

INDIANA MICHIGAN POWER COMPANY
DONALD C. COOK NUCLEAR PLANT, BRIDGMAN MI.
UNIT 1 COMMERCIAL SERVICE DATE 8-23-1975
NATIONAL BOARD NUMBER 20761

Report of Inservice Inspections
From October 26, 1995 to April 27, 1997

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Section 3	NIS-2 Repair and Replacement Reports

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From October 26, 1995 to April 27, 1997



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INTRODUCTION

This report summarizes the Inservice Inspection activities performed at the Cook Nuclear Plant, Unit 1, for the period of October 26, 1995 through April 27, 1997. These activities were performed in accordance with the requirements of the ASME Boiler and Pressure Vessel Code, Section XI, Rules for Inservice Inspection of Nuclear Power Plant Components. The second ten-year interval applicable code edition was 1983 with the 1983 summer addenda up until July 1, 1996. The third ten-year inspection interval began at that time and the applicable code for ISI activities then became the 1989 edition.

The purpose of this report is to satisfy the requirements of IWA-6000 for the submittal of the Inservice Inspection Summary Report at the completion of each refueling outage. Examination and tests conducted since the preceding summary report (October 25, 1995) are included in this report. Repairs and replacements are also included in this report per the requirements of the 1989 Edition of ASME Section XI IWA-6220(d)3. Also, contained within this report are the supplemental information requirements of IWA-6220.

Contents of this report have been reviewed and verified correct with supporting documentation available at the plant for review if required.

FORM NIS-1 OWNERS' DATA REPORT FOR INSERVICE INSPECTION

As required by the Provisions of the ASME Code Rules

1. Owner Indiana Michigan Power Company, One Summit Dr., Ft. Wayne, IN 46802
(Name and Address of Owner)
2. Plant Donald C. Cook Nuclear Plant, One Cook Place, Bridgman, MI 49106
(Name and Address of Plant)
3. Plant Unit One 4. Owner Certification of Authorization (if required) N/A
5. Commercial Service Date 8/23/75 6. National Board Number for Unit 20761 (Mich. M-09672-M)

7. Components Inspected

Component or Appurtenance	Manufacturer or Installer	Manufacturer or Installer Serial No.	State or Province No.	National Board No.
Reactor Vessel	Combustion Engineering	CE 66105	M 96762 M	20761
Pressurizer	Westinghouse	1131	M 96763 M	6849
Steam Generator 1-1	Westinghouse	1121	M 96764 M	6831
Steam Generator 1-2	Westinghouse	1122	M 96764 M	6832
Steam Generator 1-3	Westinghouse	1123	M 96765 M	6833
Steam Generator 1-4	Westinghouse	1124	M 96767 M	6834
Regenerative Heat Exchanger	Westinghouse	1831-6	-	426, 427, 428
Volume Control Tank	Joseph Oats & Sons, Inc.	1787-3A	-	375
Boron Injection Tank	Struthers-Wells Corp.	Spin No. AEP-SIATB1-01	-	-
E. Containment Spray Ht. Exchanger	Yuba	69-G-226-1A	-	-
E. Residual Heat Removal Ht. Exchanger	Engineers & Fabricators, Inc.	S-15586-A	-	-

Note: Supplemental sheets in form of lists, sketches, or drawings may be used provided (1) size is 8.5 in. x 11 in. (2) information in items 1 through 6 on this data report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-1 (back)

8. Examination Dates 10/26/95 to 4/27/97

9. Inspection Period Identification: First

10. Inspection Interval Identification: Third

11. Applicable Edition of Section XI 1989 Addenda N/A

12. Date/Revision of Inspection Plan: 11/95 Rev. 0

13. Abstract of Examinations. Include a list of examinations and tests and a statement concerning status of work required for the Inspection Plan.

See Attached

14. Abstract of Results of Examinations and Tests.

See Attached

15. Abstract of Corrective Measures.

See Attached

We certify that a) the statements made in this report are correct, b) the examinations and tests meet the Inspection Plan as required by the ASME Code, Section XI, and c) corrective measures taken conform to the rules of the ASME Code, Section XI.

Certificate of Authorization No. (if applicable) N/A Expiration Date N/A
Date 7-17 1997 Signed Indiana Michigan Power Co. By Frank [Signature]
Owner

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Michigan and employed by ARKWRIGHT MVT. INC. of Warren, Ma. have inspected the components described in this Owner's Report during the period 10/26/95 to 4/27/97, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Michael R. Matuszka Commissions Michigan 0055, Endorsements N&I
Inspector's Signature National Board, State, Province, and Endorsements

Date July 24 1997

INDIANA MICHIGAN POWER COMPANY
DONALD C. COOK NUCLEAR PLANT, BRIDGMAN MI.
UNIT 1 COMMERCIAL SERVICE DATE 8-23-1975
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EXAMINATION OF CLASS 1 AND CLASS 2 COMPONENTS

Inservice examinations were performed during the period of October 26, 1997 through April 27, 1997. All examinations were in accordance with ASME Section XI, 1989 Edition. Examinations were scheduled and performed according to the Third Ten-Year Long Term Inservice Examination and Testing Plan for Class 1, 2 and 3 Systems and Components for Donald C. Cook Nuclear Plant, Unit 1.

Nondestructive examinations were performed by Framatome Technologies (FTI) and American Electric Power (AEP) personnel in accordance with procedures which were approved for use at the Cook Nuclear Plant. FTI performed the required volumetric ultrasonic examinations and surface examination if one was needed in addition to the volumetric examination. AEP personnel performed all other surface and visual examinations.

Snubber examinations and functional testing were performed in conjunction with maintenance procedures for technical specification surveillance of snubbers.

Due to the discrepant bolts found during the 1995 Unit refueling outage, a special remote visual examination and mechanical agitation were performed on 86 core barrel/former plate bolts. The bolts examined were located behind the thermal shield on the top level former plate and on the next lower former plate in the vicinity of the discrepant bolts. The examination was performed by Master-Lee in conjunction with American Electric Power personnel using a specially designed tool due to very limited clearance between the core barrel wall and the thermal shield. All examined bolting were found to be acceptable with no signs of looseness or degradation.

AEP personnel performed VT-3 visual examinations on ISI component supports. Design sketch nonconformances and other discrepancies found were evaluated by AEPSC Nuclear Engineering. All discrepant pipe supports and associated systems met operability limits.

The summary pages in this section lists the examinations, tests performed and results. In the following summary pages, "NOREC" is no recordable indications, "GEOM" is geometric indications, and "OTHER" are indications other than geometric indications.

INDIANA MICHIGAN POWER COMPANY, DONALD C. COOK PLANT, BRIDGMAN, MI.
UNIT 1 COMMERCIAL SERVICE DATE 8-23-75, NATIONAL BOARD NUMBER 20761

DATE: 07/16/97
REVISION: 0

COOK NUCLEAR PLANT UNIT 1
INSERVICE INSPECTION SUMMARY
THIRD INTERVAL, FIRST PERIOD, FIRST OUTAGE (97RP)
CLASS 1 COMPLETED COMPONENTS

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FOR PRESSURE VESSEL AND CLOSURE HEAD

SUMMARY EXAMINATION AREA		ASME	SEC. XI	CATGY	EXAM	ITEM NO	METHOD	PROCEDURE	N	O	REMARKS
NUMBER	IDENTIFICATION								C	M	R
											CALIBRATION BLOCK

CLOSURE HEAD MERIDIONAL WELDS (FIG NO A-2)

001400	1-M-01	B-A	UT0	54-ISI-130-32	X	-	-	-	FTI - EXAMINED ACCESSIBLE LENGTH OF
	MERIDIONAL WELD AT 0 DEG.	B1.22	UT35		X	-	-	-	WELD. CATEGORY B-A REQUIRES THAT THE
			UT35T		X	-	-	-	EXTENT OF EXAMINATION BE THE ACCESSIBLE
			UT45		X	-	-	-	LENGTH OF THE WELD.
			UT45T		X	-	-	-	**RV-2**
			UT60		X	-	-	-	
			UT60T		X	-	-	-	
001900	1-M-06	B-A	UT0	54-ISI-130-32	X	-	-	-	FTI - EXAMINED ACCESSIBLE LENGTH OF
	MERIDIONAL WELD AT 300 DEG.	B1.22	UT35		X	-	-	-	WELD. CATEGORY B-A REQUIRES THAT THE
			UT35T		X	-	-	-	EXTENT OF EXAMINATION BE THE ACCESSIBLE
			UT45		X	-	-	-	LENGTH OF THE WELD.
			UT45T		X	-	-	-	**RV-2**
			UT60		X	-	-	-	
			UT60T		X	-	-	-	

CLOSURE HEAD TO FLANGE WELD (FIG NO A-2)

002700	1-C-01	B-A	MT	54-ISI-270-35	X	-	-	-	FTI - EXAMINED 33% OF WELD. FULL UT
	HEAD TO FLANGE	B1.40	UT0	54-ISI-130-32	X	-	-	-	COVERAGE OBTAINED WITH USE OF 35 DEG.
			UT35		X	-	-	-	TRANSDUCER.
			UT35T		X	-	-	-	
			UT45		X	-	-	-	**RV-2**
			UT45T		X	-	-	-	
			UT60		X	-	-	-	
			UT60T		X	-	-	-	

INDIANA MICHIGAN POWER COMPANY, DONALD C. COOK PLANT, BRIDGMAN, MI.
UNIT 1 COMMERCIAL SERVICE DATE 8-23-75, NATIONAL BOARD NUMBER 20761

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COOK NUCLEAR PLANT UNIT 1
INSERVICE INSPECTION SUMMARY
THIRD INTERVAL, FIRST PERIOD, FIRST OUTAGE (97RF)
CLASS 1 COMPLETED COMPONENTS

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SURIZER

SUMMARY	EXAMINATION AREA	ASME SEC. XI CATGY	EXAM METHOD	PROCEDURE	N O G T R E H E O E C M R	REMARKS
NUMBER	IDENTIFICATION	ITEM NO	METHOD			**CALIBRATION BLOCK**
<u>LONGITUDINAL WELDS (FIG NO A-4)</u>						
009500	1-PRZ-16 LONGITUDINAL SEAM AT +23 DEG.	B-B B2.12	UT0 UT45 UT45T UT60 UT60T	54-ISI-130-32	X - - X - - X - - X - - X - -	- FTI **RV-3**
<u>NOZZLE INSIDE RADIUS SECTIONS (FIG NO A-4)</u>						
010100	6"-1-RC-6-IRS RELIEF NOZZLE	B-D B3.120	UT20	54-ISI-132-0	X - - - - - - - -	- FTI **IR-CSCL-24-DCC**
010300	6"-1-RC-8-IRS RELIEF NOZZLE	B-D B3.120	UT20	54-ISI-132-0	X - - - - - - - -	- FTI **IR-CSCL-24-DCC**
010400	6"-1-RC-9-IRS RELIEF NOZZLE	B-D B3.120	UT20	54-ISI-132-0	X - - - - - - - -	- FTI **IR-CSCL-24-DCC**
<u>NOZZLE TO SAFE-END AND SAFE-END TO NOZZLE WELDS (FIG NO A-4)</u>						
010900	1-PRZ-20 NOZZLE TO SAFE-END	B-F B5.40	PT UT45 UT45T	54-ISI-240-36 54-ISI-121-1	X - - - X - X - -	- FTI **8-SS-X-1.4-11-DCC**
011200	1-PRZ-23 NOZZLE TO SAFE-END	B-F B5.40	PT UT45 UT45T	54-ISI-240-36 54-ISI-121-1	X - - - X - X - -	- FTI **8-SS-X-1.4-11-DCC**

INDIANA MICHIGAN POWER COMPANY, DONALD C. COOK PLANT, BRIDGMAN, MI.
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COOK NUCLEAR PLANT UNIT 1
INSERVICE INSPECTION SUMMARY
THIRD INTERVAL, FIRST PERIOD, FIRST OUTAGE (97RF)
CLASS 1 COMPLETED COMPONENTS

TEAM GENERATOR NO. 11

				N	O	
				O	G	T
				R	E	H
SUMMARY EXAMINATION AREA				E	O	E
REMARKS				C	M	R
CALIBRATION BLOCK						
NUMBER	IDENTIFICATION	ITEM NO	METHOD	PROCEDURE		

NOZZLE INSIDE RADIUS SECTIONS (FIG NO A-6)

012000	STM-11-I-IRS	B-D	UT35	54-ISI-132-0	X	-	-	-	FTI
	INLET NOZZLE INSIDE RADIUS	B3.140			-	-	-		
	SECTION				-	-	-		
					-	-	-		
									IR-CSCL-24-DCC

012100	STM-11-O-IRS	B-D	UT35	54-ISI-132-0	X	-	-	-	FTI
	OUTLET NOZZLE INSIDE RADIUS	B3.140			-	-	-		
	SECTION				-	-	-		
					-	-	-		
									IR-CSCL-24-DCC

NOZZLE TO ELBOW AND ELBOW TO NOZZLE WELDS (FIG NO A-6)

012200	STM-11-02	B-F	PT	54-ISI-240-36	X	-	-	-	FTI - EXAM LIMITED TO 75% OF WELD AND
	ELBOW TO INLET NOZZLE	B5.70	UT45RL	54-ISI-123-1	X	-	-	-	BASE METAL DUE TO THE NOZZLE
			UT45TRL		X	-	-	-	CONFIGURATION AND ONE-SIDED COVERAGE
					-	-	-	-	WITH THE 45 DEG. REFRACTED LONGITUDINAL
					-	-	-	-	TRANSDUCER.
									3378033 (MAIN RPV COOLANT)

012300	STM-11-03	B-F	PT	54-ISI-240-36	X	-	-	-	FTI - EXAM LIMITED TO 75% OF WELD AND
	OUTLET NOZZLE TO ELBOW	B5.70	UT45RL	54-ISI-123-1	X	-	-	-	BASE METAL DUE TO THE NOZZLE
			UT45TRL		X	-	-	-	CONFIGURATION AND ONE-SIDED COVERAGE
					-	-	-	-	WITH THE 45 DEG. REFRACTED LONGITUDINAL
					-	-	-	-	TRANSDUCER.
									3378033 (MAIN RPV COOLANT)

PRESSURE RETAINING BOLTING (2" AND LESS) (FIG NO A-6)

012740	STM-11-IMB-1 THROUGH 16	B-G-2	VT-1	12QHP5050NDE006	X	-	-	-	C35793-05
	INLET MANWAY BOLTING	B7.30	MT	12QHP5050NDE002	X	-	-	-	
					-	-	-	-	

012760	STM-11-OMB-1 THROUGH 16	B-G-2	VT-1	12QHP5050NDE006	X	-	-	-	J.O. C35793-05
	OUTLET MANWAY BOLTING	B7.30	MT	12QHP5050NDE002	X	-	-		
					-	-	-		



INDIANA MICHIGAN POWER COMPANY, DONALD C. COOK PLANT, BRIDGMAN, MI.
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COOK NUCLEAR PLANT UNIT 1

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INSERVICE INSPECTION SUMMARY

THIRD INTERVAL, FIRST PERIOD, FIRST OUTAGE (97RF)

CLASS 1 COMPLETED COMPONENTS

REACTOR COOLANT SYSTEM

SUMMARY	EXAMINATION AREA	ASME SEC. XI CATGY	EXAM		N O O G T R E H E O E	REMARKS
NUMBER	IDENTIFICATION	ITEM NO	METHOD	PROCEDURE	C M R	**CALIBRATION BLOCK**
<u>LINE 1-RC-1 (FIG NO A-9)</u>						
021800	1-RC-1-10N 2-IN. NOZZLE (BRANCH CONNECTION)	B-J B9.32	PT	12QHP5050NDE001	X - - - - - - - - - - -	- J.O. C35884-06
022000	1-RC-1-12N 2-1/2IN. NOZZLE (BRANCH CONNECTION)	B-J B9.32	PT	12QHP5050NDE001	X - - - - - - - - - - -	- J.O. C35884-06
<u>LINE 1-RC-2 (FIG NO A-11)</u>						
023500	1-RC-2-10S PIPE TO ELBOW	B-J B9.11	PT UT45RL UT45TRL	54-ISI-240-36 54-ISI-123-1	X - - X - - X - -	- FTI **37-CCSS-X-3.0-9-DCC**
023700	1-RC-2-23N 2-IN. NOZZLE (BRANCH CONNECTION)	B-J B9.32	PT	12QHP5050NDE001	X - - - - - - - - - - -	- J.O. C35884-06
<u>LINE 1-RC-3 (FIG NO A-14)</u>						
026500	1-RC-3-39N 2-IN. NOZZLE (BRANCH CONNECTION)	B-J B9.32	PT	12QHP5050NDE001	X - - - - - - - - - - -	- J.O. C35884-06
<u>LINE 1-RC-3 (FIG NO A-15)</u>						
027450	1-RC-3-24F PIPE TO PIPE	B-J B9.21	PT	12QHP5050NDE001	X - - - - - - - -	- J.O. C35884-06 **37-CCSS-X-3.0-9-DCC**

DATE: 07/16/97

REVISION: 0

COOK NUCLEAR PLANT UNIT 1
INSERVICE INSPECTION SUMMARY

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THIRD INTERVAL, FIRST PERIOD, FIRST OUTAGE (97RF)

CLASS 1 COMPLETED COMPONENTS

REACTOR COOLANT SYSTEM

SUMMARY EXAMINATION AREA NUMBER IDENTIFICATION	ASME SEC. XI CATGY EXAM ITEM NO METHOD	PROCEDURE	N O G T R E H E O E C M R				REMARKS
-----	-----	-----	-----	-----	-----	-----	-----

LINE 1-RC-4 (FIG NO A-17)

029300	1-RC-4-26S	B-J	PT	54-ISI-240-36	X	-	-	-	FTI
	PIPE TO ELBOW	B9.11	UT45RL	54-ISI-123-1	X	-	-	-	
			UT45TRL		X	-	-	-	
									37-CCSS-X-3.0-9-DCC

029400	1-RC-4-26SLD-I	B-J	PT	54-ISI-240-36	X	-	-	-	FTI
	LONGITUDINAL SEAM	B9.12	UT45RL	54-ISI-123-1	X	-	-	-	
			UT45TRL		X	-	-	-	
									37-CCSS-X-3.0-9-DCC

029500	1-RC-4-26SLD-O	B-J	PT	54-ISI-240-36	X	-	-	-	FTI
	LONGITUDINAL SEAM	B9.12	UT45RL	54-ISI-123-1	X	-	-	-	
			UT45TRL		X	-	-	-	
									37-CCSS-X-3.0-9-DCC

LINE 1-RC-4 (FIG NO A-18)

030300	1-RC-4-57N	B-J	PT	54-ISI-240-36	X	-	-	-	FTI - LIMITED EXAM COVERAGE OF 75%
	10-IN. NOZZLE (SET-ON BRANCH	B9.31	UT45	54-ISI-121-1	X	-	-	-	DUE TO BRANCH CONNECTION CONFIGURATION.
	CONNECTION)		UT45T		X	-	-	-	
					-	-	-	-	
									BC-10/27.5-SS-43-DCC

LINE 1-RC-5 (FIG NO A-19)

031300	1-RC-5-05F	B-J	PT	54-ISI-240-36	X	-	-	-	FTI
	PIPE TO SAFE-END	B9.11	UT45	54-ISI-121-1	X	-	-	-	
			UT45T		X	-	-	-	
									3378032 (14-SS-160-1.40)

LINE 1-RC-6 (FIG NO A-20)

031400	1-RC-6-01F	B-J	PT	54-ISI-240-36	X	-	-	-	FTI - LIMITED EXAM COVERAGE OF 92%
	SAFE-END TO PIPE	B9.11	UT45	54-ISI-121-1	X	-	-	-	DUE TO SAFE-END CONFIGURATION.
			UT45T		X	-	-	-	
			UT60		X	-	-	-	
									3378028 (6-SS-160-.71)

DATE: 07/16/97
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COOK NUCLEAR PLANT UNIT 1
INSERVICE INSPECTION SUMMARY
THIRD INTERVAL, FIRST PERIOD, FIRST OUTAGE (97RF)
CLASS 1 COMPLETED COMPONENTS

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REACTOR COOLANT SYSTEM

SUMMARY EXAMINATION AREA		ASME			N	O	
		SEC. XI			O	G	T
		CATGY	EXAM		R	E	H
		ITEM NO	METHOD	PROCEDURE	E	O	E
NUMBER	IDENTIFICATION				C	M	R
							REMARKS
							CALIBRATION BLOCK

LINE 1-RC-6 (FIG NO A-21)

032900	1-RC-6-17F	B-J	PT	12QHP5050NDE001	X	-	-	-	J.O. C35884-06
	PIPE TO VALVE	B9.21			-	-	-		

034300	1-RC-6-29F	B-J	PT	12QHP5050NDE001	X	-	-	-	J.O. C35884-06
	PIPE TO VALVE	B9.21			-	-	-		

LINE 1-RC-7 (FIG NO A-23)

035810	1-RC-7-09S-FB	B-G-2	VT-1	12QHP5050NDE006	X	-	-	-	J.O. C35884-06
	FLANGE BOLTING	B7.50			-	-	-		

LINE 1-RC-8 (FIG NO A-24)

036700	1-RC-8-09S	B-J	PT	54-ISI-240-36	X	-	-	-	FTI - LIMITED EXAM COVERAGE OF 92%
	PIPE TO FLANGE	B9.11	UT45	54-ISI-121-1	X	-	-	-	DUE TO SAFE-END CONFIGURATION.
			UT45T		X	-	-		
			UT60		X	-	-		
									3378028 (6-SS-160-.71)

LINE 1-RC-9 (FIG NO A-25)

036900	1-RC-9-01F	B-J	PT	54-ISI-240-36	X	-	-	-	FTI - LIMITED EXAM COVERAGE OF 92%
	SAFE-END TO ELBOW	B9.11	UT45	54-ISI-121-1	X	-	-	-	DUE TO SAFE-END CONFIGURATION.
			UT45T		X	-	-		
			UT60		X	-	-		
									3378028 (6-SS-160-.71)

LINE 1-RC-10 (FIG NO A-26)

038500	1-RC-10-08F	B-J	PT	54-ISI-240-36	X	-	-	-	FTI - LIMITED EXAM COVERAGE OF 94%
	TEE TO PIPE	B9.11	UT45	54-ISI-121-1	X	-	-	-	DUE TO TEE FITTING CONFIGURATION.
			UT45T		X	-	-		
									3378027 (4-SS-120-.430)

INDIANA MICHIGAN POWER COMPANY, DONALD C. COOK PLANT, BRIDGMAN, MI.
UNIT 1 COMMERCIAL SERVICE DATE 8-23-75, NATIONAL BOARD NUMBER 20761

DATE: 07/16/97
REVISION: 0

COOK NUCLEAR PLANT UNIT 1
INSERVICE INSPECTION SUMMARY
THIRD INTERVAL, FIRST PERIOD, FIRST OUTAGE (97RF)
CLASS 1 COMPLETED COMPONENTS

