189, "Evaluation-Conformance to Generic Letter 83-28,
Required Actions Based on Generic Implications of Salem ATWS
Events" (FIN-D6001), June 1986.

Dear Mr. Carrington:

Transmitted herewith is the above subject report.

The report documents INEL's evaluation of the licensee actions in response to the NRC Generic Letter 83-28, Item 2.1 (Part 1), Equipment Classification (RTS Components). Based on the licensee submittals, we conclude that the plant conforms to the requirements of the generic letter on this item.

Very truly yours,

E. W. Roberts, Manager
NRC Headquarters Support

gmp

Enclosure:
As Stated

cc: A. Toalston, NRR-PAEI (2)
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CONFORMANCE TO GENERIC LETTER 83-28
ITEM 2.1 (PART 1) EQUIPMENT CLASSIFICATION (RTS COMPONENTS)
COOK UNITS 1 AND 2

Docket Nos. 50-315/316

R. Haroldsen

Published March 1987

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Prepared for the
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555
Under DOE Contract No. DE-AC07-76ID01570
FIN No. D6001

ABSTRACT

This EG&G Idaho, Inc. report provides a review of the submittals from Cook Units 1 and 2 for conformance to Generic Letter 83-28, Item 2.1 (Part 1), equipment classification of reactor trip system components.

Docket Nos. 50-315/316
TAC Nos. 52989 and 52990

FOREWORD

This report is supplied as part of the program for evaluating Licensee/applicant conformance to Generic Letter 83-28, "Required Actions Based on Generic Implications of Salem ATWS Events." This work is being conducted for the U.S. Nuclear Regulatory Commission, Office of Nuclear Reactor Regulation, Division of PWR Licensing-A, by the EG&G Idaho, Inc.

The U.S. Nuclear Regulatory Commission funded this work under the authorization B&R No. 20-19-10-11-3, FIN No. D6001.

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1. INTRODUCTION AND SUMMARY

On February 25, 1983, both of the scram circuit breakers at Unit 1 of the Salem Nuclear Power Plant failed to open upon an automatic reactor trip signal from the reactor protection system. This incident was terminated manually by the operator about 30 seconds after the initiation of the automatic trip signal. The failure of the circuit breakers was determined to be related to the sticking of the undervoltage trip attachment. Prior to the incident, on February 22, 1983, an automatic trip signal was generated at Unit 1 of the Salem Nuclear Power Plant based on steam generator low-low level during plant startup. In this case, the reactor was tripped manually by the operator almost coincidentally with the automatic trip.

Following these incidents, on February 28, 1983, the NRC Executive Director of Operations (EDO), directed the staff to investigate the report on the generic implications of these occurrences at Unit 1 of the Salem Nuclear Power Plant. The results of the staff's inquiry into the generic implications of the Salem Unit 1 incidents are reported in NUREG-1000, "Generic Implications of the ATWS Events at the Salem Nuclear Power Plant."¹ As a result of this investigation, the Commission (NRC) requested (by Generic Letter 83-28, dated July 8, 1983)² all licensees of operating reactors, applicants for an operating license, and holders of construction permits to respond to generic issues raised by the analyses of these two ATWS events.

Part 1 of Item 2.1 of Generic Letter 83-28 requires the licensee or applicant to confirm that all reactor trip system components are identified, classified, and treated as safety-related as indicated in the following statement:

Licensees and applicants shall confirm that all components whose functioning is required to trip the reactor are identified as safety-related on documents, procedures, and information handling systems used in the plant to control safety-related activities, including maintenance, work orders, and parts replacement.

2. PLANT RESPONSE EVALUATION

The licensee for Cook Units 1 and 2, Indiana and Michigan Electric Co., provided responses relating to Item 2.1 (Part 1) in submittals dated November 4, 1983, March 30, 1984, April 10, 1985 and December 19, 1986. The first of these submittals describes the criteria used by the licensee to identify the safety-related components of the reactor trip system.

