

ATTACHMENT 2 TO AEP:NRC:1028B

EXISTING TECHNICAL SPECIFICATION PAGES
MARKED TO REFLECT PROPOSED CHANGES

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REACTIVITY CONTROL SYSTEMS

SURVEILLANCE REQUIREMENTS

4.1.1.4 The MTC shall be determined to be within its limits during each fuel cycle as follows: .

- a) The MTC shall be measured and compared to the BOL limit specified in the COLR prior to initial operation above 5% of RATED THERMAL POWER, after each fuel loading.
- b) The MTC shall be measured at any THERMAL POWER within 7 EFPD after reaching an equilibrium boron concentration of 300 ppm. The measured value shall be compared to the 300 ppm surveillance limit specified in the COLR. In the event this comparison indicates that the MTC will be more negative than the EOL limit, the MTC shall be remeasured at least once per 14 EFPD during the remainder of the fuel cycle and the MTC value compared to the EOL limit.

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6.0 ADMINISTRATIVE CONTROLS

MONTHLY REACTOR OPERATING REPORT

- 6.9.1.8 Routine reports of operating statistics and shutdown experience, including documentation of all challenges to the PORVs or safety valves, shall be submitted on a monthly basis to the U.S. Nuclear Regulatory Commission (Attn: Document Control Desk), Washington, D.C. 20555, with a copy to the Regional Office no later than the 15th of each month following the calendar month covered by the report.

CORE OPERATING LIMITS REPORT

- 6.9.1.9.1 Core operating limits shall be established and documented in the CORE OPERATING LIMITS REPORT before each reload cycle or any remaining part of a reload cycle for the following:
- a. Moderator Temperature Coefficient Limits for Specification 3/4.1.1.4,
 - b. Rod Drop Time Limits for Specification 3/4.1.3.3,
 - c. Shutdown Rod Insertion Limits for Specification 3/4.1.3.4,
 - d. Control Rod Insertion Limits for Specification 3/4.1.3.5,
 - e. Axial Flux Difference for Specification 3/4.2.1,
 - f. Heat Flux Hot Channel Factor for Specification 3/4.2.2,
 - g. Nuclear Enthalpy Rise Hot Channel Factor for Specification 3/4.2.3, and
 - h. Allowable Power Level for Specification 3/4.2.6.
- 6.9.1.9.2 The analytical methods used to determine the core operating limits shall be those previously reviewed and approved by the NRC in:
- a. WCAP-9272-P-A, "Westinghouse Reload Safety Evaluation Methodology," July 1985 (Westinghouse Proprietary),
 - b. WCAP-8385, "Power Distribution Control and Load Following Procedures - Topical Report," September 1974 (Westinghouse Proprietary),
 - c. WCAP-10216-P-A, Revision 1A, "Relaxation of Constant Axial Offset Control/ F_Q Surveillance Technical Specification," February 1994 (Westinghouse Proprietary),
 - d. WCAP-10266-P-A Rev. 2, "The 1981 Version of Westinghouse Evaluation Mode Using BASH Code," March 1987 (Westinghouse Proprietary).

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ADMINISTRATIVE CONTROLS

CORE OPERATING LIMITS REPORT (Continued)

6.9.1.9.3 The core operating limits shall be determined so that all applicable limits (e.g., fuel thermal-mechanical limits, core thermal-hydraulic limits, ECCS limits, nuclear limits such as shutdown margin, and transient and accident analysis limits) of the safety analysis are met.

6.9.1.9.4 The CORE OPERATING LIMITS REPORT, including any mid-cycle revisions or supplements thereto, shall be provided upon issuance, for each reload cycle, to the NRC document control desk with copies to the Regional Administrator and Resident Inspector.

SPECIAL REPORTS

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6.9.2 Special reports shall be submitted to the attention of the document control desk - U.S. Nuclear Regulatory Commission (Washington, D.C. 20555), with copies to the Region III Administrator and the Resident Inspector at the Cook Nuclear Plant within the time period specified for each report. These reports shall be submitted covering the activities identified below pursuant to the requirements of the applicable reference specification:

- a. Inoperable Seismic Monitoring Instrumentation, Specification 3.3.3.3.
- b. Seismic Monitoring Instrumentation Actuated, Specification 4.3.3.3.2.
- c. Inoperable Meteorological Monitoring Instrumentation, Specification 3.3.3.4.
- d. High Specific Activity in RCS Coolant, Specification 3.4.8.
- e. RCS Pressure Transient Mitigated By RHR Safety Valve or RCS Vent(s), Specification 3.4.9.3.
- f. Moderator Temperature Coefficient, Specification 3.1.1.4.
- g. Sealed Source Leakage in Excess of Limits, Specification 4.7.7.1.3.
- h. ECCS Actuation, Specifications 3.5.2 and 3.5.3.
- i. Violation of Safety Limit, Specification 6.7.1.

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- * Measurement of the MTC in accordance with 4.1.1.4.b may be suspended provided the benchmark criteria and the revised prediction as documented in the COLR are satisfied. Data required for the calculation of the revised prediction are provided in the MOST NEGATIVE MODERATOR TEMPERATURE COEFFICIENT LIMIT REPORT, per specification 6.9.1.9.5.

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- e. WCAP-13749-P-A, "Safety Evaluation Supporting The Conditional Exemption of the Most Negative EOL Moderator Temperature Coefficient Measurement," March, 1997 (Westinghouse Proprietary).