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REACTOR COOLANT SYSTEM

SUMMARY EXAMINATION AREA		ASME			N	O		
		SEC. XI			O	G	T	
		CATGY	EXAM		R	E	H	
NUMBER	IDENTIFICATION	ITEM NO	METHOD	PROCEDURE	E	O	E	REMARKS
					C	M	R	**CALIBRATION BLOCK**

<u>LINE 1-RC-10 (FIG NO A-26)</u>								
038700	1-RC-10-10S	B-J	PT	54-ISI-240-36	X	-	-	- FTI
	PIPE TO ELBOW	B9.11	UT45	54-ISI-121-1	X	-	-	
			UT45T		X	-	-	
								3378027 (4-SS-120-.430)
039100	1-RC-10-14S	B-J	PT	54-ISI-240-36	X	-	-	- FTI
	PIPE TO ELBOW	B9.11	UT45	54-ISI-121-1	X	-	-	
			UT45T		X	-	-	
								3378027 (4-SS-120-.430)
<u>LINE 1-RC-10 (FIG NO A-27)</u>								
042700	1-RC-10-38S	B-J	PT	54-ISI-240-36	X	-	-	- FTI
	PIPE TO ELBOW	B9.11	UT45	54-ISI-121-1	X	-	-	
			UT45T		X	-	-	
								3378027 (4-SS-120-.430)
<u>LINE 1-RC-11 (FIG NO A-28)</u>								
043600	1-RC-11-08S	B-J	PT	54-ISI-240-36	X	-	-	- FTI
	ELBOW TO PIPE	B9.11	UT45	54-ISI-121-1	X	-	-	
			UT45T		X	-	-	
								3378027 (4-SS-120-.430)
<u>LINE 1-RC-12 (FIG NO A-29)</u>								
045100	1-RC-12-02S	B-J	PT	54-ISI-240-36	X	-	-	- FTI
	ELBOW TO PIPE	B9.11	UT45	54-ISI-121-1	X	-	-	
			UT45T		X	-	-	
								3378027 (4-SS-120-.430)
046700	1-RC-12-22S	B-J	PT	54-ISI-240-36	X	-	-	- FTI
	PIPE TO ELBOW	B9.11	UT45	54-ISI-121-1	X	-	-	
			UT45T		X	-	-	
								3378027 (4-SS-120-.430)

INDIANA MICHIGAN POWER COMPANY, DONALD C. COOK PLANT, BRIDGMAN, MI.
UNIT 1 COMMERCIAL SERVICE DATE 8-23-75. NATIONAL BOARD NUMBER 20761

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COOK NUCLEAR PLANT UNIT 1
INSERVICE INSPECTION SUMMARY
THIRD INTERVAL, FIRST PERIOD, FIRST OUTAGE (97RF)
CLASS 1 COMPLETED COMPONENTS

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REACTOR COOLANT SYSTEM

SUMMARY EXAMINATION AREA					N	O	REMARKS	
NUMBER	IDENTIFICATION	ASME SEC. XI CATGY	EXAM ITEM NO	METHOD	PROCEDURE	O	G	T
						R	E	H
						E	O	E
						C	M	R
						CALIBRATION BLOCK		
<u>LINE 1-RC-13 (FIG NO A-30)</u>								
048400	1-RC-13-12F PIPE TO BRANCH CONNECTION	B-J B9.21	PT	12QHP5050NDE001	X - -	-	J.O.	C35884-06
						-	-	-
						-	-	-
<u>LINE 1-RC-13 (FIG NO A-31)</u>								
049500	1-RC-13-21S PIPE TO TEE	B-J B9.21	PT	12QHP5050NDE001	X - -	-	J.O.	C35884-06
						-	-	-
						-	-	-
<u>LINE 1-RC-14 (FIG NO A-32)</u>								
050600	1-RC-14-06F VALVE TO PIPE	B-J B9.21	PT	12QHP5050NDE006	X - -	-	J.O.	C35884-06
						-	-	-
						-	-	-
<u>LINE 1-RC-14 (FIG NO A-33)</u>								
052200	1-RC-14-19S PIPE TO ELBOW	B-J B9.21	PT	12QHP5050NDE001	X - -	-	J.O.	C35884-06
						-	-	-
						-	-	-
<u>LINE 1-RC-15 (FIG NO A-35)</u>								
055300	1-RC-15-19S PIPE TO ELBOW	B-J B9.21	PT	12QHP5050NDE001	X - -	-	J.O.	C35884-06
						-	-	-
						-	-	-
055800	1-RC-15-24S ELBOW TO PIPE	B-J B9.21	PT	12QHP5050NDE001	X - -	-	J.O.	C35884-06
						-	-	-
						-	-	-
<u>LINE 1-RC-16 (FIG NO A-36)</u>								
056100	1-RC-16-01S REDUCER TO PIPE	B-J B9.21	PT	12QHP5050NDE001	X - -	-	J.O.	C35884-06
						-	-	-
						-	-	-

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COOK NUCLEAR PLANT UNIT 1

INSERVICE INSPECTION SUMMARY

THIRD INTERVAL, FIRST PERIOD, FIRST OUTAGE (97RF)

CLASS 1 COMPLETED COMPONENTS

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REACTOR COOLANT SYSTEM

SUMMARY EXAMINATION AREA		ASME			N	O	
		SEC. XI			O	G	T
		CATGY	EXAM		R	E	H
		ITEM NO	METHOD	PROCEDURE	E	O	E
NUMBER	IDENTIFICATION				C	M	R
							REMARKS
							CALIBRATION BLOCK
<hr/>							
<u>LINE 1-RC-16 (FIG NO A-36)</u>							
056700	1-RC-16-05F	B-J	PT	12QHP5050NDE001	X	-	-
	PIPE TO VALVE	B9.21			-	-	J.O. C35884-06
					-	-	
					-	-	
057300	1-RC-16-11F	B-J	PT	12QHP5050NDE001	X	-	-
	PIPE TO ELBOW	B9.21			-	-	J.O. C35884-06
					-	-	
					-	-	
<u>LINE 1-RC-16 (FIG NO A-37)</u>							
057550	1-RC-16-13S-FB	B-G-2	VT-1	12QHP5050NDE006	X	-	-
	FLANGE BOLTING	B7.50			-	-	J.O. C35884-06
					-	-	
					-	-	
<u>LINE 1-RC-501 (FIG NO A-38)</u>							
060100	1-RC-501-05S	B-J	PT	12QHP5050NDE001	X	-	-
	ELBOW TO PIPE	B9.40			-	-	J.O. C35884-06
					-	-	
					-	-	
<u>LINE 1-RC-504 (FIG NO A-39)</u>							
061100	1-RC-504-03S	B-J	PT	12QHP5050NDE001	X	-	-
	PIPE TO CROSS	B9.40			-	-	J.O. C35884-06
					-	-	
					-	-	
062100	1-RC-504-13S	B-J	PT	12QHP5050NDE001	X	-	-
	PIPE TO ELBOW	B9.40			-	-	J.O. C35884-06
					-	-	
					-	-	
063100	1-RC-504-23S	B-J	PT	12QHP5050NDE001	X	-	-
	PIPE TO MANIFOLD	B9.40			-	-	J.O. C35884-06
					-	-	
					-	-	

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THIRD INTERVAL, FIRST PERIOD, FIRST OUTAGE (97RF)
CLASS 1 COMPLETED COMPONENTS

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REACTOR COOLANT SYSTEM

SUMMARY EXAMINATION AREA		ASME			N	O			
		SEC. XI			O	G	T		
		CATGY	EXAM		R	E	H		
NUMBER	IDENTIFICATION	ITEM NO	METHOD	PROCEDURE	E	O	E	REMARKS	
					C	M	R	**CALIBRATION BLOCK**	

LINE 1-RC-505 (FIG NO A-40)									
063700	1-RC-505-02S PIPE TO ELBOW	B-J B9.40	PT	12QHP5050NDE001	X	-	-	- J.O. C35884-06	
					-	-	-		
					-	-	-		
065400	1-RC-505-19S PIPE TO MANIFOLD	B-J B9.40	PT	12QHP5050NDE001	X	-	-	- J.O. C35884-05	
					-	-	-		
					-	-	-		
066200	1-RC-505-27S FLANGE TO PIPE	B-J B9.40	PT	12QHP5050NDE001	X	-	-	- J.O. C35884-06	
					-	-	-		
					-	-	-		
LINE 1-RC-506 (FIG NO A-41)									
070000	1-RC-506-15S ELBOW TO PIPE	B-J B9.40	PT	12QHP5050NDE001	X	-	-	- J.O. C35884-06	
					-	-	-		
					-	-	-		
070100	1-RC-506-28S PIPE TO FLANGE	B-J B9.40	PT	12QHP5050NDE001	X	-	-	- J.O. C35884-06	
					-	-	-		
					-	-	-		
070150	1-RC-506-289-FB FLANGE BOLTING	B-G-2 B7.50	VT-1	12QHP5050NDE006	X	-	-	- J.O. C35884-06	
					-	-	-		
					-	-	-		
071100	1-RC-506-35S VALVE TO PIPE	B-J B9.40	PT	12QHP5050NDE001	X	-	-	- J.O. C35884-06	
					-	-	-		
					-	-	-		
LINE 1-RC-507 (FIG NO A-42)									
071600	1-RC-507-04S CROSS TO REDUCER	B-J B9.40	PT	12QHP5050NDE001	X	-	-	- J.O. C35884-06	
					-	-	-		
					-	-	-		

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INSERVICE INSPECTION SUMMARY
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REACTOR COOLANT SYSTEM

SUMMARY EXAMINATION AREA		ASME			N	O	
		SEC. XI			O	G	T
		CATGY	EXAM		R	E	H
		ITEM NO	METHOD	PROCEDURE	E	O	E
NUMBER	IDENTIFICATION				C	M	R
							REMARKS
							CALIBRATION BLOCK
<hr/>							
<u>LINE 1-RC-507 (FIG NO A-42)</u>							
071700	1-RC-507-05S	B-J	PT	12QHP5050NDE001	X	-	-
	CROSS TO REDUCER	B9.40			-	-	J.O. C35884-06
					-	-	
<u>LINE 1-RC-508 (FIG NO A-43)</u>							
074200	1-RC-508-02S	B-J	PT	12QHP5050NDE001	X	-	-
	REDUCER TO TEE	B9.40			-	-	J.O. C35884-06
					-	-	
075000	1-RC-508-10S	B-J	PT	12QHP5050NDE001	X	-	-
	ELBOW TO PIPE	B9.40			-	-	J.O. C35884-06
					-	-	
075500	1-RC-508-16S	B-J	PT	12QHP5050NDE001	X	-	-
	VALVE TO PIPE	B9.40			-	-	J.O. C35884-06
					-	-	
076700	1-RC-508-20S	B-J	PT	12QHP5050NDE001	X	-	-
	TEE TO PIPE	B9.40			-	-	J.O. C35884-06
					-	-	
<u>LINE 1-RC-509 (FIG NO A-44)</u>							
080900	1-RC-509-29S	B-J	PT	12QHP5050NDE001	X	-	-
	FLANGE TO PIPE	B9.40			-	-	J.O. C35884-06
					-	-	
081100	1-RC-509-29S-B	B-J	PT	12QHP5050NDE001	X	-	-
	COUPLING TO PIPE	B9.40			-	-	J.O. C35884-06
					-	-	

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DETECTOR COOLANT SYSTEM

SUMMARY EXAMINATION AREA		ASME			N	O	
		SEC. XI			O	G	T
		CATGY	EXAM		R	E	H
NUMBER IDENTIFICATION		ITEM NO	METHOD	PROCEDURE	E	O	E
					C	M	R
							REMARKS
							CALIBRATION BLOCK

LINE 1-RC-509 (FIG NO A-44)

082100	1-RC-509-36F	B-J	PT	12QHP5050NDE001	X	-	-	-	J.O. C35884-06
	PIPE TO TEE	B9.21			-	-	-		
					-	-	-		

LINE 1-RC-510 (FIG NO A-45)

084600	1-RC-510-24S	B-J	PT	12QHP5050NDE001	X	-	-	-	J.O. C35884-06
	MANIFOLD TO PIPE	B9.40			-	-	-		
					-	-	-		

084700	1-RC-510-25S	B-J	PT	12QHP5050NDE001	X	-	-	-	J.O. C35884-06
	PIPE TO ELBOW	B9.40			-	-	-		
					-	-	-		

LINE 1-RC-511 (FIG NO A-46)

086400	1-RC-511-15F	B-J	PT	12QHP5050NDE001	-	-	X	-	J.O. C35884-06 - .125" ROUNDED
	ELBOW TO PIPE	B9.40			-	-	-		INDICATION NOTED WHICH IS ACCEPTABLE TO
					-	-	-		1989 SEC. III NB-5352.

086700	1-RC-511-16S	B-J	PT	12QHP5050NDE001	X	-	-	-	J.O. C35884-06
	PIPE TO TEE	B9.40			-	-	-		
					-	-	-		

088700	1-RC-511-36F	B-J	PT	12QHP5050NDE001	X	-	-	-	J.O. C35884-06
	PIPE TO TEE	B9.21			-	-	-		
					-	-	-		

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INSERVICE INSPECTION SUMMARY

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THIRD INTERVAL, FIRST PERIOD, FIRST OUTAGE (97RF)

CLASS 1 COMPLETED COMPONENTS

ETY INJECTION SYSTEM

SUMMARY EXAMINATION AREA		ASME			N	O	
		SEC. XI			O	G	T
		CATGY	EXAM		R	E	H
NUMBER	IDENTIFICATION	ITEM NO	METHOD	PROCEDURE	E	O	E
					C	M	R
							REMARKS
							CALIBRATION BLOCK
<hr/>							
<u>LINE 1-SI-22 (FIG NO A-48)</u>							
091600	1-SI-22-06F	B-K-1	PT	12QHP5050NDE001	-	-	X
	COLLAR TO PIPE	B10.10			-	-	-
					-	-	-
							J.O. C35884-06 - FOUR (4)
							INDICATIONS NOTED THAT WERE WITHIN
							ALLOWABLE STANDARDS.
<u>LINE 1-SI-23 (FIG NO A-49)</u>							
094100	1-SI-23-11S	B-J	PT	54-ISI-240-36	X	-	-
	ELBOW TO PIPE	B9.11	UT45	54-ISI-121-1	X	-	-
			UT45T		X	-	-
							FTI
							3378028 (6-SS-160-.71)
<u>LINE 1-SI-25 (FIG NO A-50)</u>							
096100	1-SI-25-25S	B-J	PT	54-ISI-240-36	X	-	-
	ELBOW TO PIPE	B9.11	UT45	54-ISI-121-1	X	-	-
			UT45T		X	-	-
			UT60		X	-	-
							FTI - LIMITED EXAM OF 92% COVERAGE
							DUE TO ELBOW CONFIGURATION.
							3378029 (8-SS-140-.81)
<u>LINE 1-SI-26 (FIG NO A-51)</u>							
098500	1-SI-26-17F	B-J	PT	54-ISI-240-36	X	-	-
	ELBOW TO BRANCH CONNECTION	B9.11	UT45	54-ISI-121-1	X	-	-
			UT45T		X	-	-
			UT60		X	-	-
							FTI - LIMITED EXAM OF 92% COVERAGE
							DUE TO BRANCH CONNECTION CONFIGURATION.
							3378028 (6-SS-160-.71)
<u>LINE 1-SI-27 (FIG NO A-52)</u>							
099600	1-SI-27-11S	B-J	PT	54-ISI-240-36	X	-	-
	ELBOW TO PIPE	B9.11	UT45	54-ISI-121-1	X	-	-
			UT45T		X	-	-
							FTI
							3378028 (6-SS-160-.71)

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STY INJECTION SYSTEM

SUMMARY EXAMINATION AREA		ASME			N	O	
		SEC. XI			O	G	T
		CATGY	EXAM		R	E	H
NUMBER IDENTIFICATION		ITEM NO	METHOD	PROCEDURE	E	O	E
					C	M	R
							REMARKS
							CALIBRATION BLOCK
<hr/>							
<u>LINE 1-SI-27 (FIG NO A-52)</u>							
100200	1-SI-27-18F	B-J	PT	54-ISI-240-36	X	-	-
	VALVE TO ELBOW	B9.11	UT45	54-ISI-121-1	X	-	-
			UT45T		X	-	-
			UT60		X	-	-
							- FTI - LIMITED EXAM OF 92% COVERAGE DUE TO VALVE CONFIGURATION.
							3378028 (6-SS-160-.71)
<u>LINE 1-SI-29 (FIG NO A-53)</u>							
101200	1-SI-29-06F	B-K-1	PT	54-ISI-240-36	X	-	-
	COLLAR TO PIPE	B10.10			-	-	-
					-	-	-
102100	1-SI-29-16S	B-J	PT	54-ISI-240-36	X	-	-
	PIPE TO ELBOW	B9.11	UT45	54-ISI-121-1	X	-	-
			UT45T		X	-	-
							- FTI
							3378030 (10-SS-140-1.0)
102900	1-SI-29-25F	B-J	PT	54-ISI-240-36	X	-	-
	VALVE TO ELBOW	B9.11	UT45	54-ISI-121-1	X	-	-
			UT45T		X	-	-
			UT60		X	-	-
							- FTI
							3378030 (10-SS-140-1.0)
<u>LINE 1-SI-69 (FIG NO A-54)</u>							
103100	1-SI-69-23F	B-J	PT	54-ISI-240-36	X	-	-
	VALVE TO REDUCER	B9.11	UT45	54-ISI-121-1	X	-	-
			UT45T		X	-	-
							- FTI - LIMITED EXAM OF 92% COVERAGE DUE TO VALVE CONFIGURATION.
							3378028 (6-SS-160-.71)
<u>LINE 1-SI-31 (FIG NO A-54)</u>							
104200	1-SI-31-09F	B-J	PT	54-ISI-240-36	X	-	-
	ELBOW TO PIPE	B9.11	UT45	54-ISI-121-1	X	-	-
			UT45T		X	-	-
							- FTI
							3378030 (10-SS-140-1.0)



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CLASS 1 COMPLETED COMPONENTS

SAFETY INJECTION SYSTEM

SUMMARY EXAMINATION AREA		ASME			N	O		
		SEC. XI			O	G	T	
		CATGY	EXAM		R	E	H	
		ITEM NO	METHOD	PROCEDURE	E	O	E	REMARKS
NUMBER	IDENTIFICATION				C	M	R	**CALIBRATION BLOCK**

<u>LINE 1-SI-33 (FIG NO A-55)</u>								
107400	1-SI-33-11F ELBOW TO PIPE	B-J	PT	54-ISI-240-36	X	-	-	- FTI
		B9.11	UT45	54-ISI-121-1	X	-	-	
			UT45T		X	-	-	
								3378030 (10-SS-140-1.0)
107600	1-SI-33-13S ELBOW TO PIPE	B-J	PT	54-ISI-240-36	X	-	-	- FTI
		B9.11	UT45	54-ISI-121-1	X	-	-	
			UT45T		X	-	-	
								3378030 (10-SS-140-1.0)
108800	1-SI-33-26F ELBOW TO BRANCH CONNECTION	B-J	PT	54-ISI-240-36	X	-	-	- FTI
		B9.11	UT45	54-ISI-121-1	X	-	-	
			UT45T		X	-	-	
			UT60		X	-	-	
								3378030 (10-SS-140-1.0)
<u>LINE 1-SI-35 (FIG NO A-56)</u>								
110600	1-SI-35-10S PIPE TO ELBOW	B-J	PT	54-ISI-240-36	X	-	-	- FTI
		B9.11	UT45	54-ISI-121-1	X	-	-	
			UT45T		X	-	-	
								3378030 (10-SS-140-1.0)
111200	1-SI-35-15S PIPE TO ELBOW	B-J	PT	54-ISI-240-36	X	-	-	- FTI
		B9.11	UT45	54-ISI-121-1	X	-	-	
			UT45T		X	-	-	
								3378030 (10-SS-140-1.0)
111700	1-SI-35-20F PIPE TO VALVE	B-J	PT	54-ISI-240-36	X	-	-	- FTI
		B9.11	UT45	54-ISI-121-1	X	-	-	
			UT45T		X	-	-	
								3378030 (10-SS-140-1.0)

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CLASS 1 COMPLETED COMPONENTS

SAFETY INJECTION SYSTEM

SUMMARY EXAMINATION AREA		ASME			N	O	
		SEC. XI			O	G	T
		CATGY	EXAM		R	E	H
		ITEM NO	METHOD	PROCEDURE	E	O	E
NUMBER	IDENTIFICATION				C	M	R
							REMARKS
							CALIBRATION BLOCK
<hr/>							
<u>LINE 1-SI-545 (FIG NO A-60)</u>							
112605	1-SI-545-09F	B-J	PT	12QHP5050NDE001	X	-	-
	REDUCER TO BEND	B9.40			-	-	J.O. C35884-06
					-	-	
<u>LINE 1-SI-545 (FIG NO A-61)</u>							
113000	1-SI-545-30F	B-J	PT	12QHP5050NDE001	X	-	-
	PIPE TO ELBOW	B9.40			-	-	J.O. C35884-06
					-	-	
<u>LINE 1-SI-545 (FIG NO A-62)</u>							
114100	1-SI-545-41S	B-J	PT	12QHP5050NDE001	X	-	-
	ELBOW TO PIPE	B9.40			-	-	J.O. C35884-06
					-	-	
115300	1-SI-545-53F	B-J	PT	12QHP5050NDE001	X	-	-
	ELBOW TO PIPE	B9.40			-	-	J.O. C35884-06
					-	-	
<u>LINE 1-SI-546 (FIG NO A-65)</u>							
117800	1-SI-546-33F	B-J	PT	12QHP5050NDE001	X	-	-
	ELBOW TO PIPE	B9.40			-	-	J.O. C35884-06
					-	-	
117900	1-SI-546-34S	B-J	PT	12QHP5050NDE001	X	-	-
	PIPE TO ELBOW	B9.40			-	-	J.O. C35884-06
					-	-	
118300	1-SI-546-38F	B-J	PT	12QHP5050NDE001	X	-	-
	PIPE TO ELBOW	B9.40			-	-	J.O. C35884-06
					-	-	

123250	1-SI-549-12S	B-J	PT	12QHP5050NDE001	X	-	-	-	J.O. C35884-06
	PIPE TO ELBOW	B9.40			-	-	-		

INDIANA MICHIGAN POWER COMPANY, DONALD C. COOK PLANT, BRIDGMAN, MI.
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INSERVICE INSPECTION SUMMARY

THIRD INTERVAL, FIRST PERIOD, FIRST OUTAGE (97RF)

CLASS 1 COMPLETED COMPONENTS

ETY INJECTION SYSTEM

SUMMARY EXAMINATION AREA		ASME			N	O	
		SEC. XI			O	G	T
		CATGY	EXAM		R	E	H
		ITEM NO	METHOD	PROCEDURE	E	O	E
NUMBER	IDENTIFICATION				C	M	R
							REMARKS
							CALIBRATION BLOCK

LINE 1-SI-549 (FIG NO A-69)

123300	1-SI-549-17S	B-J	PT	12QHP5050NDE001	X	-	-	-	J.O. C35884-06
	ELBOW TO PIPE	B9.40			-	-	-		

LINE 1-SI-549 (FIG NO A-72)

