

CATEGORY

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ACCESSION NBR: 9709260283 DOC. DATE: 97/09/22 NOTARIZED: NO DOCKET #
 FACIL: 50-316 Donald C. Cook Nuclear Power Plant, Unit 2, Indiana M 05000316
 AUTH. NAME AUTHOR AFFILIATION
 KINGSEED, J. Indiana Michigan Power Co.
 BLIND, A.A. Indiana Michigan Power Co.
 RECIP. NAME RECIPIENT AFFILIATION

SUBJECT: LER 97-004-00: on 970826, interim LER - change to CCW temperature w/o revision to FSAR results in condition outside design basis, was determined. Investigation continuing. Evaluation of SE ongoing. W/970922 ltr.

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Indiana Michigan
Power Company
Cook Nuclear Plant
One Cook Place
Bridgman, MI 49106



September 22, 1997

United States Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Operating Licenses DPR-74
Docket No. 50-316

Document Control Manager:

In accordance with the criteria established by 10 CFR 50.73 entitled Licensee Event Report System, the following report is being submitted:

97-004-00

Sincerely,

A handwritten signature in dark ink, appearing to read 'A. A. Blind'.

A. A. Blind
Site Vice President

/mbd

Attachment

c: A. B. Beach, Region III
E. E. Fitzpatrick
P. A. Barrett
S. J. Brewer
J. R. Padgett
D. Hahn
Records Center, INPO
NRC Resident Inspector

9709260283 970922
PDR ADOCK 05000316
S PDR

IE221



LICENSEE EVENT REPORT (LER)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNBB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)
Donald C. Cook Nuclear Plant - Unit 2DOCKET NUMBER (2)
50-316

Page 1 of 1

TITLE (4)

Interim LER - Change to Component Cooling Water Temperature Without Revision to FSAR Results in Condition Outside Design Basis

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
08	26	97	97	-- 004 --	00	09	22	97	None	
OPERATING MODE (9)			THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 50.73(a)(2)(iii) (Check one or more) (11)							
1			20.2201(b)			20.2203(a)(3)(i)			50.73(a)(2)(iii)	
POWER LEVEL (10)			20.2203(a)(1)			20.2203(a)(3)(ii)			50.73(a)(2)(iv)	
100			20.2203(a)(2)(i)			20.2203(a)(4)			50.73(a)(2)(v)	
			20.2203(a)(2)(ii)			50.36(c)(1)			50.73(a)(2)(vii)	
			20.2203(a)(2)(iii)			50.36(c)(2)			50.73(a)(2)(viii)(A)	
			20.2203(a)(2)(iv)			50.73(a)(2)(i)			50.73(a)(2)(viii)(B)	
			20.2203(a)(2)(v)			X 50.73(a)(2)(ii)			50.73(a)(2)(x)	
(Specify in Abstract below and in Text, NRC Form 366A)										

LICENSEE CONTACT FOR THIS LER (12)

NAME

TELEPHONE NUMBER (Include Area Code)

Mr. Jeb Kingseed, Nuclear Safety and Analysis Manager

616/697-5106

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS

SUPPLEMENTAL REPORT EXPECTED (14)

X	YES	NO	EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
				11	17	97

ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16)

On August 26, 1997, with Unit 2 at 100 percent Rated Thermal Power, it was determined that the unit had operated outside the design basis during the Unit 2 1996 refueling outage. It was determined that this event was reportable under 10CFR50.72(b)(1)(ii)(B) as a condition outside the design basis, and an ENS notification was made at 1553 on August 26, 1997. This interim LER is being submitted under 10CFR50.73(a)(2)(ii).

Per the FSAR, the design of the Spent Fuel Pool (SFP) heat exchanger is based on a maximum Component Cooling Water (CCW) inlet temperature of 95 degrees Fahrenheit. A safety evaluation was performed for the Unit 2 1996 refueling outage full core offload which was based on a maximum CCW inlet temperature of 90 degrees Fahrenheit to the heat exchanger. As a result of this evaluation, an administrative limit of 90 degrees Fahrenheit was imposed on both units without a safety evaluation being performed to revise the FSAR value.

Investigation into this event is continuing. In the interim, information concerning this condition has been made available to Engineering personnel. Evaluation of the safety consequences of this condition, and other similar conditions reported separately via ENS during the period August 8, 1997 to September 12, 1997, is ongoing. An update to this interim LER will be issued by November 17, 1997.