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- 6.9.1.9.5 The MOST NEGATIVE MODERATOR TEMPERATURE COEFFICIENT LIMIT REPORT shall be prepared prior to each cycle startup and maintained on file. This report will have data to be used for the determination of the revised prediction of the 300 ppm/ARO/RTP MTC, per WCAP-13749-P-A.

REACTIVITY CONTROL SYSTEMS

SURVEILLANCE REQUIREMENTS

4.1.1.4 The MTC shall be determined to be within its limits during each fuel cycle as follows:

- a) The MTC shall be measured and compared to the BOL limit specified in the COLR prior to initial operation above 5% of RATED THERMAL POWER, after each fuel loading.
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6.0 ADMINISTRATIVE CONTROLS

MONTHLY REACTOR OPERATING REPORT

- 6.9.1.8 Routine reports of operating statistics and shutdown experience, including documentation of all challenges to the PORVs or safety valves, shall be submitted on a monthly basis to the U.S. Nuclear Regulatory Commission (Attn: Document Control Desk), Washington, D.C. 20555, with a copy to the Regional Office no later than the 15th of each month following the calendar month covered by the report.

CORE OPERATING LIMITS REPORT

- 6.9.1.9.1 Core operating limits shall be established and documented in the CORE OPERATING LIMITS REPORT before each reload cycle or any remaining part of a reload cycle for the following:
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 - b. Rod Drop Time Limits for Specification 3/4.1.3.4,
 - c. Shutdown Rod Insertion Limits for Specification 3/4.1.3.5,
 - d. Control Rod Insertion Limits for Specification 3/4.1.3.6,
 - e. Axial Flux Difference for Specification 3/4.2.1,
 - f. Heat Flux Hot Channel Factor for Specification 3/4.2.2,
 - g. Nuclear Enthalpy Rise Hot Channel Factor for Specification 3/4.2.3, and
 - h. Allowable Power Level for Specification 3/4.2.6.
- 6.9.1.9.2 The analytical methods used to determine the core operating limits shall be those previously reviewed and approved by the NRC in:
- a. WCAP-9272-P-A, "Westinghouse Reload Safety Evaluation Methodology," July 1985 (Westinghouse Proprietary),
 - b. WCAP-8385, "Power Distribution Control and Load Following Procedures - Topical Report," September 1974 (Westinghouse Proprietary),
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ADMINISTRATIVE CONTROLS

CORE OPERATING LIMITS REPORT (Continued)

6.9.1.9.3 The core operating limits shall be determined so that all applicable limits (e.g., fuel thermal-mechanical limits, core thermal-hydraulic limits, ECCS limits, nuclear limits such as shutdown margin, and transient and accident analysis limits) of the safety analysis are met.

6.9.1.9.4 The CORE OPERATING LIMITS REPORT, including any mid-cycle revisions or supplements thereto, shall be provided upon issuance, for each reload cycle, to the NRC document control desk with copies to the Regional Administrator and Resident Inspector.

SPECIAL REPORTS

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6.9.2 Special reports shall be submitted to the attention of the document control desk - U.S. Nuclear Regulatory Commission (Washington, D.C. 20555), with copies to the Region III Administrator and the Resident Inspector at the Cook Nuclear Plant within the time period specified for each report. These reports shall be submitted covering the activities identified below pursuant to the requirements of the applicable reference specification:

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- c. Inoperable Meteorological Monitoring Instrumentation, Specification 3.3.3.4.
- d. High Specific Activity in RCS Coolant, Specification 3.4.8.
- e. RCS Pressure Transient Mitigated By RHR Safety Valve or RCS Vent(s), Specification 3.4.9.3.
- f. Moderator Temperature Coefficient, Specification 3.1.1.4.
- g. Sealed Source Leakage in Excess of Limits, Specification 4.7.7.1.3.
- h. ECCS Actuation, Specifications 3.5.2 and 3.5.3.
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- * Measurement of the MTC in accordance with 4.1.1.4.b may be suspended provided the benchmark criteria and the revised prediction as documented in the COLR are satisfied. Data required for the calculation of the revised prediction are provided in the MOST NEGATIVE MODERATOR TEMPERATURE COEFFICIENT LIMIT REPORT, per specification 6.9.1.9.5.

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- e. WCAP-13749-P-A, "Safety Evaluation Supporting The Conditional Exemption of the Most Negative EOL Moderator Temperature Coefficient Measurement," March, 1997 (Westinghouse Proprietary).

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- 6.9.1.9.5 The MOST NEGATIVE MODERATOR TEMPERATURE COEFFICIENT LIMIT REPORT shall be prepared prior to each cycle startup and maintained on file. This report will have data to be used for the determination of the revised prediction of the 300 ppm/ARO/RTP MTC, per WCAP-13749-P-A.

ATTACHMENT 3 TO AEP:NRC:1028B
PROPOSED TECHNICAL SPECIFICATION PAGES

3/4 LIMITING CONDITIONS FOR OPERATION AND SURVEILLANCE REQUIREMENTS
3/4.1 REACTIVITY CONTROL SYSTEMS

SURVEILLANCE REQUIREMENTS

- 4.1.1.4 The MTC shall be determined to be within its limits during each fuel cycle as follows:
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6.0 ADMINISTRATIVE CONTROLS

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