123800	1-SI-549-40S	B-J	PT	12QHP5050NDE001	X	-	-	-	J.O. C35884-06
	PIPE TO ELBOW	B9.40			-	-	-		

LINE 1-SI-549 (FIG NO A-73)

124700	1-SI-549-49F	B-J	PT	12QHP5050NDE001	X	-	-	-	J.O. C35884-06
	ELBOW TO PIPE	B9.40			-	-	-		

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COOK NUCLEAR PLANT UNIT 1
INSERVICE INSPECTION SUMMARY
THIRD INTERVAL, FIRST PERIOD, FIRST OUTAGE (97RP)
CLASS 1 COMPLETED COMPONENTS

CHEMICAL AND VOLUME CONTROL SYSTEM

				N	O		
				O	G	T	
				R	E	H	
				E	O	E	REMARKS
SUMMARY EXAMINATION AREA	CATGY	EXAM		C	M	R	**CALIBRATION BLOCK**
NUMBER IDENTIFICATION	ITEM NO	METHOD	PROCEDURE				
-----				-	-	-	-----

LINE 1-CS-96 (FIG NO A-75)

128600	1-CS-96-53S	B-J	PT	12QHP5050NDE001	X	-	-	-	J.O. C35884-06
	PIPE TO ELBOW	B9.21			-	-	-		
					-	-	-		

LINE 1-CS-99 (FIG NO A-76)

129800	1-CS-99-05S	B-J	PT	12QHP5050NDE001	X	-	-	-	J.O. C35884-06
	TEE TO PIPE	B9.21			-	-	-		
					-	-	-		

130700	1-CS-99-14S	B-J	PT	12QHP5050NDE001	X	-	-	-	J.O. C35884-06
	ELBOW TO PIPE	B9.21			-	-	-		
					-	-	-		

130000	1-CS-99-19S	B-J	PT	12QHP5050NDE001	X	-	-	-	J.O. C35884-06
	ELBOW TO PIPE	B9.21			-	-	-		
					-	-	-		

LINE 1-CS-780 (FIG NO A-78)

141800	1-CS-780-13S	B-J	PT	12QHP5050NDE001	X	-	-	-	J.O. C35884-06
	ELBOW TO PIPE	B9.40			-	-	-		
					-	-	-		

143000	1-CS-780-18F	B-J	PT	12QHP5050NDE001	X	-	-	-	J.O. C35884-06
	PIPE TO REDUCER	B9.21			-	-	-		
					-	-	-		



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INSERVICE INSPECTION SUMMARY

THIRD INTERVAL, FIRST PERIOD, FIRST OUTAGE (97RF)

CLASS 1 COMPLETED COMPONENTS

RESIDUAL HEAT REMOVAL SYSTEM

SUMMARY EXAMINATION AREA		ASME			N	O	
		SEC. XI			O	G	T
		CATGY	EXAM		R	E	H
NUMBER IDENTIFICATION		ITEM NO	METHOD	PROCEDURE	E	O	E
					C	M	R
							REMARKS
							CALIBRATION BLOCK

LINE 1-RH-28 (FIG NO A-80)

144700	1-RH-28-10S	B-J	PT	54-ISI-240-36	X	-	-	-	FTI
	PIPE TO ELBOW	B9.11	UT45	54-ISI-121-1	X	-	-		
			UT45T		X	-	-		

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INSERVICE INSPECTION SUMMARY
THIRD INTERVAL, FIRST PERIOD, FIRST OUTAGE (97RF)
CLASS 1 COMPLETED COMPONENTS

WASTE DISPOSAL SYSTEM

SUMMARY EXAMINATION AREA		ASME			N	O	
		SEC. XI			O	G	T
		CATGY	EXAM		R	E	H
		ITEM NO	METHOD	PROCEDURE	E	O	E
NUMBER	IDENTIFICATION				C	M	R
							REMARKS
							CALIBRATION BLOCK

LINE 1-WD-644 (FIG NO A-84)

147500	1-WD-644-06S	B-J	PT	12QHP5050NDE001	X	-	-	-	J.O. C35884-06
	PIPE TO TEE	B9.40			-	-	-		
					-	-	-		

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COOK NUCLEAR PLANT UNIT 1
INSERVICE INSPECTION SUMMARY
THIRD INTERVAL, FIRST PERIOD, FIRST OUTAGE (97RF)
CLASS 1 COMPLETED COMPONENTS

ATOR COOLANT PUMP NO. 11

SUMMARY EXAMINATION AREA		ASME			N	O	
		SEC. XI			O	G	T
		CATGY	EXAM		R	E	H
NUMBER	IDENTIFICATION	ITEM NO	METHOD	PROCEDURE	E	O	E
					C	M	R
							REMARKS
							CALIBRATION BLOCK

PUMP FLYWHEEL

148600	FLYWHEEL (IN PLACE)	-- RG1.14	UTO	54-ISI-117-16	X	-	-	- FTI - IN PLACE UT PERFORMED ON BORE AND KEYWAY AREA.
					-	-	-	
					-	-	-	**RC-FLYWHEEL-CS-31-DCC**

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INSERVICE INSPECTION SUMMARY
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CLASS 1 COMPLETED COMPONENTS

DETECTOR COOLANT PUMP NO. 13

SUMMARY EXAMINATION AREA		ASME				N	O	
NUMBER	IDENTIFICATION	SEC. XI	CATGY	EXAM	PROCEDURE	O	G	T
		ITEM NO	METHOD			R	E	H
						E	O	E
						C	M	R
								REMARKS
								CALIBRATION BLOCK

PUMP FLYWHEEL (FIG NO A-86)

150200	FLYWHEEL	--	UT0	54-ISI-117-16	X	-	-	-	FTI- IN-PLACE UT PERFORMED ON BORE
	(IN PLACE)	RG1.14			-	-	-	-	AND KEYWAY AREA.
					-	-	-	-	
									RC-FLYWHEEL-CS-31-DCC

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COOK NUCLEAR PLANT UNIT 1
INSERVICE INSPECTION SUMMARY
THIRD INTERVAL, FIRST PERIOD, FIRST OUTAGE (97RF)
CLASS 1 COMPLETED COMPONENTS

REACTOR COOLANT PUMP NO. 14

				N	O	
				O	G	T
				R	E	H
SUMMARY EXAMINATION AREA	CATGY	EXAM		E	O	E
NUMBER IDENTIFICATION	ITEM NO	METHOD	PROCEDURE	C	M	R
						REMARKS
						CALIBRATION BLOCK

PUMP FLYWHEEL (FIG NO A-86)

151000	FLYWHEEL	--	UT0	54-ISI-117-16	X	-	-	-	FTI - IN-PLACE UT PERFORMED ON BORE
	(IN PLACE)	RG1.14			-	-	-	-	AND KEYWAY AREA.
					-	-	-	-	
									RC-FLYWHEEL-CS-31-DCC

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COOK NUCLEAR PLANT UNIT 1
INSERVICE INSPECTION SUMMARY
THIRD INTERVAL, FIRST PERIOD, FIRST OUTAGE (97RF)
CLASS 1 COMPLETED COMPONENTS

LOW PRESSURE RETAINING BOLTING (2" AND LESS)

SUMMARY EXAMINATION AREA		ASME			N	O		
		SEC. XI			O	G	T	
		CATGY	EXAM			R	E	H
		ITEM NO	METHOD	PROCEDURE	E	O	E	REMARKS
NUMBER	IDENTIFICATION				C	M	R	**CALIBRATION BLOCK**

<u>LINE 1-SI-69 (FIG NO A-54)</u>								
225900	SI-161-L2	B-G-2	VT-1	12QHP5050NDE006	X	-	-	J.O. C35884-06
	FLANGE BOLTING	B7.70			-	-	-	
					-	-	-	

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COOK NUCLEAR PLANT UNIT 1
INSERVICE INSPECTION SUMMARY
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CLASS 1 COMPLETED COMPONENTS

LYCO SWING CHECK VALVES

				N	O	
				O	G	T
				R	E	H
SUMMARY EXAMINATION AREA				E	O	E
NUMBER IDENTIFICATION				C	M	R
						REMARKS
						CALIBRATION BLOCK

LINE 1-SI-26 (FIG NO A-51)

231300	S1-158-L1	B-M-2	VT-1	12QHP5050NDE006	X	-	-	-	J.O. C35884-06
	VALVE BODY INTERNAL SURFACE	B12.50			-	-	-		

AM GENERATOR NO. 11

SUMMARY EXAMINATION AREA		ASME	O G T	
		SEC. XI	R E H	
NUMBER	IDENTIFICATION	CATGY	EXAM	E O E
		ITEM NO	METHOD	PROCEDURE
				C M R
				REMARKS
				CALIBRATION BLOCK

CIRCUMFERENTIAL WELDS (FIG NO B-1)

300000	STM-11-04	C-A	UT0	54-ISI-130-32	-	X	-	-	FTI
	TUBE SHEET TO STUB BARREL	C1.30	UT35		-	X	-		
			UT35T		-	X	-		
			UT45		-	X	-		
			UT45T		-	X	-		**PL-3.0-CS-22-DCC**
			UT60		X	-	-		
			UT60T		X	-	-		
300010	STM-11-05	C-A	UT0	54-ISI-130-32	X	-	-	-	FTI
	STUB BARREL TO SHELL	C1.10	UT35		X	-	-		
			UT35T		X	-	-		
			UT45		X	-	-		
			UT45T		X	-	-		**PL-3.0-CS-22-DCC**

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THIRD INTERVAL, FIRST PERIOD, FIRST OUTAGE (97RF)

CLASS 2 COMPLETED COMPONENTS

AM GENERATOR NO. 14

SUMMARY EXAMINATION AREA

NUMBER IDENTIFICATION

SEC. XI

CATGY

CATGY	EXAM
-------	------

ITEM NO

METHOD

PROCEDURE

O G T

R E H

E O E

C M R

REMARKS

••CALIBRATION BLOCK••

NOZZLE INSIDE RADIUS SECTIONS (FIG NO B-4)

300420 STM-14-FWN-IRS
INSIDE RADIUS SECTION

C2.22

UT26

54-ISI-132-0

- x -

- FTI

♦♦IR-CS-33-DCC♦♦

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INSERVICE INSPECTION SUMMARY
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CLASS 2 COMPLETED COMPONENTS

GENERATIVE HEAT EXCHANGER

SUMMARY EXAMINATION AREA		ASME			N	O	
		SEC. XI			O	G	T
		CATGY	EXAM		R	E	H
		ITEM NO	METHOD	PROCEDURE	E	O	E
NUMBER	IDENTIFICATION				C	M	R
							REMARKS
							CALIBRATION BLOCK
<hr/>							
<u>CIRCUMFERENTIAL WELDS (FIG NO B-5)</u>							
300520	RHE-1-26	C-A	UT45	54-ISI-123-1	-	X	-
	CAP TO SHELL	C1.20	UT45T		X	-	-
					-	-	-
							- FTI - LIMITED EXAM OF 98% COVERAGE
							DUE TO BRANCH CONNECTION.
							9.625-CCSS-X-1.0-2-DCC
300530	RHE-1-28	C-A	UT45	54-ISI-123-1	X	-	-
	TUBE SHEET TO SHELL	C1.30	UT45T		X	-	-
					-	-	-
							- LIMITED EXAM OF 82% COVERAGE DUE TO
							VESSEL SUPPORT AND BRANCH CONNECTION.
							9.625-CCSS-X-1.0-2-DCC
300540	RHE-1-29	C-A	UT45	54-ISI-123-1	X	-	-
	SHELL TO TUBE SHEET	C1.30	UT45T		X	-	-
					-	-	-
							- FTI - LIMITED EXAM OF 53% COVERAGE
							DUE TO VESSEL SUPPORT AND BRANCH
							CONNECTION.
							9.625-CCSS-X-1.0-2-DCC
300550	RHE-1-32	C-A	UT45	54-ISI-123-1	-	X	-
	SHELL TO CAP	C1.20	UT45T		X	-	-
					-	-	-
							- FTI - LIMITED EXAM OF 99% COVERAGE DUE
							TO BRANCH CONNECTION.
							9.625-CCSS-X-1.0-2-DCC





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INSERVICE INSPECTION SUMMARY
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CLASS 2 COMPLETED COMPONENTS

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HEAT EXCHANGER EAST

				N	O	
				O	G	T
				R	E	H
SUMMARY EXAMINATION AREA	CATGY	EXAM		E	O	E
NUMBER IDENTIFICATION	ITEM NO	METHOD	PROCEDURE	C	M	R
				-	-	-

REMARKS
CALIBRATION BLOCK

CIRCUMFERENTIAL WELDS (FIG NO B-9)

300749	E-RHRHEX-1	C-A	UT45	54-ISI-121-1	X	-	-	-	FTI - LIMITED EXAM OF 71% COVERAGE
	FLANGE TO SHELL	C1.10	UT45T		X	-	-	-	DUE TO FLANGE CONFIGURATION AND NOZZLE.
					-	-	-		

PL-SS-1.0-27-DCC

NOZZLE TO SHELL AND SHELL TO NOZZLE WELDS (FIG NO B-9)

300760	E-RHRHEX-IN	C-B	UT45	54-ISI-121-1	X	-	-	-	FTI - LIMITED EXAM OF 83% COVERAGE
	NOZZLE TO SHELL	C2.21	UT45T		X	-	-	-	DUE TO FLANGE CONFIGURATION AND SUPPORT
					-	-	-		INTERFERENCE.

PL-SS-1.0-27-DCC

INTEGRALLY WELDED VESSEL SUPPORTS (FIG NO B-9)

300800	E-RHRHEX-1VS	C-C	PT	12QHP5050NDE001	-	-	X	-	J.O. C35893-07 - ONE ROUNDED
	VESSEL SUPPORT	C3.10			-	-	-	-	INDICATION OF .156" ACCEPTABLE PER 1989
					-	-	-	-	ASME SEC. III NC-5352. LIMITED EXAM OF
					-	-	-	-	90% COVERAGE DUE TO CONCRETE SUPPORT
					-	-	-	-	INTERFERENCE.

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SUMMARY EXAMINATION AREA	
NUMBER	IDENTIFICATION

ASME
SEC. XI

N O
O G T
R E H
E O E
C M R

REMARKS

****CALIBRATION BLOCK****

301680	1-SI-3-12F	C-F-1	PT	54-ISI-240-36	X - - - FTI
	ELBOW TO PIPE	C5.11	UT45	54-ISI-121-1	X - -
			UT45T		X - -
					12-SS-40-.406-29-DCC

301970	1-SI-3-31S	C-F-1	PT	54-ISI-240-36	X	-	-	-	FTI
	PIPE TO ELBOW	C5.11	UT45	54-ISI-121-1	X	-	-		
			UT45T		X	-	-		
									14-SS-40-.438-30-DCC

302140	1-SI-4-02S	C-F-1	PT	54-ISI-240-36	X - - - FTI
	PIPE TO ELBOW	C5.11	UT45	54-ISI-121-1	X - -
			UT45T		X - -
					14-SS-40-.438-30-DCC

302180	1-SI-4-05S-PL 1 THROUGH 4 PIPE LUG	C-C C3.20	PT	12QHP5050NDE001	X - - -	J.O. C35893-07
--------	---------------------------------------	--------------	----	-----------------	---------	----------------

303180	1-SI-25-23F*	C-F-1	PT	54-ISI-240-36	X - - - FTI
	PIPE TO ELBOW	C5.11	UT45	54-ISI-121-1	X - -
			UT45T		X - -
					3378029 (8-SS-140-.81)

303190	1-SI-25-24F	C-F-1	PT	54-ISI-240-36	X - -	- FTI - LIMITED EXAM OF 92% COVERAGE
	ELBOW TO VALVE	C5.11	UT45	54-ISI-121-1	X - -	DUE TO VALVE CONFIGURATION.
			UT45T		X - -	
			UT60		X - -	
						3378029 (8-SS-140-.81)

307110	1-RH-7-27F	C-F-1	PT	54-ISI-240-36	X	-	-	-	FTI
	PIPE TO REDUCER	C5.11	UT45	54-ISI-121-1	-	X	-		
			UT45T		X	-	-		

3378029 (8-SS-140-.81)

COOK NUCLEAR PLANT UNIT 1

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INSERVICE INSPECTION SUMMARY

INSERVICE INSPECTION SUMMARY

THIRD INTERVAL, FIRST PERIOD, FIRST OUTAGE (97RF)

CLASS 2 COMPLETED COMPONENTS

STAINMENT SPRAY PIPING SYSTEM

N O

O G T

R E H

E O E

C M R

METHOD

METHOD

• • • • •

PROCEDURE

E O E

C M R

— — —

REMARKS

****CALIBRATION BLOCK****

.....

LINE 1-CTS-6 (FIG NO B-48)

C-F-1

PT

54-ISI-240-36

X **-** **-**

- FTI

C5.11

UT45

54-ISI-121-1

x - -

UT45T

x - -

****8-SS-40-.322-17-DCC****

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INSERVICE INSPECTION SUMMARY
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CLASS 2 COMPLETED COMPONENTS

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WATER SYSTEM

SUMMARY EXAMINATION AREA		ASME			N	O	
NUMBER	IDENTIFICATION	SEC. XI	CATGY	EXAM	R	E	H
		ITEM NO	METHOD	PROCEDURE	E	O	E
					C	M	R
							REMARKS
							CALIBRATION BLOCK
<u>LINE 1-FW-10 (FIG NO B-49)</u>							
310850	1-FW-10-12S-PL 1 AND 2 PIPE LUG	C-C C3.20	MT	12QHP5050NDE002	X	-	- J.O. C35901-03
					-	-	
					-	-	
<u>LINE 1-FW-15 (FIG NO B-53)</u>							
311760	1-FW-15-13F PIPE TO PIPE	C-F-2 C5.51	MT UT45 UT45T	54-ISI-270-35 54-ISI-124-1	X	-	- FTI
					X	-	
					X	-	
							14-CS-80-.750-12-DCC
<u>LINE 1-FW-30 (FIG NO B-59)</u>							
312800	1-FW-30-01S CAP TO TEE	C-F-2 C5.51	MT UT45 UT45T	54-ISI-270-35 54-ISI-124-1	X	-	- FTI - LIMITED EXAM OF 92% COVERAGE
					X	-	DUE TO THE PROXIMITY OF AN ELECTRICAL
					X	-	CONDUIT.
							6-CS-80-.432-13-DCC
<u>LINE 1-FW-31 (FIG NO B-60)</u>							
313070	1-FW-31-06S PIPE TO ELBOW	C-F-2 C5.51	MT UT45 UT45T	54-ISI-270-35 54-ISI-124-1	X	-	- FTI
					X	-	
					X	-	
							6-CS-80-.432-13-DCC

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CLASS 2 COMPLETED COMPONENTS

DOWN STEAM ISOLATION VALVE-LINE 1-MS-2

				N	O	
				O	G	T
				R	E	H
SUMMARY EXAMINATION AREA				CATGY	EXAM	REMARKS
NUMBER	IDENTIFICATION	ITEM NO	METHOD	PROCEDURE	C	M R
				CALIBRATION BLOCK		
				C	M	R
				C	M	R

VALVE BODY WELDS (FIG NO B-75, B-76)

317060	MSIV-MRV-210-1	C-G	MT	12QHP5050NDE002	X - -	- J.O. C35893-07
	CYLINDER WELD	C6.20			- - -	
					- - -	

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D N STEAM ISOLATION VALVE-LINE 1-MS-11

SUMMARY EXAMINATION AREA		ASME			N	O	
		SEC. XI			O	G	T
		CATGY	EXAM		R	E	H
NUMBER	IDENTIFICATION	ITEM NO	METHOD	PROCEDURE	E	O	E
					C	M	R
					REMARKS		
					CALIBRATION BLOCK		

INTEGRALLY WELDED SUPPORTS (FIG NO B-75, B-76)

317170	MSIV-MRV-230-S1	C-C	MT	12QHP5050NDE002	X	-	-	- J.O. C35901-03 - LIMITED EXAM OF 79%
		C3.40			-	-	-	COVERAGE DUE TO SUPPORT STRUCTURE.
					-	-	-	

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CLASS 2 COMPLETED COMPONENTS

EMERGENCY CORE COOLING SYSTEM

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	ASME SEC. XI CATGY	EXAM METHOD	PROCEDURE	N O O G T R E H E O E C M R			REMARKS
-----	-----	-----	-----	-----	-	-	-	-----
<u>LINE 1-SI-5 (FIG NO B-77)</u>								
323030	1-SI-5-04S PIPE TO TEE	C-F-1 CS.11H	PT UT45 UT45T	54-ISI-240-36 54-ISI-121-1	X	-	-	- FTI **8-SS-10-.148-16-DCC**
323080	1-SI-5-09S PIPE TO TEE	C-F-1 CS.11H	PT UT45 UT45T	54-ISI-240-36 54-ISI-121-1	X	-	-	- FTI **6-SS-40-.280-28-DCC**
<u>LINE 1-SI-11A (FIG NO B-79)</u>								
325010	1-SI-11A-02S ELBOW TO PIPE	C-F-1 CS.21H	PT UT45 UT45T	54-ISI-240-36 54-ISI-121-1	X	-	-	- FTI **4-SS-80-.337-20-DCC**
325130	1-SI-11A-14F PIPE TO ELBOW	C-F-1 CS.21H	PT UT45 UT45T	54-ISI-240-36 54-ISI-121-1	X	-	-	- FTI **4-SS-80-.337-20-DCC**

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CLASS 2 COMPLETED COMPONENTS

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CHEMICAL AND VOLUME CONTROL SYSTEM

SUMMARY	EXAMINATION AREA	ASME SEC. XI	CATGY	EXAM	N O G T R E H E O E C M R	REMARKS
NUMBER	IDENTIFICATION	ITEM NO	METHOD	PROCEDURE		**CALIBRATION BLOCK**
<u>LINE 1-CS-32 (FIG NO B-89)</u>						
332300	1-CS-32-31S PIPE TO TEE	C-P-1 C5.21H	PT UT45 UT45T	54-ISI-240-36 54-ISI-121-1	X - - - X - X - -	- FTI- LIMITED EXAM OF 94% COVERAGE DUE TO TEE FITTING CONFIGURATION. **3378026 (3-SS-160-.430)**
<u>LINE 1-CS-34 (FIG NO B-90)</u>						
333370	1-CS-34-38S ELBOW TO PIPE	C-F-1 C5.21H	PT UT45 UT45T	54-ISI-240-36 54-ISI-121-1	X - - X - - X - -	- FTI **3378027 (4-SS-120-.430)**
<u>LINE 1-CS-36 (FIG NO B-91)</u>						
334000	1-CS-36-01S TEE TO PIPE	C-F-1 C5.21H	PT UT45 UT45T	54-ISI-240-36 54-ISI-121-1	X - - - X - X - -	- FTI- LIMITED EXAM OF 94% COVERAGE DUE TO TEE FITTING CONFIGURATION. **3378026 (3-SS-160-.430)**
<u>LINE 1-CS-41 (FIG NO B-92)</u>						
335050	1-CS-41-06F ELBOW TO VALVE	C-F-1 C5.21H	PT UT45 UT45T	54-ISI-240-36 54-ISI-121-1	X - - - X - X - -	- FTI **3378026 (3-SS-160-.430)**
<u>LINE 1-CS-747 (FIG NO B-95)</u>						
338070	1-CS-747-08S PIPE TO ELBOW	C-F-1 C5.30H	PT	12QHP5050NDE001	X - - - - -	- J.O. C35893-07

