

CATEGORY 1

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 AUTH. NAME AUTHOR AFFILIATION
 KINGSBEE, J. Indiana Michigan Power Co.
 BLIND, A. A. Indiana Michigan Power Co.
 RECIP. NAME RECIPIENT AFFILIATION

SUBJECT: LER 97-014-00: on 970829, interim LER - potential for operation
 in unanalyzed condition was determined. Caused by postulated
 elevated control room temperatures. TS previously revised to
 limit control room temperatures to 95 F investigated.

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Indiana Michigan
Power Company
Cook Nuclear Plant
One Cook Place
Bridgman, MI 49106



September 29, 1997

United States Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Operating Licenses DPR-58
Docket No. 50-315

Document Control Manager:

In accordance with the criteria established by 10 CFR 50.73 entitled Licensee Event Report System, the following report is being submitted:

97-014-00

Sincerely,

A handwritten signature in cursive script, appearing to read "A. A. Blind", is written above the typed name.

A. A. Blind
Site Vice President

/mbd

Attachment

c: A. B. Beach, Region III
E. E. Fitzpatrick
P. A. Barrett
S. J. Brewer
J. R. Padgett
D. Hahn
Records Center, INPO
NRC Resident Inspector

9710030104 970929
PDR ADDCK 05000315
S PDR

Handwritten: Iccoh



LICENSEE EVENT REPORT (LER)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)
Donald C. Cook Nuclear Plant - Unit 1DOCKET NUMBER (2)
50-315

Page 1 of 1

TITLE (4)

Interim LER - Potential for Operation in Unanalyzed Condition Due to Postulated Elevated Control Room Temperatures

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
08	29	97	97	014	00	09	29	97	Cook, Unit 2	50-316
									FACILITY NAME	DOCKET NUMBER

OPERATING MODE (9)	1	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 50.72(b)(2)(ii)(A) (Check one or more) (11)			
POWER LEVEL (10)	100	20.2201(b)	20.2203(a)(3)(i)	50.73(a)(2)(iii)	73.71(b)
		20.2203(a)(1)	20.2203(a)(3)(ii)	50.73(a)(2)(iv)	73.71(c)
		20.2203(a)(2)(i)	20.2203(a)(4)	50.73(a)(2)(v)	OTHER
		20.2203(a)(2)(ii)	50.36(c)(1)	50.73(a)(2)(vii)	(Specify in Abstract below and in Text, NRC Form 366A)
		20.2203(a)(2)(iii)	50.36(c)(2)	50.73(a)(2)(viii)(A)	
		20.2203(a)(2)(iv)	50.73(a)(2)(i)	50.73(a)(2)(viii)(B)	
	20.2203(a)(2)(v)	X 50.73(a)(2)(ii)	50.73(a)(2)(x)		

LICENSEE CONTACT FOR THIS LER (12)

NAME
Mr. Jeb Kingseed, Nuclear Safety and Analysis ManagerTELEPHONE NUMBER (Include Area Code)
616/697-5106

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS

SUPPLEMENTAL REPORT EXPECTED (14)

<input checked="" type="checkbox"/> YES	<input type="checkbox"/> NO	EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
			11	17	97

ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16)

On August 29, 1997, with Units 1 and 2 at 100% Rated Thermal Power, it was determined that both units had operated in an unanalyzed condition due to the potential effects of elevated temperatures on Control Room instrumentation. It was determined that this was reportable under 10CFR50.72(b)(2)(ii)(A) as an unanalyzed condition, and an ENS notification was made at 1412 hours on August 29, 1997. This interim LER is being submitted under 10CFR50.72(a)(2)(ii) as an unanalyzed condition.

It was determined that in August of 1988, during a 22 day period when Lake Michigan water temperature averaged 81 degrees Fahrenheit, the temperature inside the Control Room could have reached 110.4 degrees Fahrenheit had the non-safety grade chillers not functioned. As a result of the elevated Control Room temperature, the lifetime for some of the instrumentation in the Control Room would have been shortened to 150 hours, or 6.25 days. At the time the plant's Technical Specifications allowed continuous operation with Control Room temperature up to 120 degrees Fahrenheit, and the impact of this shorted instrument lifespan on plant operation was not evaluated. Operation of the plant during this time period is considered operation in an unanalyzed condition.

The Technical Specification were previously revised to limit Control Room temperature to 95 degrees Fahrenheit. Investigation into this condition continues. Evaluation of the safety consequences of this condition will be completed and an update to this interim LER will be issued by November 17, 1997.