

CATEGORY 1

REGULATOR INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR: 9710240049 DOC. DATE: 97/10/13 NOTARIZED: NO DOCKET #
 FACIL: 50-315 Donald C. Cook Nuclear Power Plant, Unit 1, Indiana M 05000315
 AUTH. NAME AUTHOR AFFILIATION
 PISARSKY, F. American Electric Power Co., Inc.
 BLIND, A.A. American Electric Power Co., Inc.
 RECIP. NAME RECIPIENT AFFILIATION

SUBJECT: LER 97-022-00: on 970911, determined that conditions had not been properly established in CCW sys to meet plant piping design code requirements. Caused by manual valves between two devices not controlled. Update scheduled. W/971013 ltr.

DISTRIBUTION CODE: IE22T COPIES RECEIVED: LTR 1 ENCL 1 SIZE: 2
 TITLE: 50.73/50.9 Licensee Event Report (LER), Incident Rpt, etc.

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Indiana Michigan
Power Company
Cook Nuclear Plant
One Cook Place
Bridgman, MI 49106



October 13, 1997

United States Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Operating Licenses DPR-58
Docket No. 50-315

Document Control Manager:

In accordance with the criteria established by 10 CFR 50.73 entitled Licensee Event Report System, the following report is being submitted:

97-022-00

Sincerely,

A handwritten signature in dark ink, appearing to read "A. A. Blind", is written over the typed name.

A. A. Blind
Site Vice President

/mbd

Attachment

c: A. B. Beach, Region III
E. E. Fitzpatrick
P. A. Barrett
S. J. Brewer
J. R. Padgett
D. Hahn
Records Center, INPO
NRC Resident Inspector

IE221

9710240049 971013
PDR ADDCK 05000315
S PDR



LICENSEE EVENT REPORT (LER)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)
Donald C. Cook Nuclear Plant - Unit 1DOCKET NUMBER (2)
50-315

Page 1 of 1

TITLE (4)

Interim LER - Failure to Comply with USAS B31.1 Power Piping Code Results in Condition That Could Have Prevented the Fulfillment of a Safety Function of a System

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
09	11	'97	97	022	00	10	13	97	Cook Unit 2	50-316
OPERATING MODE (9)			THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 50.73(a)(2)(v)(C) (Check one or more) (11)							
5			20.2201(b)			20.2203(a)(3)(i)			50.73(a)(2)(iii)	
POWER LEVEL (10)			20.2203(a)(1)			20.2203(a)(3)(ii)			50.73(a)(2)(iv)	
0			20.2203(a)(2)(i)			20.2203(a)(4)			X 50.73(a)(2)(v)	
			20.2203(a)(2)(ii)			50.36(c)(1)			50.73(a)(2)(vii)	
			20.2203(a)(2)(iii)			50.36(c)(2)			50.73(a)(2)(viii)(A)	
			20.2203(a)(2)(iv)			50.73(a)(2)(i)			50.73(a)(2)(viii)(B)	
			20.2203(a)(2)(v)			50.73(a)(2)(ii)			50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME
Mr. Frank Pisarsky, Mechanical Component Engineering SupervisorTELEPHONE NUMBER (Include Area Code)
616/465-5901, x2607

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRPDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRPDS

SUPPLEMENTAL REPORT EXPECTED (14)

X YES	NO	EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
			10	31	97

ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16)

On September 11, 1997, with Units 1 and 2 in cold shutdown, it was determined that conditions had not been properly established in the Component Cooling Water (CCW) system to meet plant piping design code requirements. It was determined that this event was reportable under 10 CFR 50.72(b)(2)(iii)(C) and 10 CFR 50.72(b)(2)(iii)(D), as a condition that alone could have prevented the fulfillment of the safety function of a system needed to control the release of radioactive material and mitigate the consequences of an accident. An ENS notification was made at 1956 hours on the same day. This interim LER is being submitted under 10 CFR 50.73(a)(2)(v)(C) and 10 CFR 50.73(a)(2)(v)(D).

Chapter 9.5 of the Donald C. Cook Final Safety Analysis Report states: "The relief valve on the component [cooling water] surge tank is sized to relieve the maximum flow rate of water that would enter the surge tank following a rupture of a reactor coolant thermal barrier cooling coil. The set pressure assures that the design pressure of the component cooling system is not exceeded." The piping design code at the Cook Plant is USAS B31.1 power piping code, 1967 edition. Code requirement B31.1 states that an intercepting stop valve cannot be located between the source of pressure and the pressure relief device credited for protecting the pipe. In this instance, the pressure source is the thermal barrier cooling coil; the pressure relief device is the safety relief valve on the CCW surge tank. Contrary to the code requirement there were manual valves between the two devices that were not controlled in accordance with or exempted from USAS B31.1, 1967 edition. An update to this LER is scheduled to be issued by October 31, 1997, pending final review of the event. In the interim, the intervening valves have been physically sealed open and placed under administrative control.