J.O. C35884-06

— — —

F1.10WC

DATE: 07/16/97

REVISION: 0

COOK NUCLEAR PLANT UNIT 1

INSERVICE INSPECTION SUMMARY

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THIRD INTERVAL, FIRST PERIOD, FIRST OUTAGE (97RF)

CLASS 2 COMPLETED COMPONENTS

CHEMICAL AND VOLUME CONTROL TANK

ASME

SEC. XI

CATGY

EXAM

ITEM NO

METHOD

PROCEDURE

N O

O G T

R E H

E O E

C M R

REMARKS

CALIBRATION BLOCK

SUMMARY EXAMINATION AREA

NUMBER IDENTIFICATION

COMPONENT SUPPORTS (FIG NO B-6)

700070 1-CVCT-SC-1

F-A

VT-3

12QHP5050NDE006

X - - -

J.O. C35893-07

F1.40

- - -

700080 1-CVCT-SC-2

F-A

VT-3

12QHP5050NDE006

X - - -

J.O. C35893-07

F1.40

- - -

INDIANA MICHIGAN POWER COMPANY, DONALD C. COOK PLANT, BRIDGMAN, MI.
UNIT 1 COMMERCIAL SERVICE DATE 8-23-75, NATIONAL BOARD NUMBER 20761

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COOK NUCLEAR PLANT UNIT 1
INSERVICE INSPECTION SUMMARY
THIRD INTERVAL, FIRST PERIOD, FIRST OUTAGE (97RF)
CLASS 2 COMPLETED COMPONENTS

IRON INJECTION TANK

SUMMARY EXAMINATION AREA		ASME			N	O	
		SEC. XI			O	G	T
		CATGY	EXAM		R	E	H
		ITEM NO	METHOD	PROCEDURE	E	O	E
NUMBER	IDENTIFICATION				C	M	R
							REMARKS
							CALIBRATION BLOCK

COMPONENT SUPPORTS (FIG NO B-11)

700190	1-BIT-SC-1	F-A	VT-3	12QHP5050NDE006	X	-	-	-	J.O. C35893-07
		F1.40			-	-	-		
					-	-	-		
700200	1-BIT-SC-2	F-A	VT-3	12QHP5050NDE006	X	-	-	-	J.O. C35893-07
		F1.40			-	-	-		
					-	-	-		

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CLASS 2 COMPLETED COMPONENTS

ASME

SEC. XI

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ITEM NO	METHOD
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PROCEDURE

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C M R

REMARKS

****CALIBRATION BLOCK****

LINE 1-SI-12 (FIG NO B-82)

741500	1-GSI-R-8	F-A	VT-3	12QHP5050NDE006	X	-	-	-	J.O. C35893-07
	PIPE SUPPORT COMPONENTS	F1.20WA			-	-	-		

741540	1-GSI-L-12	F-A	VT-3	12QHP5050NDE006	X	-	-	-	J.O. C35893-07
	PIPE SUPPORT COMPONENTS	F1.20WA			-	-	-		

LINE 1-SI-80 (FIG NO B-88)

744020	1-ASI-L-923	F-A	VT-3	12QHP5050NDE006	-	-	X	-	J.O. C35893-07 - DESIGN/AS-FOUND
	PIPE SUPPORT COMPONENTS	F1.20WB			-	-	-	-	DISCREPANCY. CONDITION WAS EVALUATED TO
					-	-	-	-	BE ACCEPTABLE. DRAWING TO BE REVISED.

INDIANA MICHIGAN POWER COMPANY
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TECHNICAL SPECIFICATION/INSERVICE INSPECTION
SURVEILLANCE OF HYDRAULIC SNUBBERS

The Unit 1 Tech. Spec./ISI snubber examination and test results for the period of October 25, 1995 to the end of the Unit 1 1997 Refueling Outage are noted below.

In past reporting periods, all Tech. Spec. snubbers were examined (visually inspected) prior to the test sample was reinstalled. Due to Tech. Spec. Amendment No. 173, the examination frequency has decreased and entire population examinations were not performed during this reporting period for the Grinnell Accessible Snubbers. However, all of the snubbers that were functionally tested were examined prior to removal and again after reinstallation. The Functional Test surveillances are separated in two (2) categories: "Grinnell" and "Steam Generator". The Visual Examination Surveillances were performed on two (2) categories: Grinnell Inaccessible and Steam Generator.

Current lists of those snubbers classified as "Accessible", "Inaccessible", and "Steam Generator" are listed on the following pages.

I FUNCTIONAL TESTS

<u>Category</u>	<u>Compl. Date</u>	<u>Comments/Result</u>
a.Grinnell	03-21-97	ten tested, all acceptable, see attached sheet
b.Steam Generator	03-19-97	four tested, all acceptable, see attached sheet

II VISUAL EXAMINATIONS

<u>Category</u>	<u>Compl. Date</u>	<u>Comments/Result</u>
a.Grinnell "Inaccessible"	03-13-97 04-04-97	all acceptable all acceptable
c.Steam Generator	03-15-97 04-04-97	all acceptable all acceptable

Details of each examination and test are included in applicable Job Order.

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FUNCTIONAL TEST SURVEILLANCES

<u>Snubber Mark No.</u>	<u>Date Tested</u>	<u>As-Found</u>
<u>Grinnell</u>		
1-GRC-S566	03-15-97	Pass
1-GRC-S594	03-15-97	Pass
1-GRC-S599	03-14-97	Pass
1-GRC-S604	03-14-97	Pass
1-FWS-9	03-14-97	Pass
1-MSS-2	03-13-97	Pass
1-GCCW-S838	03-21-97	Pass
12-GCCW-S839	03-21-97	Pass
1-GBD-S563	03-14-97	Pass
1-GRH-S7-B	03-21-97	Pass
<u>Steam Generator</u>		
1-OME-3-3-HSD-3L	03-15-97	Pass
1-OME-3-3-HSD-1L	03-17-97	Pass
1-OME-3-3-HSD-1U	03-17-97	Pass
1-OME-3-4-HSD-3U	03-19-97	Pass



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UNIT 1 GRINNELL "INACCESSIBLE" SNUBBERS

<u>Tech. Spec.</u>	<u>Mark Number</u>	<u>Azimuth</u>	<u>Elevation</u>	<u>Size (in.)</u>
1	1-GRC-S519	310	683	2 1/2 x 5
2	1-GRC-S537	25	610	2 1/2 x 5
3	1-GRC-S538	41	614	2 1/2 x 5
4	1-GRC-S555	141	614	2 1/2 x 5
5	1-GRC-S562	154	610	2 1/2 x 5
6	1-GRC-S564	313	614	2 1/2 x 5
7	1-GRC-S566	332	610	2 1/2 x 5
8	1-GRC-S573	223	614	2 1/2 x 5
9	1-GRC-S575	208	610	2 1/2 x 5
10	1-GRC-S582	212	617	1 1/2 x 5
11	1-GRC-S587	260	622	2 1/2 x 5
12	1-GRC-S592	292	683	3 1/4 x 5
13	1-GRC-S594	292	691	3 1/4 x 5
14	1-GRC-S596	285	691	2 1/2 x 5
15	1-GRC-S598	292	670	2 1/2 x 5
16	1-GRC-S599	287	672	2 1/2 x 5
17	1-GRC-S604	286	688	1 1/2 x 5
18	1-GRC-S608	286	693	1 1/2 x 5
19	1-GRC-S614	282	681	1 1/2 x 5
20	1-AFW-S4023	31	634	2 1/2 x 5
21	1-AFW-S4024-L	26	633	2 1/2 x 5
22	1-AFW-S4024-U	26	636	2 1/2 x 5
23	1-AFW-S4021	20	629	2 1/2 x 5
24	1-AFW-S4028-L	154	636	2 1/2 x 5
25	1-AFW-S4028-U	154	640	2 1/2 x 5
26	1-AFW-S4027	163	634	2 1/2 x 5
27	1-AFW-S4025	157	629	2 1/2 x 5
28	1-AFW-S4031	204	634	2 1/2 x 5
29	1-AFW-S4032-L	200	633	2 1/2 x 5
30	1-AFW-S4032-U	200	636	2 1/2 x 5
31	1-AFW-S4029	194	629	2 1/2 x 5
32	1-AFW-S4036-L	334	633	2 1/2 x 5
33	1-AFW-S4036-U	334	636	2 1/2 x 5
34	1-AFW-S4035	330	634	2 1/2 x 5
35	1-AFW-S4033	343	629	2 1/2 x 5

The balance of the Inaccessible Tech. Spec. Snubbers have been eliminated or replaced with struts based on piping analysis.

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UNIT 1 GRINNELL "ACCESSIBLE" SNUBBERS

<u>Tech. Spec.</u>	<u>Mark Number</u>	<u>Azimuth</u>	<u>Elevation</u>	<u>Size (in.)</u>
48	1-GCCW-S309		597	4 x 5
50	1-GCCW-S838		621	1 1/2 x 5
51	1-GCCW-S839		621	1 1/2 x 5
56	1-GBD-S563	277	608	1 1/2 x 5
57	1-GBD-S569	278	608	1 1/2 x 5
58	1-GBD-S573	181	607	1 1/2 x 5
59	1-GBD-S574	181	607	1 1/2 x 5

The balance of the Accessible Tech. Spec. Snubbers have been eliminated or replaced with struts based on piping analysis.

ISI/Non-Tech. Spec. Snubbers

1-GFW-S852	1 1/2 x 5
1-GFW-S854	1 1/2 x 5
1-GFW-S861	1 1/2 x 5
1-GFW-S862	2 1/2 x 5
1-GFW-S866	3 1/4 x 5

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UNIT 1, 800 Kip MCDOWELL-WELLMAN
STEAM GENERATOR SNUBBERS

<u>COMPONENT NUMBER</u>	<u>TECH. SPEC. NO.</u>	<u>SERIAL NUMBER</u>
1-OME-3-1-HSD-3U	79	25.12620.007-1
1-OME-3-1-HSD-3L	80	25.12620.007-5
1-OME-3-1-HSD-1U	81	25.12620.007-7
1-OME-3-1-HSD-1L	82	25.12620.007-14
<hr/>		
1-OME-3-2-HSD-1U	85	25.12620.007-9
1-OME-3-2-HSD-1L	84	25.12620.007-6
1-OME-3-2-HSD-3U	86	25.12620.007-15
1-OME-3-2-HSD-3U	83	25.12620.007-3
<hr/>		
1-OME-3-3-HSD-3U	88	25.12620.007-4
1-OME-3-3-HSD-3L	87	25.12620.007-2
1-OME-3-3-HSD-1U	90	25.12620.007-16
1-OME-3-3-HSD-1L	89	25.12620.007-8
<hr/>		
1-OME-3-4-HSD-1U	93	25.12620.007-12
1-OME-3-4-HSD-1L	92	25.12620.007-11
1-OME-3-4-HSD-3U	94	25.12620.007-13
1-OME-3-4-HSD-3L	91	25.12620.007-10

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SYSTEM PRESSURE TESTS

Inservice Inspection System Pressure Tests were performed during the 1997 Unit One Refueling Outage. These tests were in accordance with the requirements of ASME Section XI, 1989 Edition, Articles IWA-5000, IWB-5000, IWC-5000, and Code Case N-533 as applicable.

System functional, inservice and leakage tests were conducted on class 1 and 2 systems. Approximately fifty (50) system pressure tests were performed during the outage. The following is a list of systems in which VT-2 examinations were performed along with the job order activities:

<u>JOB ORDER ACTIVITY</u>	<u>SYSTEM INSPECTED</u>
R65227-01	RCS Full Pressure /Temperature Walkdown
R65911-01	RHR Suction from Loop 2 Hot Leg
-02	Low Head SI @ Cold Legs
-04	Low Head SI @ Hot Legs 2 & 3
-06	Low Head SI @ Hot Legs 1 & 4
-08	Boron Injection piping @ Cold Legs
-10	Excess Letdown HX from Loop 4
-11	Loop 1 Hot Leg Sample @ CPN-66
-12	Loop 3 Hot Leg Sample @ CPN-66
-13	Pressurizer Liquid Space @ CPN-66
-21	Mode 5 Walkdown per Code Case N-533 AEP:NRC:0969AY
R63064-01	Accumulator & Discharge Piping
-02	Station Drainage @ CPN-67
-04	Station Drainage @ CPN-41
-05	CCW Excess Letdown, RCP Thermal Barrier Oil Coolers & Reactor Support Coolers
-06	East RHR/CTS Recirc Piping
-07	West RHR/CTS Recirc Piping
-08	East Train RHR Normal Cooldown
-09	West Train RHR Normal Cooldown
-10	Feedwater & Main Stem including Steam Generator secondary side.
-11	Safety Valve Discharge @ CPN-15
-12	Control Air @ CPN-74
-13	Control Air @ CPN-29
-14	Nitrogen Piping to Accumulators @ CPN-36
-16	Excess Letdown Heat Exchanger
-18	Refueling Water Supply @ CPN-36
-19	Containment Purge @ CPN-65

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JOB ORDER ACTIVITY

SYSTEM INSPECTED

R63064-20	Containment Purge @ CPN-59
-21	Containment Purge @ CPN-64
-22	Containment Purge @ CPN-63
-23	Containment Purge @ CPN-60
-24	Containment Purge @ CPN-62
-25	Containment Purge @ CPN-61
-26	Pressurizer Steam Space @ CPN-66
-28	PAS Return Piping @ CPN-67
-32	#1 & 3 Hot Leg Sample @ CPN-66
-34	Pressurizer Liquid Space @ CPN-66
-40	Plant Air @ CPN-57
-42	Plant Air @ CPN-29
-45	1-ERS-1400 Supply @ CPN-31
-47	1-ERS-1300 Pas @ CPN-32
-51	1-ERS-1300/1400 supply @ CPN-70
-52	Upper Containment Air Sample @ CPN-89
-54	Instrument Room Air Sample @ CPN-92
-56	PAS Cntmt Hydrogen Valves @ CPN-93 & 95

The results of the system functional, inservice and leakage tests were all within acceptable ranges.

All procedures satisfy the testing and documentation requirements set forth in ASME Section XI, 1989 Edition, for Class 1 and 2 components and systems for Cook Nuclear Plant.

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STEAM GENERATOR INSPECTIONS AND REPAIR

Unit One Steam Generator inspections and repairs were performed from February 28, 1997 to April, 27, 1997 in accordance with plant Technical Specifications.

The following eddy current examinations were performed:

- a. Bobbin coil testing was performed on 100% of the tubes in service in all four steam generators.
- b. 3-Coil MRPC testing from the top of the tube sheet plus 3" to the end of the tube was performed on 100% of the unsleeved hot leg tubes in service and 20% of the cold leg tubes.
- c. A Zetec PlusPoint™ probe was used to inspect 100% of the sleeved tubes.
- d. A rotating pancake coil probe was used to inspect low row (tightly radiused) tubes in all generators. Rows 1, 2, and 3 were examined in steam generator #11 and rows 1 and 2 in steam generators #12, #13, and #14.

The following hydrostatic pressure tests were performed:

- a. 400 psig tests were performed prior to and following tube repairs.
- b. Twenty-five insitu pressure tests were performed in accordance with EPRI Guidelines.

One tube pull was conducted to examine the active degradation mechanisms in the generators.

All tubes exhibiting leaks or with indications exceeding Technical Specification limits were repaired by plugging or rerolling during this outage. No sleeving was performed.

INDIANA MICHIGAN POWER COMPANY
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TUBES INSPECTED/REPAIRED PER STEAM GENERATOR

Steam Generator				
Tubes	#11	#12	#13	#14
Inspected Bobbin Coil	2997	3082	3195	3092
Hot Leg 3-Coil MRPC Inspected	2171	2902	2736	2718
Cold Leg 3-Coil MRPC Inspected	678	678	678	678
Sleeve PlusPoint™ Inspected	826	180	459	374
Low Row RPC	224	174	160	173
Rerolled	197	137	297	74
Plugged	285	130	254	310

ACCUMULATED PERCENTAGE OF TUBES PLUGGED IN EACH STEAM
 GENERATOR, INCLUDING SLEEVE EQUIVALENT PLUGS:

Cook Nuclear Plant Plugging and Sleeving Summary Hot Leg					
Steam Generator	Sleeved Tubes	Sleeve Equiv. Plugs	Plugged Tubes	Total Equiv. Plugs	Percent Plugged Tubes
S/G 11	808	34.38	676	710.38	20.97%
S/G 12	170	7.23	436	313.66	13.08%
S/G 13	443	18.85	254	213.53	13.75%
S/G 14	373	15.87	606	311.91	18.36%
TOTALS	1794	76.33	2165	2241.33	16.54%

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NIS-2 Owner's Report for Repairs or Replacements
Index

COMPONENT NO.	JOB ORDER	ISI CLASS	REPAIR/REPLACEMENT
1-ACS-R930	C28393-14	2	REPLACEMENT
1-AIRLOCK-C650 EQUIPMENT HATCH	R51519-09	MC	REPLACEMENT
1-AIRLOCK-C650	R51519-11	MC	REPLACEMENT
1-ARC-R4000 1-ARC-R5300 1-ARC-R5301 1-ARC-R5302	C34216-01	1	REPLACEMENT
1-ASI-R5301	C34216-07	2	REPLACEMENT
1-CCM-432	C35012-03	2	REPAIR/REPLACEMENT
1-CCM-433	C34552-03	2	REPLACEMENT
1-CCM-451	C40400-01	2	REPLACEMENT
1-CCM-452	C40401-01	2	REPLACEMENT
1-CS-292	R57215-04	2	REPLACEMENT
1-CS-325	R51459-02	1	REPLACEMENT
1-CS-442-1	C33791-01	2	REPLACEMENT
1-DCR-206	C38539-01	2	REPLACEMENT
1-DCR-310	C40743-02	2	REPLACEMENT
1-FW-118-3	R61017-03	2	REPLACEMENT
1-FW-132-3	R20806-06	2	REPLACEMENT
1-FW-S9	C33453-01	2	REPLACEMENT
1-GBD-S563	C33456-01	2	REPLACEMENT
1-GCCW-R241	C38695-01	2	REPLACEMENT
1-GCS-R505	C34216-01	1	REPLACEMENT

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1-GCS-R531 1-GCS-R534 1-GCS-R626 1-GCS-R631 1-ACS-R4004	C34216-07	2	REPLACEMENT
1-GCS-R533	C33899-02	2	REPLACEMENT
1-GMS-L801	C39026-02	2	REPLACEMENT
1-GRC-R501 1-GRC-R524 1-GRC-R534	C34216-01	1	REPLACEMENT
1-GRC-R503	C33899-01	1	REPLACEMENT
1-GRC-R558	C34891-02	1	REPLACEMENT
1-GRC-R660 1-GRC-R622	C34216-01	1	REPLACEMENT
1-GRC-S566	C33446-01	1	REPLACEMENT
1-GRC-S604	C33452-01	1	REPLACEMENT
1-GRC-V550	C34216-01	1	REPLACEMENT
1-GRH-L51	C33638-01	2	REPLACEMENT
1-GRH-R503	C34215-01	2	REPLACEMENT
1-GSI-R527	C37156-02	2	REPLACEMENT
1-GSI-R542 1-GSI-R562 1-GSI-R589 1-GSI-R588 1-GSI-R634	C34215-01	2	REPLACEMENT
1-GSI-R559 1-GSI-R576 1-GSI-R637 1-ASI-R5302	C34215-01	1	REPLACEMENT
1-GSI-R575	C34216-07	1	REPLACEMENT
1-GSI-R660 1-GSI-R697 1-GSI-R703 1-GSI-R705	C34216-01	1	REPLACEMENT
1-ICM-250	C39642-03	1	REPLACEMENT

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1-IRV-311	C33810-07	2	REPLACEMENT
1-MRV-211	R36162-02	2	REPLACEMENT
1-MRV-221	R36163-02	2	REPLACEMENT
1-MRV-231	R36164-02	2	REPLACEMENT
1-MRV-241	C40682-02	2	REPLACEMENT
1-MS-135-2	C37089-08	2	REPLACEMENT
1-NMO-151	C28172-02	1	REPLACEMENT
1-NMO-152	C28173-02	1	REPLACEMENT
1-NMO-153	C28174-02	1	REPLACEMENT
1-NRV-152	C33535-04	1	REPLACEMENT
1-NRV-153	C33537-04	1	REPLACEMENT
1-NRV-163	C34692-03 -10	1	REPAIR/REPLACEMENT
1-NRV-164	R20880-03	1	REPAIR/REPLACEMENT
1-OME-1 Lower Internals	C38602-07	1	REPLACEMENT
1-PP-45-3	R51617-06	1	REPLACEMENT
1-QMO-200	C28166-02 16	1	REPLACEMENT
1-QMO-201	C28167-02	1	REPLACEMENT
1-QMO-225	C28393-11	2	REPLACEMENT
1-QMO-226	C28397-02 11	2	REPLACEMENT
1-QRV-61	R36016-02	1	REPLACEMENT
1-QRV-62	R36017-02	1	REPLACEMENT
1-QT-506	R35549-17	2	REPLACEMENT
1-RC-108-L2	C40235-01	1	REPAIR
1-RC-111-L1	C38052-04	1	REPLACEMENT
1-RH-128E	C34037-01	2	REPLACEMENT
1-SI-130	C24126-03	2	REPAIR

INDIANA MICHIGAN POWER COMPANY
DONALD C. COOK NUCLEAR PLANT, BRIDGMAN MI.
UNIT 1 COMMERCIAL SERVICE DATE 8-23-1975
NATIONAL BOARD NUMBER 20761

1-SI-158-L1	C40031-04	1	REPLACEMENT
1-SI-158-L2	C40545-04	1	REPLACEMENT
1-SI-158-L3	C40547-04	1	REPLACEMENT
1-SI-161-L4	C39895-03	1	REPLACEMENT
1-SI-166-3	R52451-03	1	REPLACEMENT
1-SI-166-L1	C39888-03	1	REPLACEMENT
1-SI-169-L4	C33542-02	2	REPAIR
1-SI-170-L3	R49141-04	1	REPLACEMENT
1-SV-1A-3	R59811-01	2	REPLACEMENT
1-SV-1B-2	R61019-01	2	REPLACEMENT
1-SV-2A-1	R62233-01	2	REPLACEMENT
1-SV-2A-3	C39824-01	2	REPLACEMENT
1-SV-2B-2	R56435-01	2	REPLACEMENT
1-SV-2B-3	R61020-01	2	REPLACEMENT
1-SV-45A	R52036-16	1	REPLACEMENT
1-SV-45B	R52037-16	1	REPLACEMENT
1-SV-45C	R52239-16	1	REPLACEMENT
1-TK-33	R22763-06	2	REPLACEMENT
1-WCR-933	C13407-01	2	REPLACEMENT