The submittal states that the method used to control activities relating to safety-related components is different than that described in Item 2.1 (Part 1) of the Generic Letter 83-28. It is stated that the N-list is one of the documents utilized to identify equipment as being safety-related. However, it also states that the list is not all inclusive and therefore not the sole source of reference. Other documents such as the FSAR, Technical Specifications, communications to the NRC, flow diagrams, isometrics, electrical one line and elementary diagrams are also consulted.

In the subsequent submittals, the licensee stated that a computerized component classification data-base was being implemented which will incorporate references to existing relevant drawings and procedures. The data-base has been designed to integrate existing documents and information systems to manage purchase, maintenance and replacement of safety-related components.

The December 19, 1986 submittal confirmed that computerized component classification system had been completed and is functional.

2.1 Conclusion

Based on our review of the licensee's submittals, we find that the licensee has identified the components necessary to perform reactor trip and that these components are classified safety-related in an equipment classification system which controls activities relating to the

safety-related components. We therefore, find that the licensee's responses meet the requirements of Item 2.1 (Part 1) of Generic Letter 83-28 and are acceptable.

References

1. Letter, M. P. Alexich, Indiana and Michigan Electric Co., to D. G. Eisenhut, NRC, November 4, 1983.
2. Letter, M. P. Alexich, Indiana and Michigan Electric Co., to D. G. Eisenhut, NRC, March 30, 1984.
3. Letter, M. P. Alexich, Indiana and Michigan Electric Co., to D. G. Eisenhut, NRC, April 10, 1985.
4. Letter, M. P. Alexich, Indiana and Michigan Electric Co., to H. R. Denton, NRC, December 19, 1986.

3. GENERIC REFERENCES

1. Generic Implications of ATWS Events at the Salem Nuclear Power Plant, NUREG-1000, Volume 1 April 1983; Volume 2, July 1983.
2. NRC Letter, D. G. Eisenhut to all Licensees of Operating Reactors, Applicants for Operating License, and Holders of Construction Permits, "Required Actions Based on Generic Implications of Salem ATWS Events (Generic Letter 83-28)," July 8, 1983.

BIBLIOGRAPHIC DATA SHEET

EGG-NTA-7609

SEE INSTRUCTIONS ON THE REVERSE

2 TITLE AND SUBTITLE

CONFORMANCE TO GENERIC LETTER 83-28, ITEM 2.1
(PART 1), EQUIPMENT CLASSIFICATION
COOK UNITS 1 AND 2

J LEAVE BLANK

4 DATE REPORT COMPLETED

MONTH

YEAR

March

1987

6 DATE REPORT ISSUED

MONTH

YEAR

March

1987

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7. PERFORMING ORGANIZATION NAME AND MAILING ADDRESS (Include 2-d Code)

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8. PROJECT/TASK/WORK UNIT NUMBER

9. FIN OR GRANT NUMBER

D6001

10. SPONSORING ORGANIZATION NAME AND MAILING ADDRESS (Include Zip Code)

Division of Systems Integration
Office of Nuclear Reactor Regulation
U. S. Nuclear Regulatory Commission
Washington, DC 20555

11a. TYPE OF REPORT

Technical Evaluation Report

b. PERIOD COVERED (Inclusive dates)

12 SUPPLEMENTARY NOTES

13. ABSTRACT (200 words or less)

This EG&G Idaho, Inc. report provides a review of the submittals from Cook Units 1 and 2 for conformance to Generic Letter 83-28 Item 2.1 (Part 1) Equipment Classification (RTS Components).

14 DOCUMENT ANALYSIS a. KEYWORDS/DESCRIPTORS

b. IDENTIFIERS/OPEN-ENDED TERMS

15 AVAILABILITY STATEMENT

Unlimited
Distribution

16 SECURITY CLASSIFICATION

(This page)

Unclassified

(This report)

Unclassified

17. NUMBER OF PAGES

18 PRICE