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

NIS-2-127-97

1. Owner AMERICAN ELECTRIC POWER COMPANY Date 04/09/97
 Name
P.O. Box 60 Fort Wayne, IN 46801 Sheet 1 of 2
 Address
2. Plant D.C. COOK NUCLEAR POWER PLANT Unit # 1
 Name
One Cook Place, Bridgman, MI 49106 C28393-14 R/R1392
 Address Repair Org. P.O. No., Job No., etc
3. Work Performed by Installation Services Type Code Symbol Stamp N/A
 Name Authorization No. N/A
Same as #2 Expiration Date N/A
 Address
4. Identification of System CHEMICAL AND VOLUME CONTROL ASME CODE CLASS 2
5. (a) Applicable Const. Code ANSI B31.1 19 83Ed., N/A Add. N/A Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1989
6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manuf. Serial No.	Nat. Board No.	Other Identification	Year Built	Repaired Replaced or Replacement	ASME Code Stamped (Yes or No)
1-ACS-R930 W4 x 13 BEAM	ENERGY STEEL & SUPPLY CO.	ASP# 16150	N/A	A36 (90)	N/A	REPLACE- MENT	NO
1-ACS-R930 3 x 3 x 3/8 CS ANGLE	CONSOLIDATED POWER SUPPLY	ASP# 15971	N/A	A36 (90)	N/A	REPLACE- MENT	NO
1-ACS-R930 3/8 x 2 CS U-BOLT	NPS INDUSTRIES	ASP# 13582	N/A	SA36	N/A	REPLACE- MENT	NO

7. Description of Work MODIFIED SUPPORT 1-ACS-R930 TO ACCOMMODATE REDESIGNED PIPING PER MM-526

8. Test Conducted: Hydrostatic ☐ Pneumatic ☐ Nominal Operating Pressure ☐
 N/A ☒ Other ☐ Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided
 (1) size is 8-1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks

Applicable Manufacturer's Data Reports to be attached

Ref. JO: C28393-14

DESIGN CHANGE: MM-526

REPAIR/REPLACEMENT TRAVELLER: RR95-1392

ISI CLASS: 2

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this REPLACEMENT conforms to the rules of the ASME Code, Section XI.
repair or replacement

Type Code Symbol Stamp N/ACertificate of Authorization No. N/A Expiration Date N/A

Signed Wayne Cretsinger, Senior Planner
Owner or Owner's Designee, Title

Date APRIL 10, 1997

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Michigan and employed by ARKWRIGHT MUTUAL INS. CO. of WALTHAM, MASS. have inspected the components described in this Owner's Report during the period 6/23/95 to 4/14/97, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

MR. Matusz Commissions MICH 0055, ENR NSE
Inspector's Signature National Board, State, Province, Endorsements

Date April 14 1997

1. Owner AMERICAN ELECTRIC POWER COMPANY Date 04/04/97
 Name
P.O. Box 60 Fort Wayne, IN 46801 Sheet 1 of 2
 Address
2. Plant D.C. COOK NUCLEAR POWER PLANT Unit # 1
 Name
One Cook Place, Bridgman, MI 49106 R51519-11
 Address Repair Org. P.O. No., Job No., etc
3. Work Performed by Installation Services Type Code Symbol Stamp N/A
 Name Authorization No. N/A
Same as #2 Expiration Date N/A
 Address
4. Identification of System CONTAINMENT AIRLOCK ASME CODE CLASS MC
5. (a) Applicable Const. Code N/A 19 Ed., N/A Add. N/A Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1989
6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manuf. Serial No.	Nat. Board No.	Other Identification	Year Built	Repaired Replaced or Replacement	ASME Code Stamped (Yes or No)
CTMT AIRLOCK							
5/8" H.H. NUT	NOVA	ASP# 17915	N/A	SA194 GR.2H	N/A	REPLACE-MENT	NO
5/8" C.S. WASHER	ACCUTECH	ASP# 15215	N/A	F436	N/A	REPLACE-MENT	NO

7. Description of Work REPLACED 5/8" NUT AND WASHER THAT WERE UNUSABLE

8. Test Conducted: Hydrostatic ☐ Pneumatic ☐ Nominal Operating Pressure ☐
 N/A ☒ Other ☐ Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

Remarks

Applicable Manufacturer's Data Reports to be attached

Ref. JO: R51519-11 DESIGN CHANGE: N/A ISI CLASS: MC

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this REPLACEMENT conforms to the rules of the ASME Code, Section XI.
repair or replacement

Type Code Symbol Stamp N/ACertificate of Authorization No. N/A Expiration Date N/A

Signed Wayne Cretsinger, Senior Planner
Owner or Owner's Designee, Title

Date APRIL 10, 1997

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Michigan and employed by ARKWRIGHT MUTUAL INS. CO. of WALTHAM, MASS. have inspected the components described in this Owner's Report during the period 4/4/97 to 4/14/97, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

M. H. Matusz Commissions M-20055, END N&I
Inspector's Signature National Board, State, Province, Endorsements

Date April 14 1997

NIS-2-123-97

1. Owner AMERICAN ELECTRIC POWER COMPANY Date 04/04/97
Name
P.O. Box 60 Fort Wayne, IN 46801 Sheet 1 of 2
Address
2. Plant D.C. COOK NUCLEAR POWER PLANT Unit # 1
Name
One Cook Place, Bridgman, MI 49106 R51519-09
Address Repair Org. P.O. No., Job No., etc
3. Work Performed by Installation Services Type Code Symbol Stamp N/A
Name Authorization No. N/A
Same as #2 Expiration Date N/A
Address
4. Identification of System CONTAINMENT EQUIPMENT HATCH ASME CODE CLASS MC
5. (a) Applicable Const. Code N/A 19 Ed. N/A Add. N/A Code Case
(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1989
6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component CTMT. EQUIP. HATCH	Name of Manufacturer	Manuf. Serial No.	Nat. Board No.	Other Identification	Year Built	Repaired Replaced or Replacement	ASME Code Stamped (Yes or No)
1 1/4" C.S. BOLT	ACCUTECH	ASP# 9861	N/A	SA193 GR.B7	N/A	REPLACE-MENT	NO
1 1/4" C.S. WASHER	ACCUTECH	ASP# 15609	N/A	F436	N/A	REPLACE-MENT	NO

7. Description of Work REPLACED 1 1/4" BOLT AND WASHER THAT WERE DAMAGED DURING REMOVAL.

8. Test Conducted: Hydrostatic ☐ Pneumatic ☐ Nominal Operating Pressure ☐
N/A ☒ Other ☐ Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks

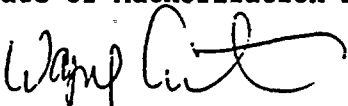
Applicable Manufacturer's Data Reports to be attached

Ref. JO: R51519-09 DESIGN CHANGE: N/A ISI CLASS: MC

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this
REPLACEMENT conforms to the rules of the ASME Code, Section XI.
repair or replacement

Type Code Symbol Stamp N/ACertificate of Authorization No. N/A Expiration Date N/A



Signed Wayne Cretsinger, Senior Planner
Owner or Owner's Designee, Title

Date APRIL 10, 19 97

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Michigan and employed by ARKWRIGHT MUTUAL INS. CO. of WALTHAM, MASS. have inspected the components described in this Owner's Report during the period 4/4/97 to 4/14/97, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.


Inspector's Signature

Commissions MICH 0055, END NFI
National Board, State, Province, Endorsements

Date April 14 19 97

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS

As Required by the Provisions of the ASME Code Section XI

NIS-2-184-97

1. Owner AMERICAN ELECTRIC POWER COMPANY Date 05/01/97
 Name
P.O. Box 60 Fort Wayne, IN 46801 Sheet 1 of 3
 Address
2. Plant D.C. COOK NUCLEAR POWER PLANT Unit # 1
 Name
One Cook Place, Bridgman, MI 49106 C34216-01 R/R1521 THRU 1524
 Address Repair Org. P.O. No., Job No., etc
3. Work Performed by Installation Services Type Code Symbol Stamp N/A
 Name Authorization No. N/A
Same as #2 Expiration Date N/A
 Address
4. Identification of System REACTOR COOLANT ASME CODE CLASS 1
5. (a) Applicable Const. Code ANSI B31.1 19 83Ed., N/A Add. N/A Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1989

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manuf. Serial No.	Nat. Board No.	Other Identification	Year Built	Repaired Replaced or Replacement	ASME Code Stamped (Yes or No)
1-ARC-R4000 3 x 3 x 3/8 CS ANGLE	M&E# 30-153032	ASP# 18722	N/A	A36 (90)	N/A	REPLACE- MENT	NO
1-ARC-R5300 3 x 3 x 1/4 TUBE STEEL	M&E# 30-150169	ASP# 18441	N/A	A500 GR B (90)	N/A	REPLACE- MENT	NO
1-ARC-R5300 1/4 SS PLATE	M&E# 30-153223	ASP# 19108	N/A	A240 TP 304 (93B)	N/A	REPLACE- MENT	NO
1-ARC-R5300 3 x 3 x 3/8 CS ANGLE	M&E# 30-153032	ASP# 18722	N/A	A36 (90)	N/A	REPLACE- MENT	NO
1-ARC-R5300 4 x 4 x 1/4 TUBE STEEL	M&E# 30-153269	ASP# 17641	N/A	A500 GR B (93)	N/A	REPLACE- MENT	NO
1-ARC-R5300 1/4 CS PLATE	M&E# 30-153133	ASP# 18784	N/A	A36 (91)	N/A	REPLACE- MENT	NO

7. Description of Work MODIFIED SUPPORTS 1-ARC-R4000, R5300, R5301 AND R5302
PER DCP-0046.

8. Test Conducted: Hydrostatic ☐ Pneumatic ☐ Nominal Operating Pressure ☐
 N/A ☒ Other ☐ Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS

As Required by the Provisions of the ASME Code Section XI

NIS-2-184-97

1. Owner AMERICAN ELECTRIC POWER COMPANY Date 05/01/97
 Name
P.O. Box 60 Fort Wayne, IN 46801 Sheet 2 of 3
 Address
2. Plant D.C. COOK NUCLEAR POWER PLANT Unit # 1
 Name
One Cook Place, Bridgman, MI 49106 C34216-01 R/R1521 THRU 1524
 Address Repair Org. P.O. No., Job No., etc
3. Work Performed by Installation Services Type Code Symbol Stamp N/A
 Name Authorization No. N/A
Same as #2 Expiration Date N/A
 Address
4. Identification of System REACTOR COOLANT ASME CODE CLASS 1
5. (a) Applicable Const. Code ANSI B31.1 19 83Ed., N/A Add. N/A Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1989

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manuf. Serial No.	Nat. Board No.	Other Identification	Year Built	Repaired Replaced or Replacement	ASME Code Stamped (Yes or No)
1-ARC-R5300 1/4 CS PLATE	M&E# 30-153132	ASP# 18884	N/A	A36 (90)	N/A	REPLACE- MENT	NO
1-ARC-R5301 1/2 CS PLATE	M&E# 30-153188	ASP# 19061	N/A	A36 (93A)	N/A	REPLACE- MENT	NO
1-ARC-R5301 3 x 3 x 1/4 TUBE STEEL	M&E# 30-150169	ASP# 18441	N/A	A500 GR B (90)	N/A	REPLACE- MENT	NO
1-ARC-R5302 2 1/2 x 2 1/2 x 1/4 TUBE STEEL	M&E# 30-150167	ASP# 12087	N/A	A500 GR B (90)	N/A	REPLACE- MENT	NO
1-ARC-R5302 1/4 CS PLATE	M&E# 30-153133	ASP# 18784	N/A	A36 (91)	N/A	REPLACE- MENT	NO
1-ARC-R5302 2 x 2 x 1/4 CS ANGLE	M&E# 30-153012	ASP# 18841	N/A	A36 (90)	N/A	REPLACE- MENT	NO

7. Description of Work MODIFIED SUPPORTS 1-ARC-R4000, R5300, R5301 AND R5302
PER DCP-0046.

8. Test Conducted: Hydrostatic ☐ Pneumatic ☐ Nominal Operating Pressure ☐
 N/A ☒ Other ☐ Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

3. Remarks

Applicable Manufacturer's Data Reports to be attached

Ref. JO: C34216-01 DESIGN CHANGE: DCP-0046

REPAIR/REPLACEMENT TRAVELER: RR96-1521 ISI CLASS: 1

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this REPLACEMENT conforms to the rules of the ASME Code, Section XI. repair or replacement

Type Code Symbol Stamp N/ACertificate of Authorization No. N/A Expiration Date N/A

Signed Wayne Cretsinger, Senior Planner
Owner or Owner's Designee, Title

Date MAY 01, 19 97

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Michigan and employed by ARKWRIGHT MUTUAL INS. CO. of WALTHAM, MASS. have inspected the components described in this Owner's Report during the period 5/3/96 to 5/15/97, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Inspector's Signature

Commissions Mich 0055, ENDRIT
National Board, State, Province, Endorsements

Date May 15 19 97

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

NIS-2-192-97

1. Owner AMERICAN ELECTRIC POWER COMPANY Date 05/05/97
 P.O. Box 60 Fort Wayne, IN 46801 Sheet 1 of 2
 Address.
2. Plant D.C. COOK NUCLEAR POWER PLANT Unit # 1
 One Cook Place, Bridgman, MI 49106 C34216-07 R/R1493
 Address Repair Org. P.O. No., Job No., etc
 Installation Services Type Code Symbol Stamp N/A
 Same as #2 Authorization No. N/A
 Address Expiration Date N/A
4. Identification of System SAFETY INJECTION ASME CODE CLASS 2
5. (a) Applicable Const. Code ANSI B31.1 19 83Ed., N/A Add. N/A Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1989

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manuf. Serial No.	Nat. Board No.	Other Identification	Year Built	Repaired Replaced or Replacement	ASME Code Stamped (Yes or No)
1-ASI-R5301 3 x 3 x 1/4 TUBE STEEL	M&E# 30-150169	ASP# 18441	N/A	A500 GR B (90)	N/A	REPLACE- MENT	NO
1-ASI-R5301 2 x 2 x 1/4 CS ANGLE	M&E# 30-153012	ASP# 18841	N/A	A36 (90)	N/A	REPLACE- MENT	NO
1-ASI-R5301 1/2 x 7 HILTI BOLT	M&E# 30-046271	ASP# 18649	N/A	HILTI KB II 12-7	N/A	REPLACE- MENT	NO
1-ASI-R5301 3/4 CS PLATE	M&E# 30-153195	ASP# 15663	N/A	A36 (93A)	N/A	REPLACE- MENT	NO

7. Description of Work INSTALLED SUPPORT 1-ASI-R5301 PER DCP-0046.

8. Test Conducted: Hydrostatic ☐ Pneumatic ☐ Nominal Operating Pressure ☐
 N/A ☒ Other ☐ Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

Remarks

Applicable Manufacturer's Data Reports to be attached

Ref. JO: C34216-07

DESIGN CHANGE: DCP-0046

REPAIR/REPLACEMENT TRAVELER: RR96-1488A

ISI CLASS: 2

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this REPLACEMENT conforms to the rules of the ASME Code, Section XI.
repair or replacement

Type Code Symbol Stamp N/ACertificate of Authorization No. N/A Expiration Date N/A

Signed Wayne Cretsinger, Senior Planner
Owner or Owner's Designee, Title

Date MAY 05, 19 97

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Michigan and employed by ARKWRIGHT MUTUAL INS. CO. of WALTHAM, MASS. have inspected the components described in this Owner's Report during the period 2-19-97 to 5-29-97, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Inspector's Signature

Commissions

MICHIGAN, END NFI
National Board, State, Province, Endorsements

Date MAY 29, 19 97

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner AMERICAN ELECTRIC POWER Date 03-23-97
Name
P.O. Box 60 Fort Wayne, IN 46801 Sheet 1 of 2
Address
2. Plant D.C. COOK NUCLEAR POWER PLANT Unit # 1
Name
One Cook Place, Bridgman, MI 49106 C35012-3& 12, 1-CCM-432
Address Repair Org. P.O. No., Job No., etc
3. Work Performed by Maintenance Department Type Code Symbol Stamp N/A
Name Authorization No. N/A
Same as #2 Expiration Date N/A
Address
4. Identification of System Component Cooling Water
5. (a) Applicable Const. Code ANSI B31.1 19 83Ed., N/A Add. N/A Code Case
(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1989,
No Addenda

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manuf. Serial No.	Nat. Board No.	Other Identification	Year Built	Repaired Replaced or Replacement	ASME Code Stamped (Yes or No)
1-CCM-432							
Yoke SA-182 F316				M&E# 30-035793 ASP#18275	n/a	replace	NO
support SA-36				existing	n/a	repair	NO

7. Description of Work (See Remarks)
8. Test Conducted: Hydrostatic _____ Pneumatic _____ Nominal Operating Pressure _____
N/A _____ Other VT-2 Pressure _____ psi Test Temp. _____ °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks Replaced Yoke assembly and burred/ground out section of
Applicable Manufacturer's Data Reports to be attached

adjacent support for removal of screwed type Yoke

with no welding performed on hanger work

Ref. JO:C35012-3.12 File: 1-CCM-432

ISI Class: 2

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this
replacement/repair conforms to the rules of the ASME Code, Section XI.
 repair or replacement

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed Frank Pisarsky Date 4/9, 1977
 Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Michigan and employed by ARKWRIGHT MUTUAL INS. CO.* of NORWOOD, MASS. have inspected the components described in this Owner's Report during the period 3-5-96 to 4-10-97, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

M. R. McIntosh Commissions Mich BOSS, ENGINEER
 Inspector's Signature National Board, State, Province, Endorsements

Date April 10 19 97

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner AMERICAN ELECTRIC POWER Date 03-20-97
Name
P.O. Box 60 Fort Wayne, IN 46801 Sheet 1 of 2
Address
2. Plant D.C. COOK NUCLEAR POWER PLANT Unit # 1
Name
One Cook Place, Bridgman, MI 49106 C34552-3, 1-CCM-433, vlv.
Address Repair Org. P.O. No., Job No., etc
3. Work Performed by Maintenance Department Type Code Symbol Stamp N/A
Name Authorization No. N/A
Same as #2 Expiration Date N/A
Address
4. Identification of System Component Cooling Water
5. (a) Applicable Const. Code ANSI B31.1 19 83Ed., N/A Add. N/A Code Case
(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1989
No Addenda

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manuf. Serial No.	Nat. Board No.	Other Identification	Year Built	Repaired Replaced or Replacement	ASME Code Stamped (Yes or No)
1-CCM-433							
Yoke assy SA-182 F316	Conval			M&E# 30-035333 ASP#18110	n/a	replace	NO

7. Description of Work (See Remarks)
8. Test Conducted: Hydrostatic _____ Pneumatic _____ Nominal Operating Pressure _____
N/A _____ Other VT-2 Pressure _____ psi Test Temp. _____ °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks Mechanically replaced the yoke assembly
 Applicable Manufacturer's Data Reports to be attached

Ref. JO:C34552-3 File: 1-CCM-433 ISI Class: 2

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this
replacement conforms to the rules of the ASME Code, Section XI.
 repair or replacement

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed Frank Pisarsky, Maint. Eng. Supervisor Date 7/3, 1997
 Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Michigan and employed by ARKWRIGHT MUTUAL INS. CO.* of NORWOOD, MASS. have inspected the components described in this Owner's Report during the period 1-30-96 to 7-15-97, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

MT Mutual Commissions Michigan, End NBT
 Inspector's Signature National Board, State, Province, Endorsements

Date July 15 19 97

~~FACTORY MUTUAL SYSTEM~~

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner AMERICAN ELECTRIC POWER Date 04-08-97
Name _____
P.O. Box 60 Fort Wayne, IN 46801 Sheet 1 of 2
Address _____
2. Plant D.C. COOK NUCLEAR POWER PLANT Unit _____ # 1
Name _____
One Cook Place, Bridgman, MI 49106 C40400-01, 1-CCM-451, vlv.
Address _____ Repair Org. P.O. No., Job No., etc _____
3. Work Performed by Maintenance Department Type Code Symbol Stamp N/A
Name _____ Authorization No. N/A
Same as #2 Expiration Date N/A
Address _____
4. Identification of System _____ Component Cooling Water
5. (a) Applicable Const. Code ANSI B31.1 19 83Ed., N/A Add. N/A Code Case _____
(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1989,
No Addenda

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manuf. Serial No.	Nat. Board No.	Other Identification	Year Built	Repaired Replaced or Replacement	ASME Code Stamped (Yes or No)
1-CCM-451							
Bolts SA-307 Grade B				P.O.# 66224- 042-7 ASP#19650	n/a	replace	NO

7. Description of Work _____ (See Remarks)
8. Test Conducted: Hydrostatic _____ Pneumatic _____ Nominal Operating Pressure _____
N/A ☒ Other _____ Pressure _____ psi Test Temp. _____ °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks Replaced the flange bolting for the mechanically installed
Applicable Manufacturer's Data Reports to be attached
valve due to dimensional and identification purposes.

Ref. JO:C40400-01 File: 1-CCM-451 ISI Class: 2

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this
replacement conforms to the rules of the ASME Code, Section XI.
 repair or replacement

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed Frank Pisarsky for Maint. Eng. Supervisor Date 6/22, 1997
 Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Michigan and employed by ARKWRIGHT MUTUAL INS. CO.* of NORWOOD, MASS. have inspected the components described in this Owner's Report during the period 4-7-97 to 6-27-97, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

M. J. Heston Commissions MICH 0055, END N&T
 Inspector's Signature National Board, State, Province, Endorsements

Date JUNE 22 1997

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner AMERICAN ELECTRIC POWER Date 04-08-97
Name _____
- P.O. Box 60 Fort Wayne, IN 46801 Sheet 1 of 2
Address _____
2. Plant D.C. COOK NUCLEAR POWER PLANT Unit _____ # 1
Name _____
- One Cook Place, Bridgman, MI 49106 C40401-01, 1-CCM-452, vlv.
Address _____ Repair Org. P.O. No., Job No., etc _____
3. Work Performed by Maintenance Department Type Code Symbol Stamp N/A
Name _____ Authorization No. N/A
Same as #2 Expiration Date N/A
Address _____
4. Identification of System _____ Component Cooling Water
5. (a) Applicable Const. Code ANSI B31.1 19 83Ed., N/A Add. N/A Code Case _____
(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1989, No Addenda
6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manuf. Serial No.	Nat. Board No.	Other Identification	Year Built	Repaired Replaced or Replacement	ASME Code Stamped (Yes or No)
1-CCM-452							
Bolts SA-307 Grade B				P.O.# 66224- 042-7 ASP#19650	n/a	replace	NO

7. Description of Work (See Remarks)
8. Test Conducted: Hydrostatic _____ Pneumatic _____ Nominal Operating Pressure _____
N/A ☒ Other _____ Pressure _____ psi Test Temp. _____ °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks Replaced the flange bolting for the mechanically installed
Applicable Manufacturer's Data Reports to be attached
valve due to dimensional and identification purposes.

Ref. JO:C40401-01 File: 1-CCM-452 ISI Class: 2

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this
replacement conforms to the rules of the ASME Code, Section XI.
 repair or replacement

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed Frank Pisarsky, Maint. Eng. Supervisor Date 6/22, 1997
 Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Michigan and employed by ARKWRIGHT MUTUAL INS. CO.* of NORWOOD, MASS. have inspected the components described in this Owner's Report during the period 4/7/97 to 6/27/97, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

MR. Waterhouse
 Inspector's Signature

Commissions MR. LOOS, END N.E.I.
 National Board, State, Province, Endorsements

Date JUNE 27 19 97

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS

As Required by the Provisions of the ASME Code Section XI

NIS-2-137-97

1. Owner AMERICAN ELECTRIC POWER COMPANY Date 04/09/97
P.O. Box 60 Fort Wayne, IN 46801 Sheet 1 of 2
Address
2. Plant D.C. COOK NUCLEAR POWER PLANT Unit # 1
One Cook Place, Bridgman, MI 49106 R57215-04
Address Repair Org. P.O. No., Job No., etc
3. Work Performed by Installation Services Type Code Symbol Stamp N/A
Same as #2 Name Authorization No. N/A
Address Expiration Date N/A
4. Identification of System CHEMICAL AND VOLUME CONTROL ASME CODE CLASS 2
5. (a) Applicable Const. Code ANSI B31.1 19 83Ed., N/A Add. N-416-1 Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1989
6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manuf. Serial No.	Nat. Board No.	Other Identification	Year Built	Repaired Replaced or Replacement	ASME Code Stamped (Yes or No)
1-CS-292 2" SCH 40 PIPE	RADNOR ALLOYS INC.	ASP# 12420	N/A	SA312 TP 304	N/A	REPLACE- MENT	NO
1-CS-292 2" 1500# SS CHECK VALVE	CONVAL INC.	ASP# 18264	N/A	SA182 F 316	N/A	REPLACE- MENT	NO
1-CS-292 2" 3000# SS COUPLING	COMSOLIDATED POWER SUPPLY	ASP# 18795	N/A	SA182 F 304	N/A	REPLACE- MENT	NO

7. Description of Work REPLACE CHECK VALVE 1-CS-292 PER MM404

8. Test Conducted: Hydrostatic ☐ Pneumatic ☐ Nominal Operating Pressure ☒
 N/A ☐ Other ☐ Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

Remarks

Applicable Manufacturer's Data Reports to be attached

Ref. JO: R57215-04 DESIGN CHANGE: MM-404

REPAIR/REPLACEMENT TRAVELLER: R57215-04 ISI CLASS: 2

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this
REPLACEMENT conforms to the rules of the ASME Code, Section XI.
 repair or replacement

Type Code Symbol Stamp N/ACertificate of Authorization No. N/A Expiration Date N/A

Signed Wayne Cretsinger, Senior Planner
 Owner or Owner's Designee, Title

Date APRIL 10, 19 97

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Michigan and employed by ARKWRIGHT MUTUAL INS. CO. of WALTHAM, MASS. have inspected the components described in this Owner's Report during the period 2/27/97 to 4/16/97, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Inspector's Signature

Commissions

MICH 0055, END N.E.I.

National Board, State, Province, Endorsements

Date April 16, 19 97

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS

As Required by the Provisions of the ASME Code Section XI

1. Owner AMERICAN ELECTRIC POWER Date 03-18-97
 Name
P.O. Box 60 Fort Wayne, IN 46801 Sheet 1 of 2
 Address
2. Plant D.C. COOK NUCLEAR POWER PLANT Unit # 1
 Name
One Cook Place, Bridgman, MI 49106 R51459-02, 1- CS-325, vlv
 Address Repair Org. P.O. No., Job No., etc
3. Work Performed by Maintenance Department Type Code Symbol Stamp N/A
 Name Authorization No. N/A
Same as #2 Expiration Date N/A
 Address
4. Identification of System Chemical Volume & Control (CVCS)
5. (a) Applicable Const. Code ANSI B31.1 19 83Ed., N/A Add. N/A Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1989,
No Addenda
6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manuf. Serial No.	Nat. Board No.	Other Identification	Year Built	Repaired Replaced or Replacement	ASME Code Stamped (Yes or No)
1-CS-325							
Disc Piston SA-479 Nitronic 60	Conval			30-035495 ASP# 7528	n/a	replace	NO

7. Description of Work (See Remarks)
8. Test Conducted: Hydrostatic _____ Pneumatic _____ Nominal Operating Pressure _____
 N/A ☒ Other _____ Pressure _____ psi Test Temp. _____ °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks Mechanically replaced the valve's disc piston
 Applicable Manufacturer's Data Reports to be attached

Ref. JO:R51459-02 File: 1-CS-325 ISI Class: 1

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this
replacement conforms to the rules of the ASME Code, Section XI.
 repair or replacement

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed Frank Pisarsky Maint. Eng. Supervisor Date 6/22, 1997
 Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Michigan and employed by ARKWRIGHT MUTUAL INS. CO.* of NORWOOD, MASS. have inspected the components described in this Owner's Report during the period 3-18-97 to 6-27-97, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

M. J. Waters Commissions Mich 0055, ENJ NIT
 Inspector's Signature National Board, State, Province, Endorsements

Date JUNE 27 1997



FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner AMERICAN ELECTRIC POWER Date 03-20-97
Name
P.O. Box 60 Fort Wayne, IN 46801 Sheet 1 of 2
Address
2. Plant D.C. COOK NUCLEAR POWER PLANT Unit # 1
Name
One Cook Place, Bridgman, MI 49106 C33791-01, 1-CS-442-1, vlv.
Address Repair Org. P.O. No., Job No., etc
3. Work Performed by Maintenance Department Type Code Symbol Stamp N/A
Name Authorization No. N/A
Same as #2 Expiration Date N/A
Address
4. Identification of System Reactor Coolant system
5. (a) Applicable Const. Code ANSI B31.1 19 83Ed., N/A Add. N/A Code Case
(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1989
No Addenda
6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manuf. Serial No.	Nat. Board No.	Other Identification	Year Built	Repaired Replaced or Replacement	ASME Code Stamped (Yes or No)
1-CS-442-1							
Bonnet: SA-479 tp316				M&E#: 30035335 ASP# 312	n/a	replace	NO

7. Description of Work (See Remarks)
8. Test Conducted: Hydrostatic _____ Pneumatic _____ Nominal Operating Pressure _____
N/A _____ Other VT-2 Pressure _____ psi Test Temp. _____ °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks Replaced the valve's bonnet
Applicable Manufacturer's Data Reports to be attached

Ref. JO:C33791-1 File: 1-CS-442-1 ISI Class: 2

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this
replacement conforms to the rules of the ASME Code, Section XI.
 repair or replacement

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed Frank Pisarsky, Maint. Eng. Supervisor Date 6/30, 19 97
 Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Michigan and employed by ARKWRIGHT MUTUAL INS. CO.* of NORWOOD, MASS. have inspected the components described in this Owner's Report during the period 3-20-97 to 7-3-97, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Mr. M. H. M. H. M. H. Commissions MICHOSS, END NIT
 Inspector's Signature National Board, State, Province, Endorsements

Date July 3, 19 97

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner AMERICAN ELECTRIC POWER Date 03-23-97
Name
P.O. Box 60 Fort Wayne, IN 46801 Sheet 1 of 2
Address
2. Plant D.C. COOK NUCLEAR POWER PLANT Unit # 1
Name
One Cook Place, Bridgman, MI 49106 C38539-1, 1-DCR-206, vlv
Address Repair Org. P.O. No., Job No., etc
3. Work Performed by Maintenance Department Type Code Symbol Stamp N/A
Name Authorization No. N/A
Same as #2 Expiration Date N/A
Address
4. Identification of System Reactor Coolant
5. (a) Applicable Const. Code ANSI B31.1 19 83Ed., N/A Add. N/A Code Case
(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1989,
No Addenda

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manuf. Serial No.	Nat. Board No.	Other Identification	Year Built	Repaired Replaced or Replacement	ASME Code Stamped (Yes or No)
1=DCR-206							
Nut 1/2" SA-194 gr.2H				M&E#: 30-046621 ASP#13381	n/a	replace	NO

7. Description of Work (See Remarks)
8. Test Conducted: Hydrostatic _____ Pneumatic _____ Nominal Operating Pressure _____
N/A ☒ Other _____ Pressure _____ psi Test Temp. _____ °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks Replaced existing body to bonnet standard size hex nut
Applicable Manufacturer's Data Reports to be attached
with new heavy hex nut.

Ref. JO:C38539-1 File: 1-DCR-206 ISI Class: 2

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this
replacement conforms to the rules of the ASME Code, Section XI.
 repair or replacement

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed Frank Pisarsky, Maint. Eng. Supervisor Date 4/9, 19 97
 Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Michigan and employed by ARKWRIGHT MUTUAL INS. CO.* of NORWOOD, MASS. have inspected the components described in this Owner's Report during the period 4/2/97 to 4/10/97, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Mike Matzenga Commissions Michigan, ENR NFE
 Inspector's Signature National Board, State, Province, Endorsements

Date April 10 19 97



FORM 115-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner AMERICAN ELECTRIC POWER Date 04-24-97
 Name _____
P.O. Box 60 Fort Wayne, IN 46801 Sheet 1 of 2
 Address _____
2. Plant D.C. COOK NUCLEAR POWER PLANT Unit _____ # 1
 Name _____
One Cook Place, Bridgman, MI 49106 C40743-02, 1-DCR-310, Vlv.
 Address _____ Repair Org. P.O. No., Job No., etc _____
3. Work Performed by Maintenance Department Type Code Symbol Stamp N/A
 Name _____ Authorization No. N/A
Same as #2 Expiration Date N/A
 Address _____
4. Identification of System _____ Steam generator blowdown
5. (a) Applicable Const. Code ANSI B31.1 19 83Ed., N/A Add. N/A Code Case _____
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1989,
No Addenda
6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manuf. Serial No.	Nat. Board No.	Other Identification	Year Built	Repaired Replaced or Replacement	ASME Code Stamped (Yes or No)
1-DCR-310							
Bonnet SA-182 gr.F5A				30-040710 ASP#17154	n/a	replace	NO

7. Description of Work _____ (See Remarks)
8. Test Conducted: Hydrostatic _____ Pneumatic _____ Nominal Operating Pressure _____
 N/A _____ Other VT-2 Pressure _____ psi Test Temp. _____ °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks Mechanically replace the valve's bonnet
Applicable Manufacturer's Data Reports to be attached

Ref. JO:C40743-02 File: 1-DCR-310 ISI Class: 2

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this
replacement conforms to the rules of the ASME Code, Section XI.
 repair or replacement

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed Frank Pisarsky, Maint. Eng. Supervisor Date 6/22, 19 77
 Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Michigan and employed by ARKWRIGHT MUTUAL INS. CO.* of NORWOOD, MASS. have inspected the components described in this Owner's Report during the period 4/23/77 to 6/27/77, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Mr. Motasough Commissions Mich 0055, GND N.E.T.
 Inspector's Signature National Board, State, Province, Endorsements

Date June 27 19 77

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner INDIANA MICHIGAN POWER COMPANY Date 03-17-97
Name
P.O. Box 60 Fort Wayne, IN 46801 Sheet 1 of 2
Address
2. Plant D.C. COOK NUCLEAR POWER PLANT Unit # 1
Name
One Cook Place, Bridgman, MI 49106 R61017-3, 1-FW-118-3 vlv.
Address Repair Org. P.O. No., Job No., etc
3. Work Performed by Maintenance Department Type Code Symbol Stamp N/A
Name Authorization No. N/A
Same as #2 Expiration Date N/A
Address
4. Identification of System Feedwater System
5. (a) Applicable Const. Code ANSI B31.1 19 83Ed., N/A Add. N/A Code Case
(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1989,
No Addenda

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manuf. Serial No.	Nat. Board No.	Other Identification	Year Built	Repaired or Replacement	ASME Code Stamped (Yes or No)
1-FW-118-3							
Nuts SA-194 gr.2H				M&E#: 30-211970 ASP#18093 & 19173	n/a	replace	NO

7. Description of Work (See Remarks)
8. Test Conducted: Hydrostatic _____ Pneumatic _____ Nominal Operating Pressure _____
N/A ☒ Other _____ Pressure _____ psi Test Temp. _____ °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.



FORM NIS-2 (Back)

9. Remarks Replaced 3 nuts to the valve's bonnets studs
Applicable Manufacturer's Data Reports to be attached

Ref. JO:R61017-3 File: 1-FW-118-3 ISI Class: 2

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this
replacement conforms to the rules of the ASME Code, Section XI.
 repair or replacement

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed Frank Pisarsky, Maint. Eng. Supervisor Date 4/3, 1997
 Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Michigan and employed by ARKWRIGHT MUTUAL INS. CO.* of NORWOOD, MASS. have inspected the components described in this Owner's Report during the period 3-16-97 to 7-3-97, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Michael Waterhouse Commissions Mich. Code, ENCL NIT
 Inspector's Signature National Board, State, Province, Endorsements

~~*FACTORY MUTUAL SYSTEM~~

Date July 3 1997

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner AMERICAN ELECTRIC POWER Date 03-11-97
Name
P.O. Box 60 Fort Wayne, IN 46801 Sheet 1 of 2
Address
2. Plant D.C. COOK NUCLEAR POWER PLANT Unit # 1
Name
One Cook Place, Bridgman, MI 49106 R20806-06, 1-FW-132-3 vlv.
Address Repair Org. P.O. No., Job No., etc
3. Work Performed by Maintenance Department Type Code Symbol Stamp N/A
Name Authorization No. N/A
Same as #2 Expiration Date N/A
Address
4. Identification of System Auxiliary Feedwater
5. (a) Applicable Const. Code ANSI B31.1 19 83Ed., N/A Add. N/A Code Case
(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1989,
No Addenda

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manuf. Serial No.	Nat. Board No.	Other Identification	Year Built	Repaired Replaced or Replacement	ASME Code Stamped (Yes or No)
1-FW-132-3							
Anti rotation lugs:SA-36 Disc:SA-515 gr.70				N/A(see remarks)	n/a	install	NO

7. Description of Work (See Remarks)
8. Test Conducted: Hydrostatic _____ Pneumatic _____ Nominal Operating Pressure _____
N/A _____ Other VT/MT Pressure _____ psi Test Temp. _____ °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.



FORM NIS-2 (Back)

9. Remarks Installed Anti rotation lugs on valve disc by welding. Per
Applicable Manufacturer's Data Reports to be attached

Engineering Direction, the material of the Anti rotation lugs
may be Standard/Commercial grade therefore, no ASP# available

Ref. JO:R20806-06 File: 1-FW-132-3 ISI Class: 2

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this
replacement conforms to the rules of the ASME Code, Section XI.
 repair or replacement

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed Frank Pisarsky, Maint. Eng. Supervisor Date 4/24, 1997
 Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Michigan and employed by ARKWRIGHT-MUTUAL INS. CO.* of NORWOOD, MASS. have inspected the components described in this Owner's Report during the period 1/30/96 to 5/27/97, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

MR. Matusz Commissions Mich 0055, ENR NFI
 Inspector's Signature National Board, State, Province, Endorsements

Date May 27 1997

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner AMERICAN ELECTRIC POWER Date 03-19-97
Name _____
P.O. Box 60 Fort Wayne, IN 46801 Sheet 1 of 2
Address _____
2. Plant D.C. COOK NUCLEAR POWER PLANT Unit _____ # 1
Name _____
One Cook Place, Bridgman, MI 49106 C33453-1, 1-FW-S9, Snubber
Address _____ Repair Org. P.O. No., Job No., etc _____
3. Work Performed by Maintenance Department Type Code Symbol Stamp N/A
Name _____ Authorization No. N/A
Same as #2 Expiration Date N/A
Address _____
4. Identification of System Feedwater
5. (a) Applicable Const. Code ANSI B31.1 19 83Ed., N/A Add. N/A Code Case _____
(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1989,
No Addenda
6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manuf. Serial No.	Nat. Board No.	Other Identification	Year Built	Repaired Replaced or Replacement	ASME Code Stamped (Yes or No)
1-FW-S9							
Config 'A' kit	Grinnell			M&E#: 30040551 ASP#19456	n/a	replace	NO

7. Description of Work (See Remarks)
8. Test Conducted: Hydrostatic _____ Pneumatic _____ Nominal Operating Pressure _____
N/A _____ Other VT-3/4 Pressure _____ psi Test Temp. _____ °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks Installed Configuration 'A' kit per MM300
Applicable Manufacturer's Data Reports to be attached

This work involved three job orders: C33453-1, R55962-1, R55965-1

Ref. JO: C33453-1 File: 1-FW-S9 ISI Class: 2

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this
replacement conforms to the rules of the ASME Code, Section XI.
 repair or replacement

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed Frank Pisarsky, Maint. Eng. Supervisor Date 7/2, 1997
 Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Michigan and employed by ARKWRIGHT MUTUAL INS. CO.* of NORWOOD, MASS. have inspected the components described in this Owner's Report during the period 2-9-96 to 2-16-97, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

M. J. Whitcomb
 Inspector's Signature

Commissions MICH, MASS, END, NFI
 National Board, State, Province, Endorsements

Date July 16 1997

1. Owner AMERICAN ELECTRIC POWER Date 03-17-97

Name

P.O. Box 60 Fort Wayne, IN 46801 Sheet 1 of 2

Address
2. Plant D.C. COOK NUCLEAR POWER PLANT Unit # 1

Name

One Cook Place, Bridgman, MI 49106 C33456-1, 1-GBD-S563, snubber

Address

Repair Org. P.O. No., Job No., etc
3. Work Performed by Maintenance Department Type Code Symbol Stamp N/A

Name

Authorization No. N/A

Same as #2 Expiration Date N/A

Address
4. Identification of System Blowdown
5. (a) Applicable Const. Code ANSI B31.1 19 83Ed., N/A Add. N/A Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1989,
No Addenda

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manuf. Serial No.	Nat. Board No.	Other Identification	Year Built	Repaired Replaced or Replacement	ASME Code Stamped (Yes or No)
1-GBD-S563							
Config 'A' Kit	Grinnell			M&E# 30040550 ASP#15697	n/a	replace	NO
Load Pin	Grinnell			M&E# 30024975 ASP#10364	n/a	replace	NO

7. Description of Work (See Remarks)
8. Test Conducted: Hydrostatic _____ Pneumatic _____ Nominal Operating Pressure _____
N/A Other VT-3/4 Pressure _____ psi Test Temp. _____ °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks Installed Configuration 'A' kit per MM300 and tapered load pin
Applicable Manufacturer's Data Reports to be attached

This work involved work on three job orders: R19744-1, R19743-1
and C33456-1

Ref. JO:C33456-1 File: 1-GBD-S563 ISI Class: 2

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this
replacement conforms to the rules of the ASME Code, Section XI.
 repair or replacement

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed Frank Pisarsky, Maint. Eng. Supervisor Date 7/8, 1977
 Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Michigan and employed by ARKWRIGHT MUTUAL INS. CO.* of NORWOOD, MASS. have inspected the components described in this Owner's Report during the period 2/9/96 to 7/15/97, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

W. H. H. H. H. H. Commissions MICH 0055, END N.E.
 Inspector's Signature National Board, State, Province, Endorsements

Date July 15 1997

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS

As Required by the Provisions of the ASME Code Section XI

NIS-2-126-97

1. Owner AMERICAN ELECTRIC POWER COMPANY Date 04/09/97
 Name
P.O. Box 60 Fort Wayne, IN 46801 Sheet 1 of 2
 Address
2. Plant D.C. COOK NUCLEAR POWER PLANT Unit # 1
 Name
One Cook Place, Bridgman, MI 49106 C38695-01 R/R1617
 Address Repair Org. P.O. No., Job No., etc
3. Work Performed by Installation Services Type Code Symbol Stamp N/A
 Name Authorization No. N/A
Same as #2 Expiration Date N/A
 Address
4. Identification of System COMPONENT COOLING WATER ASME CODE CLASS 2
5. (a) Applicable Const. Code ANSI B31.1 19 83Ed., N/A Add. N/A Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1989
6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manuf. Serial No.	Nat. Board No.	Other Identification	Year Built	Repaired Replaced or Replacement	ASME Code Stamped (Yes or No)
1-GCCW-R241 7/8" C.S. PLATE	DUBOSE NATIONAL ENERGY	ASP# 17466	N/A	A36 (93A)	N/A	REPLACE- MENT	NO

7. Description of Work REPLACED SLIDE PLATE ON SUPPORT 1-GCCW-R241
8. Test Conducted: Hydrostatic ☐ Pneumatic ☐ Nominal Operating Pressure ☐
 N/A ☒ Other ☐ Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks

Applicable Manufacturer's Data Reports to be attached

Ref. JO: C38695-01

DESIGN CHANGE: N/A

REPAIR/REPLACEMENT TRAVELLER: RR96-1617

ISI CLASS: 2

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this REPLACEMENT conforms to the rules of the ASME Code, Section XI.
repair or replacement

Type Code Symbol Stamp N/ACertificate of Authorization No. N/A Expiration Date N/A

Signed Wayne Cretsinger, Senior Planner
Owner or Owner's Designee, Title

Date APRIL 10, 1997

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Michigan and employed by ARKWRIGHT MUTUAL INS. CO. of WALTHAM, MASS. have inspected the components described in this Owner's Report during the period 12/10/96 to 4/14/97, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

M. H. Waters, Jr. Commissions MICH 0055, ENR N&I
Inspector's Signature National Board, State, Province, Endorsements

Date April 14 1997

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

NIS-2-180-97

1. Owner AMERICAN ELECTRIC POWER Date 05/01/97
Name
P.O. Box 60 Fort Wayne, IN 46801 Sheet 1 of 2
Address
2. Plant D.C. COOK NUCLEAR POWER PLANT Unit # 1
Name
One Cook Place, Bridgman, MI 49106 C34216-01 R/R1512
Address Repair Org. P.O. No., Job No., etc
3. Work Performed by Installation Services Type Code Symbol Stamp N/A
Name Authorization No. N/A
Same as #2 Expiration Date N/A
Address
4. Identification of System CHEMICAL AND VOLUME CONTROL ASME CODE CLASS 1
5. (a) Applicable Const. Code ANSI B31.1 19 83Ed., N/A Add. N/A Code Case
(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1989
6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manuf. Serial No.	Nat. Board No.	Other Identification	Year Built	Repaired Replaced or Replacement	ASME Code Stamped (Yes or No)
1-GCS-R505 3/4 CS PLATE	M&E# 30-153172	ASP# 16215	N/A	A36 (91)	N/A	Replace- ment	NO
1-GCS-R505 5/8 CS PLATE	M&E# 30-153194	ASP# 16017	N/A	A36 (93)	N/A	Replace- ment	NO
1-GCS-R505 1/2 X 2 BOLT	M&E# 30-211245	ASP# 15215	N/A	SA193 B7	N/A	Replace- ment	NO
1-GCS-R505 1/2 CS NUT	M&E# 30-046621	ASP# 13381	N/A	SA194 2H	N/A	Replace- ment	NO

7. Description of Work MODIFIED SUPPORT 1-GCS-R505 PER DCP-0046

8. Test Conducted: Hydrostatic ☐ Pneumatic ☐ Nominal Operating Pressure ☐
N/A ☒ Other ☐ Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in. x 1 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

Remarks

Applicable Manufacturer's Data Reports to be attached

Ref. JO: C34216-01

DESIGN CHANGE: DCP-0046

REPAIR/REPLACEMENT TRAVELLER: RR96-1512

ISI CLASS: 1

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this REPLACEMENT conforms to the rules of the ASME Code, Section XI.
 repair or replacement

Type Code Symbol Stamp N/ACertificate of Authorization No. N/A Expiration Date N/A

Signed Wayne Cretsinger, Senior Planner
 Owner or Owner's Designee, Title

Date MAY 01, 1997

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of MICHIGAN and employed by ARKWRIGHT MUTUAL INS. CO. of WALTHAM, MASS. have inspected the components described in this Owner's Report during the period 4/26/96 to 5/14/97, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Inspector's Signature

Commissions

Mich 0655, ENJ NIT
 National Board, State, Province, Endorsements
Date May 14, 1997

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

NIS-2-188-97

1. Owner AMERICAN ELECTRIC POWER COMPANY Date 05/02/97
 Name
P.O. Box 60 Fort Wayne, IN 46801 Sheet 1 of 3
 Address
2. Plant D.C. COOK NUCLEAR POWER PLANT Unit # 1
 Name
One Cook Place, Bridgman, MI 49106 R/R1471, 1472,
 Address C34216-07 R/R1476, 1477 AND 1485
 Repair Org. P.O. No., Job No., etc
3. Work Performed by Installation Services Type Code Symbol Stamp N/A
 Name
Same as #2 Authorization No. N/A
 Address Expiration Date N/A
4. Identification of System CHEMICAL AND VOLUME CONTROL ASME CODE CLASS 1
5. (a) Applicable Const. Code ANSI B31.1 19 83Ed., N/A Add. N/A Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1989

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manuf. Serial No.	Nat. Board No.	Other Identification	Year Built	Repaired Replaced or Replacement	ASME Code Stamped (Yes or No)
1-GCS-R531 FILLER METAL	M&E# N/A	ASP# 16497	N/A	ER308	N/A	REPLACE- MENT	NO
1-GCS-R534 3/8 SS PLATE	M&E# 30-153224	ASP# 9509	N/A	A240 TP 304 (90)	N/A	REPLACE- MENT	NO
1-GCS-R534 1/4 CS PLATE	M&E# 30-153132	ASP# 18884	N/A	A36 (90)	N/A	REPLACE- MENT	NO
1-GCS-R534 1/2 x 1 1/2 BOLT	M&E# 30-211240	ASP# 15841	N/A	SA193 B7	N/A	REPLACE- MENT	NO
1-GCS-R534 1/2 CS NUT	M&E# 30-046621	ASP# 13381	N/A	SA194 2H	N/A	REPLACE- MENT	NO
1-GCS-R626 1/2 SS PLATE	M&E# 30-153226	ASP# 9509	N/A	A240 TP 304 (91A)	N/A	REPLACE- MENT	NO

7. Description of Work MODIFIED SUPPORTS 1-GCS-R531, R534, R626, R631 AND
1-ACS-R4004 PER DCP-0046.

8. Test Conducted: Hydrostatic ☐ Pneumatic ☐ Nominal Operating Pressure ☐
 N/A ☒ Other ☐ Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

NIS-2-188-97

1. Owner AMERICAN ELECTRIC POWER COMPANY Date 05/02/97
 P.O. Box 60 Fort Wayne, IN 46801 Sheet 2 of 3
 Name Address
 2. Plant D.C. COOK NUCLEAR POWER PLANT Unit # 1
 One Cook Place, Bridgman, MI 49106 R/R1471, 1472,
 Name Address C34216-07 R/R1476, 1477 AND 1485
 Repair Org. P.O. No., Job No., etc
 3. Work Performed by Installation Services Type Code Symbol Stamp N/A
 Same as #2 Name Authorization No. N/A
 Address Expiration Date N/A
 4. Identification of System CHEMICAL AND VOLUME CONTROL ASME CODE CLASS 1
 5. (a) Applicable Const. Code ANSI B31.1 19 83Ed., N/A Add. N/A Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1989

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manuf. Serial No.	Nat. Board No.	Other Identification	Year Built	Repaired Replaced or Replacement	ASME Code Stamped (Yes or No)
1-GCS-R631 3/8 CS PLATE	M&E# 30-153143	ASP# 17466	N/A	A36 (93A)	N/A	REPLACE- MENT	NO
1-GCS-R631 3/8 x 2 U-BOLT	M&E# 30-024112	ASP# 13582	N/A	FIG #137N	N/A	REPLACE- MENT	NO
1-GCS-R631 3/8 WASHER	M&E# 30-212670	ASP# 18430	N/A	F436	N/A	REPLACE- MENT	NO
1-GCS-R631 5/8 CS PLATE	M&E# 30-153194	ASP# 16017	N/A	A36 (93)	N/A	REPLACE- MENT	NO
1-ACS-R4004 3 x 3 x 1/4 TUBE STEEL	M&E# 30-150169	ASP# 18929	N/A	A500 GR B (90)	N/A	REPLACE- MENT	NO
1-ACS-R4004 1/4 CS PLATE	M&E# 30-153132	ASP# 18884	N/A	A36 (90)	N/A	REPLACE- MENT	NO

7. Description of Work MODIFIED SUPPORTS 1-GCS-R531, R534, R626, R631 AND
1-ACS-R4004 PER DCP-0046.

8. Test Conducted: Hydrostatic ☐ Pneumatic ☐ Nominal Operating Pressure ☐
 N/A ☒ Other ☐ Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

Remarks

Applicable Manufacturer's Data Reports to be attached

Ref. JO: C34216-07

DESIGN CHANGE: DCP-0046

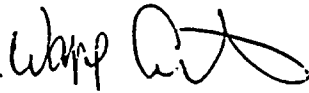
REPAIR/REPLACEMENT TRAVELER: RR96-1471

ISI CLASS: 2

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this
REPLACEMENT conforms to the rules of the ASME Code, Section XI.
repair or replacement

Type Code Symbol Stamp N/ACertificate of Authorization No. N/A Expiration Date N/A


Signed Wayne Cretsinger, Senior Planner
Owner or Owner's Designee, Title

Date MAY 28, 1997

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Michigan and employed by ARKWRIGHT MUTUAL INS. CO. of WALTHAM, MASS. have inspected the components described in this Owner's Report during the period 4-26-96 to 5-29-97, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.


Inspector's Signature

Commissions MICHIGAN, END NIT
National Board, State, Province, Endorsements

Date MAY 29, 1997

NIS-2-130-97

Owner AMERICAN ELECTRIC POWER COMPANY

Date 04/09/97

Name

P.O. Box 60 Fort Wayne, IN 46801

Sheet 1 of 2

Address

2. Plant D.C. COOK NUCLEAR POWER PLANT

Unit # 1

Name

One Cook Place, Bridgman, MI 49106

C33899-02 R/R 1500

Address

Repair Org. P.O. No., Job No., etc

3. Work Performed by Installation Services

Type Code Symbol Stamp N/A

Name

Authorization No. N/A

Same as #2

Expiration Date N/A

Address

4. Identification of System CHEMICAL AND VOLUME CONTROL ASME CODE CLASS 1

5. (a) Applicable Const. Code ANSI B31.1 19 83Ed., N/A Add. N/A Code Case

(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1989

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manuf. Serial No.	Nat. Board No.	Other Identification	Year Built	Repaired Replaced or Replacement	ASME Code Stamped (Yes or No)
1-GCS-R533 1/4 CS PLATE	ENERGY & PROCESS CORP	ASP# 16925	N/A	A36 (91)	N/A	REPLACE-MENT	NO

7. Description of Work ADDED STIFFENERS TO SUPPORT 1-GCS-R533

8. Test Conducted: Hydrostatic ☐ Pneumatic ☐ Nominal Operating Pressure ☐

N/A ☒ Other ☐ Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

Remarks

Applicable Manufacturer's Data Reports to be attached

Ref. JO: C33899-02

DESIGN CHANGE: N/A

REPAIR/REPLACEMENT TRAVELLER: RR96-1500

ISI CLASS: 2

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this REPLACEMENT conforms to the rules of the ASME Code, Section XI.
repair or replacement

Type Code Symbol Stamp N/ACertificate of Authorization No. N/A Expiration Date N/A

Signed Wayne Cretsinger, Senior Planner
Owner or Owner's Designee, Title

Date APRIL 10, 1997

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Michigan and employed by ARKWRIGHT MUTUAL INS. CO. of WALTHAM, MASS. have inspected the components described in this Owner's Report during the period 4/3/96 to 4/17/97, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Inspector's Signature

Commissions

McL0055, EN&NET
National Board, State, Province, EndorsementsDate April 17, 1997

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

NIS-2-167-97

1. Owner AMERICAN ELECTRIC POWER COMPANY Date 04/18/97
 Name
P.O. Box 60 Fort Wayne, IN 46801 Sheet 1 of 2
 Address
2. Plant D.C. COOK NUCLEAR POWER PLANT Unit # 1
 Name
One Cook Place, Bridgman, MI 49106 C39026-02
 Address Repair Org. P.O. No., Job No., etc
3. Work Performed by Installation Services Type Code Symbol Stamp N/A
 Name Authorization No. N/A
Same as #2 Expiration Date N/A
 Address

4. Identification of System MAIN STEAM ASME CODE CLASS 2
5. (a) Applicable Const. Code ANSI B31.1 19 83Ed., N/A Add. N/A Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1989
6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manuf. Serial No.	Nat. Board No.	Other Identification	Year Built	Repaired Replaced or Replacement	ASME Code Stamped (Yes or No)
1-GMS-L801 2 x 2 x 1/4 CS ANGLE	M&E# 30-153012	ASP# 18841	N/A	A36 (90)	N/A	REPLACE- MENT	NO
1-GMS-L801 3/4 CS PLATE	M&E# 30-153195	ASP# 15663	N/A	A36 (93A)	N/A	REPLACE- MENT	NO

7. Description of Work MODIFIED SUPPORT 1-GMS-L801 TO CORRECT CLEARANCE GAP.

8. Test Conducted: Hydrostatic ☐ Pneumatic ☐ Nominal Operating Pressure ☐
 N/A ☒ Other ☐ Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks

Applicable Manufacturer's Data Reports to be attached

Ref. JO: C39026-02

DESIGN CHANGE: N/A

ISI CLASS: 2

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this REPLACEMENT conforms to the rules of the ASME Code, Section XI. repair or replacement

Type Code Symbol Stamp N/ACertificate of Authorization No. N/A Expiration Date N/A

Signed Wayne Cretsinger, Senior Planner
Owner or Owner's Designee, Title

Date APRIL 18, 1997

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Michigan and employed by ARKWRIGHT MUTUAL INS. CO. of WALTHAM, MASS. have inspected the components described in this Owner's Report during the period 3-10-97 to 5-5-97, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Inspector's Signature

Commissions

Mich 0055, END NIT

National Board, State, Province, Endorsements

Date May 5 19 97

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS

As Required by the Provisions of the ASME Code Section XI

NIS-2-182-97

1. Owner AMERICAN ELECTRIC POWER COMPANY Date 05/01/97
P.O. Box 60 Fort Wayne, IN 46801 Sheet 1 of 3
 Name Address
2. Plant D.C. COOK NUCLEAR POWER PLANT Unit # 1
One Cook Place, Bridgman, MI 49106 C34216-01 R/R1515, 1517 AND 1518
 Name Address Repair Org. P.O. No., Job No., etc
3. Work Performed by Installation Services Type Code Symbol Stamp N/A
Same as #2 Name Authorization No. N/A
 Address Expiration Date N/A
4. Identification of System REACTOR COOLANT ASME CODE CLASS 1
5. (a) Applicable Const. Code ANSI B31.1 19 83Ed., N/A Add. N/A Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1989
6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manuf. Serial No.	Nat. Board No.	Other Identification	Year Built	Repaired Replaced or Replacement	ASME Code Stamped (Yes or No)
1-GRC-R501 1/4 CS PLATE	M&E# 30-153133	ASP# 18784	N/A	A36 (91)	N/A	REPLACE- MENT	NO
1-GRC-R501 5/8 x 2 BOLT	M&E# 30-048070	ASP# 13404	N/A	A307 GR A (89)	N/A	REPLACE- MENT	NO
1-GRC-R501 5/8 CS NUT	M&E# 30-211880	ASP# 17915	N/A	SA194 2H	N/A	REPLACE- MENT	NO
1-GRC-R524 3/4 x 3/4 x 3/8 CS ANGLE	M&E# 30-153033	ASP# 12784	N/A	A36 (91)	N/A	REPLACE- MENT	NO
1-GRC-R524 1/2 x 2 BOLT	M&E# 30-211245	ASP# 15215	N/A	SA193 B7	N/A	REPLACE- MENT	NO
1-GRC-R524 1/2 CS NUT	M&E# 30-046621	ASP# 13381	N/A	SA194 2H	N/A	REPLACE- MENT	NO

7. Description of Work MODIFIED SUPPORTS 1-GRC-R501, R524 AND R534 PER DCP-0046.

8. Test Conducted: Hydrostatic ☐ Pneumatic ☐ Nominal Operating Pressure ☐
 N/A ☒ Other ☐ Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI.

NIS-2-182-97

1. Owner AMERICAN ELECTRIC POWER COMPANY Date 05/01/97
 Name
 P.O. Box 60 Fort Wayne, IN 46801 Sheet 2 of 3
 Address
2. Plant D.C. COOK NUCLEAR POWER PLANT Unit # 1
 Name
 One Cook Place, Bridgman, MI 49106 C34216-01 R/R1515, 1517 AND 1518
 Address Repair Org. P.O. No., Job No., etc
 One Cook Place, Bridgman, MI 49106 Type Code Symbol Stamp N/A
 Name Authorization No. N/A
 Same as #2 Expiration Date N/A
 Address
4. Identification of System REACTOR COOLANT ASME CODE CLASS 1
5. (a) Applicable Const. Code ANSI B31.1 19 83Ed. N/A Add. N/A Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1989
6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manuf. Serial No.	Nat. Board No.	Other Identification	Year Built	Repaired Replaced or Replacement	ASME Code Stamped (Yes or No)
1-GRC-R524 1/4 CS PLATE	M&E# 30-153133	ASP# 18784	N/A	A36 (91)	N/A	REPLACE- MENT	NO
1-GRC-R534 1/4 CS PLATE	M&E# 30-153133	ASP# 18784	N/A	A36 (91)	N/A	REPLACE- MENT	NO
1-GRC-R534 1/2 x 2 BOLT	M&E# 30-211245	ASP# 15215	N/A	SA193 B7	N/A	REPLACE- MENT	NO
1-GRC-R534 1/2 CS NUT	M&E# 30-046621	ASP# 13381	N/A	SA194 2H	N/A	REPLACE- MENT	NO
1-GRC-R534 5/8 x 2 BOLT	M&E# 30-048070	ASP# 13404	N/A	A307 GR B (89)	N/A	REPLACE- MENT	NO
1-GRC-R534 5/8 CS NUT	M&E# 30-211880	ASP# 17915	N/A	SA194 2H	N/A	REPLACE- MENT	NO

7. Description of Work MODIFIED SUPPORTS 1-GRC-R501, R524 AND R534 PER DCP-0046.

8. Test Conducted: Hydrostatic ☐ Pneumatic ☐ Nominal Operating Pressure ☐
 N/A ☒ Other ☐ Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

Remarks

Applicable Manufacturer's Data Reports to be attached

Ref. JO: C34216-01

DESIGN CHANGE: DCP-0046

REPAIR/REPLACEMENT TRAVELER: RR96-1515

ISI CLASS: 1

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this REPLACEMENT conforms to the rules of the ASME Code, Section XI. repair or replacement

Type Code Symbol Stamp N/ACertificate of Authorization No. N/A Expiration Date N/A

Signed Wayne Cretsinger, Senior Planner
Owner or Owner's Designee, Title

Date MAY 01, 19 97

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Michigan and employed by ARKWRIGHT MUTUAL INS. CO. of WALTHAM, MASS. have inspected the components described in this Owner's Report during the period 4/26/96 to 5/15/97, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Inspector's Signature

Commissions

Mich 0055, END NFI

National Board, State, Province, Endorsements

Date May 15, 19 97

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

NIS-2-131-97

1. Owner AMERICAN ELECTRIC POWER COMPANY Date 04/09/97
 Name
P.O. Box 60 Fort Wayne, IN 46801 Sheet 1 of 2
 Address
2. Plant D.C. COOK NUCLEAR POWER PLANT Unit # 1
 Name
One Cook Place, Bridgman, MI 49106 C33899-01 R/R 1501
 Address Repair Org. P.O. No., Job No., etc
3. Work Performed by Installation Services Type Code Symbol Stamp N/A
 Name
Same as #2 Authorization No. N/A
 Address Expiration Date N/A
4. Identification of System REACTOR COOLANT ASME CODE CLASS 1
5. (a) Applicable Const. Code ANSI B31.1 19 83Ed. N/A Add. N/A Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1989
6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manuf. Serial No.	Nat. Board No.	Other Identification	Year Built	Repaired Replaced or Replacement	ASME Code Stamped (Yes or No)
1-GRC-R503 1/4 CS PLATE	CONSOLIDATED POWER SUPPLY	ASP# 18884	N/A	A36 (90)	N/A	REPLACE- MENT	NO
1-GRC-R503 2 x 2 x 3/8 CS ANGLE	ENERGY & PROCESS CORP.	ASP# 11779	N/A	A36 (90)	N/A	REPLACE- MENT	NO

7. Description of Work ADDED STIFFENERS AND ANGLES TO SUPPORT 1-GRC-R503

8. Test Conducted: Hydrostatic ☐ Pneumatic ☐ Nominal Operating Pressure ☐
 N/A ☒ Other ☐ Pressure N/A psi. Test Temp. N/A °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided
 (1) size is 8-1/2 in. x 11 in., (2) information in items 1 through 6 on this report is
 included on each sheet, and (3) each sheet is numbered and the number of sheets is
 recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks

Applicable Manufacturer's Data Reports to be attached

Ref. JO: C33899-01

DESIGN CHANGE: N/A

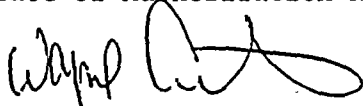
REPAIR/REPLACEMENT TRAVELLER: RR96-1501

ISI CLASS: 1

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this REPLACEMENT conforms to the rules of the ASME Code, Section XI.
repair or replacement

Type Code Symbol Stamp N/ACertificate of Authorization No. N/A Expiration Date N/A


Signed Wayne Cretsinger, Senior Planner
Owner or Owner's Designee, Title

Date APRIL 10, 19 97

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Michigan and employed by ARKWRIGHT MUTUAL INS. CO. of WALTHAM, MASS. have inspected the components described in this Owner's Report during the period 4-3-96 to 4-15-97, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.


Inspector's Signature

Commissions MICH 0055, END N&I
National Board, State, Province, Endorsements

Date April 15, 19 97

NIS-2-129-97

1. Owner AMERICAN ELECTRIC POWER COMPANY Date 04/09/97
 Name
P.O. Box 60 Fort Wayne, IN 46801 Sheet 1 of 2
 Address
2. Plant D.C. COOK NUCLEAR POWER PLANT Unit # 1
 Name
One Cook Place, Bridgman, MI 49106 C34891-02
 Address Repair Org. P.O. No., Job No., etc
3. Work Performed by Installation Services Type Code Symbol Stamp N/A
 Name Authorization No. N/A
Same as #2 Expiration Date N/A
 Address
4. Identification of System REACTOR COOLANT ASME CODE CLASS 1
5. (a) Applicable Const. Code ANSI B31.1 19 83Ed., N/A Add. N/A Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1989
6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manuf. Serial No.	Nat. Board No.	Other Identification	Year Built	Repaired Replaced or Replacement	ASME Code Stamped (Yes or No)
1-GRC-R558 1/8 CS PLATE	CONSOLIDATED POWER SUPPLY	ASP# 16652	N/A	A570 GR45 (92)	N/A	REPLACE- MENT	NO

7. Description of Work ADDED SHIM TO SUPPORT 1-GRC-R558 FOR GAP

8. Test Conducted: Hydrostatic ☐ Pneumatic ☐ Nominal Operating Pressure ☐

N/A ☒ Other ☐ Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks

Applicable Manufacturer's Data Reports to be attached

Ref. JO: C34891-02 DESIGN CHANGE: N/A

REPAIR/REPLACEMENT TRAVELLER: C34891-02 ISI CLASS: 1

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this
REPLACEMENT conforms to the rules of the ASME Code, Section XI.
 repair or replacement

Type Code Symbol Stamp N/ACertificate of Authorization No. N/A Expiration Date N/A

Signed Wayne Cretsinger, Senior Planner
 Owner or Owner's Designee, Title

Date APRIL 10, 1997

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Michigan and employed by ARKWRIGHT MUTUAL INS. CO. of WALTHAM, MASS. have inspected the components described in this Owner's Report during the period 2-21-97 to 4-15-97, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

MA Mutsaers Commissions MICH, OOS, END NFI
 Inspector's Signature National Board, State, Province, Endorsements

Date April 15, 1997

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

NIS-2-186-97

1. Owner AMERICAN ELECTRIC POWER COMPANY Date 05/01/97
Name
P.O. Box 60 Fort Wayne, IN 46801 Sheet 1 of 2
Address
2. Plant D.C. COOK NUCLEAR POWER PLANT Unit # 1
Name
One Cook Place, Bridgman, MI 49106 C34216-01
Address Repair Org. P.O. No., Job No., etc
3. Work Performed by Installation Services Type Code Symbol Stamp N/A
Name
Same as #2 Authorization No. N/A
Address Expiration Date N/A
4. Identification of System REACTOR COOLANT ASME CODE CLASS 1
5. (a) Applicable Const. Code ANSI B31.1 19 83Ed., N/A Add. N/A Code Case
(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1989

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manuf. Serial No.	Nat. Board No.	Other Identification	Year Built	Repaired Replaced or Replacement	ASME Code Stamped (Yes or No)
1-GRC-R660 1/2 x 2 BOLT	M&E# 30-211245	ASP# 15215	N/A	SA193 B7	N/A	REPLACE- MENT	NO
1-GRC-R622 1/2 CS NUT	M&E# 30-046621	ASP# 13381	N/A	SA194 2H	N/A	REPLACE- MENT	NO

7. Description of Work MODIFIED SUPPORT 1-GRC-R622 PER DCP-0046.

8. Test Conducted: Hydrostatic ☐ Pneumatic ☐ Nominal Operating Pressure ☐
N/A ☒ Other ☐ Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks

Applicable Manufacturer's Data Reports to be attached

Ref. JO: C34216-01

DESIGN CHANGE: DCP-0046

REPAIR/REPLACEMENT TRAVELER: C34216-01

ISI CLASS: 1

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this
REPLACEMENT conforms to the rules of the ASME Code, Section XI.
 repair or replacement

Type Code Symbol Stamp N/ACertificate of Authorization No. N/A Expiration Date N/A

Signed Wayne Cretsinger, Senior Planner
 Owner or Owner's Designee, Title

Date MAY 01, 19 97

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Michigan and employed by ARKWRIGHT MUTUAL INS. CO. of WALTHAM, MASS. have inspected the components described in this Owner's Report during the period 3/23/97 to 5/15/97, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Inspector's Signature

Commissions MICHIGAN, ENDPN

National Board, State, Province, Endorsements

Date May 15, 19 97

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner AMERICAN ELECTRIC POWER Date 03-20-97
Name
P.O. Box 60 Fort Wayne, IN 46801 Sheet 1 of 2
Address
2. Plant D.C. COOK NUCLEAR POWER PLANT Unit # 1
Name
One Cook Place, Bridgman, MI 49106 C33446-1, 1-GRC-S566, snubber
Address Repair Org. P.O. No., Job No., etc
3. Work Performed by Maintenance Department Type Code Symbol Stamp N/A
Name Authorization No. N/A
Same as #2 Expiration Date N/A
Address
4. Identification of System Reactor Coolant
5. (a) Applicable Const. Code ANSI B31.1 19 83Ed., N/A Add. N/A Code Case
(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1989,
No Addenda
6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manuf. Serial No.	Nat. Board No.	Other Identification	Year Built	Repaired Replaced or Replacement	ASME Code Stamped (Yes or No)
1-GRC-S566							
Config 'A' kit	Grinnell			N&E# 30040551 ASP#17059	n/a	replace	NO
Load Pin	Grinnell			M&E# 30024977 ASP#15148	n/a	replace	NO

7. Description of Work (See Remarks)
8. Test Conducted: Hydrostatic _____ Pneumatic _____ Nominal Operating Pressure _____
N/A _____ Other VT-3/4 Pressure _____ psi Test Temp. _____ °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.



FORM NIS-2 (Back)

9. Remarks Installed Configuration 'A' kit per MM300 and tapered loak pin
Applicable Manufacturer's Data Reports to be attached

This work involved three job orders: C33446-1, R55967-1, R22648-1

Ref. JO: C33446-01 File: 1-GRC-S566 ISI Class: 1

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this
replacement conforms to the rules of the ASME Code, Section XI.
 repair or replacement

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed Frank Pisarsky, Maint. Eng. Supervisor Date 7/8, 19 97
 Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Michigan and employed by ARKWRIGHT MUTUAL INS. CO.* of NORWOOD, MASS. have inspected the components described in this Owner's Report during the period 2-9-96 to 2-16-97, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Mike Whitcomb Commissioned Michigan, ENDR
 Inspector's Signature National Board, State, Province, Endorsements

Date July 16 19 97

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner AMERICAN ELECTRIC POWER Date 03-17-97
Name
P.O. Box 60 Fort Wayne, IN 46801 Sheet 1 of 2
Address
2. Plant D.C. COOK NUCLEAR POWER PLANT Unit # 1
Name
One Cook Place, Bridgman, MI 49106 C33452-1, 1-GRC-S604, snubber
Address Repair Org. P.O. No., Job No., etc
3. Work Performed by Maintenance Department Type Code Symbol Stamp N/A
Name Authorization No. N/A
Same as #2 Expiration Date N/A
Address
4. Identification of System Reactor Coolant
5. (a) Applicable Const. Code ANSI B31.1 19 83Ed., N/A Add. N/A Code Case
(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1989, No Addenda

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manuf. Serial No.	Nat. Board No.	Other Identification	Year Built	Repaired Replaced or Replacement	ASME Code Stamped (Yes or No)
1-GRC-S604							
Config 'A' kit	Grinnell			M&E# 30040550 ASP#15697	n/a	replace	NO

7. Description of Work (See Remarks)
8. Test Conducted: Hydrostatic _____ Pneumatic _____ Nominal Operating Pressure _____
N/A _____ Other VT-3/4 Pressure _____ psi Test Temp. _____ °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks Installed a configuration 'A' kit per MM300
Applicable Manufacturer's Data Reports to be attached

This work involved the following job orders:

C33452-1, R55478-1, R55477-1

Ref. JO:C33452-1 File: 1-GRC-S604 ISI Class: 1

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this
replacement conforms to the rules of the ASME Code, Section XI.
 repair or replacement

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed Frank Pisarsky Maint. Eng. Supervisor Date 7/8, 1997
 Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Michigan and employed by ARKWRIGHT MUTUAL INS. CO.* of NORWOOD, MASS. have inspected the components described in this Owner's Report during the period 1-30-96 to 7-16-97, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Mr. M. M. M. M. M.
 Inspector's Signature

Commissions Michigan, END NET
 National Board, State, Province, Endorsements

Date July 16 1997

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

NIS-2-183-97

1. Owner AMERICAN ELECTRIC POWER Date 05/01/97
 Name
P.O. Box 60 Fort Wayne, IN 46801 Sheet 1 of 2
 Address
2. Plant D.C. COOK NUCLEAR POWER PLANT Unit # 1
 Name
One Cook Place, Bridgman, MI 49106 C34216-01 R/R1519
 Address Repair Org. P.O. No., Job No., etc
3. Work Performed by Installation Services Type Code Symbol Stamp N/A
 Name Authorization No. N/A
Same as #2 Expiration Date N/A
 Address
4. Identification of System REACTOR COOLANT ASME CODE CLASS 1
5. (a) Applicable Const. Code ANSI B31.1 19 83Ed. "N/A" Add. N/A Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1989
6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manuf. Serial No.	Nat. Board No.	Other Identification	Year Built	Repaired Replaced or Replacement	ASME Code Stamped (Yes or No)
1-GRC-V550 1/4 SS PLATE	M&E# 30-153223	ASP# 19108	N/A	A240 TP 304 (93B)	N/A	Replace- ment	NO
1-GRC-V550 1/4 x 1/4 U-BOLT	M&E# 30-024104	ASP# 16895	N/A	FIG 137N	N/A	Replace- ment	NO
1-GRC-V550 1/8 CS PLATE	M&E# 30-153125	ASP# 16652	N/A	A570 GR45 (92)	N/A	Replace- ment	NO
1-GRC-V550 1/4 CS PLATE	M&E# 30-153132	ASP# 18884	N/A	A36 (90)	N/A	Replace- ment	NO

7. Description of Work MODIFIED SUPPORT 1-GRC-V550 PER DCP-0046

8. Test Conducted: Hydrostatic ☐ Pneumatic ☐ Nominal Operating Pressure ☐
 N/A ☒ Other ☐ Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

Remarks

Applicable Manufacturer's Data Reports to be attached

Ref. JO: C34216-01

DESIGN CHANGE: DCP-0046

REPAIR/REPLACEMENT TRAVELLER: RR96-1515

ISI CLASS: 1

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this REPLACEMENT conforms to the rules of the ASME Code, Section XI. repair or replacement

Type Code Symbol Stamp N/ACertificate of Authorization No. N/A Expiration Date N/A

Signed Wayne Cretsinger, Senior Planner
Owner or Owner's Designee, Title

Date MAY 01, 19 97

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of MICHIGAN and employed by ARKWRIGHT MUTUAL INS. CO. of WALTHAM, MASS. have inspected the components described in this Owner's Report during the period 4/26/96 to 5/13/97, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Inspector's Signature

Commissions

Mich 0055, END N&I
National Board, State, Province, Endorsements
Date May 15 19 97

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS

As Required by the Provisions of the ASME Code Section XI

1. Owner INDIANA MICHIGAN POWER COMPANY Date 01-08-96
 Name
P.O. Box 60 Fort Wayne, IN 46801 Sheet 1 of 2
 Address
2. Plant D.C. COOK NUCLEAR POWER PLANT Unit # 1
 Name
One Cook Place, Bridgman, MI 49106 C33638-1 1-GRH-L-51
 Address Repair Org. P.O. No., Job No., etc
3. Work Performed by Maintenance Department Type Code Symbol Stamp N/A
 Name Authorization No. N/A
Same as #2 Expiration Date N/A
 Address
4. Identification of System Residual Heat Removal system
5. (a) Applicable Const. Code ANSI B31.1 19 83Ed., N/A Add. N/A Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1983 Ed.
Summer Addenda, 1983

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manuf. Serial No.	Nat. Board No.	Other Identification	Year Built	Repaired Replaced or Replacement	ASME Code Stamped (Yes or No)
1-GRH-L-51							
Shim ASTM A-570-92				M&E#: 30-153125 ASP16652	n/a	repair	NO

7. Description of Work (See Remarks)
8. Test Conducted: Hydrostatic _____ Pneumatic _____ Nominal Operating Pressure _____
 N/A _____ Other VT-3 Pressure _____ psi Test Temp. _____ °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks Installed shim by welding to correct gap in support to design
Applicable Manufacturer's Data Reports to be attached
criteria.

Ref. JO:C33638-1 File: 1-GRH-L-51 ISI Class: 2

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this
repair conforms to the rules of the ASME Code, Section XI.
 repair or replacement

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed Frank Pisarsky, Maint. Eng. Supervisor Date 4/3, 19 97
 Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Michigan and employed by ARKWRIGHT MUTUAL INS. CO. of NORWOOD, MASS. have inspected the components described in this Owner's Report during the period 11-14-95 to 4/9/97, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

W. A. Matusz Commissions MICHIGAN, ENDNIT
 Inspector's Signature National Board, State, Province, Endorsements

~~*FACTORY MUTUAL SYSTEM~~

Date April 9 19 97

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

NIS-2-175-97

1. Owner AMERICAN ELECTRIC POWER COMPANY Date 04/19/97
 Name
P.O. Box 60 Fort Wayne, IN 46801 Sheet 1 of 2
 Address
2. Plant D.C. COOK NUCLEAR POWER PLANT Unit # 1
 Name
One Cook Place, Bridgman, MI 49106 C34215-01 R/R1499
 Address Repair Org. P.O. No., Job No., etc
3. Work Performed by Installation Services Type Code Symbol Stamp N/A
 Name Authorization No. N/A
Same as #2 Expiration Date N/A
 Address
4. Identification of System RESIDUAL HEAT REMOVAL ASME CODE CLASS 1
5. (a) Applicable Const. Code ANSI B31.1 19 83Ed., N/A Add. N/A Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1989
6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manuf. Serial No.	Nat. Board No.	Other Identification	Year Built	Repaired Replaced or Replacement	ASME Code Stamped (Yes or No)
1-GRH-R503 3/4 CS PLATE	M&E# 30-153217	ASP# 16493	N/A	A36 (91)	N/A	REPLACE- MENT	NO

7. Description of Work MODIFIED SUPPORT 1-GRH-R503 PER DCP-0046.

8. Test Conducted: Hydrostatic ☐ Pneumatic ☐ Nominal Operating Pressure ☐
- N/A ☒ Other ☐ Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks

Applicable Manufacturer's Data Reports to be attached

Ref. JO: C34215-01

DESIGN CHANGE: DCP-0046

REPAIR/REPLACEMENT TRAVELER: RR96-1499

ISI CLASS: 2

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this REPLACEMENT conforms to the rules of the ASME Code, Section XI. repair or replacement

Type Code Symbol Stamp N/ACertificate of Authorization No. N/A Expiration Date N/A

Signed Wayne Cretsinger, Senior Planner
Owner or Owner's Designee, Title

Date APRIL 19, 1997

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Michigan and employed by ARKWRIGHT MUTUAL INS. CO. of WALTHAM, MASS. have inspected the components described in this Owner's Report during the period 4/26/96 to 5/6/97, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Inspector's Signature

Commissions

National Board, State, Province, Endorsements

Date May 6, 1997

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

NIS-2-135-97

1. Owner AMERICAN ELECTRIC POWER COMPANY Date 04/09/97
 Name
P.O. Box 60 Fort Wayne, IN 46801 Sheet 1 of 3
 Address
2. Plant D.C. COOK NUCLEAR POWER PLANT Unit # 1
 Name
One Cook Place, Bridgman, MI 49106 C37156-02
 Address Repair Org. P.O. No., Job No., etc
3. Work Performed by Installation Services Type Code Symbol Stamp N/A
 Name Authorization No. N/A
Same as #2 Expiration Date N/A
 Address
4. Identification of System SAFETY INJECTION ASME CODE CLASS 2
5. (a) Applicable Const. Code ANSI B31.1 19 83Ed., N/A Add. N/A Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1989
6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manuf. Serial No.	Nat. Board No.	Other Identification	Year Built	Repaired Replaced or Replacement	ASME Code Stamped (Yes or No)
1-GSI-R527 1/4 CS PLATE	CONSOLIDATED POWER SUPPLY	ASP# 18784	N/A	A36 (91)	N/A	REPLACE- MENT	NO
1-GSI-R527 1/4 CS PLATE	CONSOLIDATED POWER SUPPLY	ASP# 18884	N/A	A36 (90)	N/A	REPLACE- MENT	NO
1-GSI-R527 3/8 CS PLATE	DUBOSE NATIONAL ENERGY	ASP# 17466	N/A	A36 (93A)	N/A	REPLACE- MENT	NO

7. Description of Work REWORKED SUPPORT 1-GSI-R527 TO MEET DESIGN REQUIREMENTS

8. Test Conducted: Hydrostatic ☐ Pneumatic ☐ Nominal Operating Pressure ☐
 N/A ☒ Other ☐ Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

NIS-2-135-97

1. Owner AMERICAN ELECTRIC POWER COMPANY Date 04/09/97
Name
P.O. Box 60 Fort Wayne, IN 46801 Sheet 2 of 3
Address
2. Plant D.C. COOK NUCLEAR POWER PLANT Unit # 1
Name
One Cook Place, Bridgman, MI 49106 C37156-02
Address Repair Org. P.O. No., Job No., etc
3. Work Performed by Installation Services Type Code Symbol Stamp N/A
Name Authorization No. N/A
Same as #2 Expiration Date N/A
Address
4. Identification of System SAFETY INJECTION ASME CODE CLASS 2
5. (a) Applicable Const. Code ANSI B31.1 19 83Ed., N/A Add. N/A Code Case
(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1989
6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manuf. Serial No.	Nat. Board No.	Other Identification	Year Built	Repaired Replaced or Replacement	ASME Code Stamped (Yes or No)
1-GSI-R527 1/2 CS PLATE	CONSOLIDATED POWER SUPPLY	ASP# 16973	N/A	A36 (91)	N/A	REPLACE- MENT	NO
1-GSI-R527 1" HILTI BOLT	HILTI INC.	ASP# 14640	N/A	HILTI KB II (93)	N/A	REPLACE- MENT	NO

7. Description of Work REWORKED SUPPORT 1-GSI-R527 TO MEET DESIGN REQUIREMENTS

8. Test Conducted: Hydrostatic ☐ Pneumatic ☐ Nominal Operating Pressure ☐
N/A ☒ Other ☐ Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

Remarks

Applicable Manufacturer's Data Reports to be attached

Ref. JO: C37156-02 DESIGN CHANGE: N/A

REPAIR/REPLACEMENT TRAVELLER: C37156-02 ISI CLASS: 2

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this REPLACEMENT conforms to the rules of the ASME Code, Section XI.
repair or replacement

Type Code Symbol Stamp N/ACertificate of Authorization No. N/A Expiration Date N/A

Signed Wayne Cretsinger, Senior Planner
Owner or Owner's Designee, Title

Date APRIL 10, 1997

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Michigan and employed by ARKWRIGHT MUTUAL INS. CO. of WALTHAM, MASS. have inspected the components described in this Owner's Report during the period 1/28/97 to 4/16/97, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

MA [Signature]
Inspector's Signature

Commissions MA 0055, END N.E.I.
National Board, State, Province, Endorsements

Date APRIL 16 19 97

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS

As Required by the Provisions of the ASME Code Section XI

NIS-2-173-97

1. Owner AMERICAN ELECTRIC POWER COMPANY Date 04/19/97
 Name
P.O. Box 60 Fort Wayne, IN 46801 Sheet 1 of 3
 Address
2. Plant D.C. COOK NUCLEAR POWER PLANT Unit # 1
 Name
One Cook Place, Bridgman, MI 49106 C34215-01 R/R1488 THRU 1492
 Address Repair Org. P.O. No., Job No., etc
3. Work Performed by Installation Services Type Code Symbol Stamp N/A
 Name Authorization No. N/A
Same as #2 Expiration Date N/A
 Address
4. Identification of System SAFETY INJECTION ASME CODE CLASS 2
5. (a) Applicable Const. Code ANSI B31.1 19 83Ed. N/A Add. N/A Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1989

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manuf. Serial No.	Nat. Board No.	Other Identification	Year Built	Repaired Replaced or Replacement	ASME Code Stamped (Yes or No)
1-GSI-R542 3/8 SS PLATE	M&E# 30-153224	ASP# 9509	N/A	A240 TYPE 304 (90)	N/A	REPLACE- MENT	NO
1-GSI-R562 3/8 CS PLATE	M&E# 30-153143	ASP# 17466	N/A	A36 (93A)	N/A	REPLACE- MENT	NO
1-GSI-R562 1/4 CS PLATE	M&E# 30-153128	ASP# 14197	N/A	A36 (90)	N/A	REPLACE- MENT	NO
1-GSI-R589 2 x 2 x 1/4 CS ANGLE	M&E# 30-153012	ASP# 17202	N/A	A36 (90)	N/A	REPLACE- MENT	NO
1-GSI-R588 3/8 CS PLATE	M&E# 30-153192	ASP# 15485	N/A	A36 (93A)	N/A	REPLACE- MENT	NO

7. Description of Work MODIFIED SUPPORTS 1-GSI-R542, R562, R589, R588 AND R634 PER DCP-0046

8. Test Conducted: Hydrostatic ☐ Pneumatic ☐ Nominal Operating Pressure ☐
 N/A ☒ Other ☐ Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

NIS-2-173-97

1. Owner AMERICAN ELECTRIC POWER COMPANY Date 04/19/97
Name
P.O. Box 60 Fort Wayne, IN 46801 Sheet 2 of 3
Address
2. Plant D.C. COOK NUCLEAR POWER PLANT Unit # 1
Name
One Cook Place, Bridgman, MI 49106 C34215-01 R/R1488 THRU 1492
Address Repair Org. P.O. No., Job No., etc
3. Work Performed by Installation Services Type Code Symbol Stamp N/A
Name Authorization No. N/A
Same as #2 Expiration Date N/A
Address
4. Identification of System SAFETY INJECTION ASME CODE CLASS 2
5. (a) Applicable Const. Code ANSI B31.1 19 83Ed., N/A Add. N/A Code Case
(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1989

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manuf. Serial No.	Nat. Board No.	Other Identification	Year Built	Repaired Replaced or Replacement	ASME Code Stamped (Yes or No)
1-GSI-R588 3/8 SS PLATE	M&E# 30-153224	ASP# 9509	N/A	A240 TYPE 304 (90)	N/A	REPLACE- MENT	NO
1-GSI-R588 1/2 CS PLATE	M&E# 30-153147	ASP# 16476	N/A	A36 (90)	N/A	REPLACE- MENT	NO
1-GSI-R634 2 x 2 x 1/4 CS ANGLE	M&E# 30-153012	ASP# 17202	N/A	A36 (90)	N/A	REPLACE- MENT	NO

7. Description of Work MODIFIED SUPPORTS 1-GSI-R542, R562, R589, R588 AND R634 PER DCP-0046.

8. Test Conducted: Hydrostatic ☐ Pneumatic ☐ Nominal Operating Pressure ☐
N/A ☒ Other ☐ Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks

Applicable Manufacturer's Data Reports to be attached

Ref. JO: C34215-01

DESIGN CHANGE: DCP-0046

REPAIR/REPLACEMENT TRAVELER: RR96-1488

ISI CLASS: 2

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this
REPLACEMENT conforms to the rules of the ASME Code, Section XI.
 repair or replacement

Type Code Symbol Stamp N/ACertificate of Authorization No. N/A Expiration Date N/A

Signed Wayne Cretsinger, Senior Planner
 Owner or Owner's Designee, Title

Date APRIL 19, 1997

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Michigan and employed by ARKWRIGHT MUTUAL INS. CO. of WALTHAM, MASS. have inspected the components described in this Owner's Report during the period 4/26/96 to 5/6/97, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Inspector's Signature

Commissions Mich 0055, END N.B.T.
 National Board, State, Province, Endorsements

Date May 6 19 97

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

NIS-2-174-97

1. Owner AMERICAN ELECTRIC POWER COMPANY Date 04/19/97
Name
P.O. Box 60 Fort Wayne, IN 46801 Sheet 1 of 3
Address
2. Plant D.C. COOK NUCLEAR POWER PLANT Unit # 1
Name
One Cook Place, Bridgman, MI 49106 C34215-01 R/R1494 THRU 1497
Address Repair Org. P.O. No., Job No., etc
3. Work Performed by Installation Services Type Code Symbol Stamp N/A
Name Authorization No. N/A
Same as #2 Expiration Date N/A
Address
4. Identification of System SAFETY INJECTION ASME CODE CLASS 1
5. (a) Applicable Const. Code ANSI B31.1 19 83Ed., N/A Add. N/A Code Case
(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1989
6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manuf. Serial No.	Nat. Board No.	Other Identification	Year Built	Repaired Replaced or Replacement	ASME Code Stamped (Yes or No)
1-GSI-R559 3/8 CS PLATE	M&E# 30-153142	ASP# 16013	N/A	A36 (90)	N/A	REPLACE- MENT	NO
1-GSI-R559 1/2 CS PLATE	M&E# 30-153145	ASP# 12240	N/A	A36 (93A)	N/A	REPLACE- MENT	NO
1-GSI-R576 1/2 CS PLATE	M&E# 30-153147	ASP# 16476	N/A	A36 (90)	N/A	REPLACE- MENT	NO
1-GSI-R576 3/8 SS PLATE	M&E# 30-153224	ASP# 9509	N/A	A240 TYPE 304 (90)	N/A	REPLACE- MENT	NO
1-GSI-R637 1½ x 1½ x ¼ CS ANGLE	M&E# 30-153010	ASP# 11779	N/A	A36 (93A)	N/A	REPLACE- MENT	NO
1-GSI-R637 1/8 SS PLATE	M&E# 30-153156	ASP# 12890	N/A	A479 TYPE 304 (90)	N/A	REPLACE- MENT	NO

7. Description of Work MODIFIED SUPPORTS 1-GSI-R559, R576, R637 AND
1-ASI-R5302 PER DCP-0046.

8. Test Conducted: Hydrostatic ☐ Pneumatic ☐ Nominal Operating Pressure ☐
N/A ☒ Other ☐ Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.



FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

NIS-2-174-97

1. Owner AMERICAN ELECTRIC POWER COMPANY Date 04/19/97
Name
P.O. Box 60 Fort Wayne, IN 46801 Sheet 2 of 3
Address
2. Plant D.C. COOK NUCLEAR POWER PLANT Unit # 1
Name
One Cook Place, Bridgman, MI 49106 C34215-01 R/R1494 THRU 1497
Address Repair Org. P.O. No., Job No., etc
3. Work Performed by Installation Services Type Code Symbol Stamp N/A
Name Authorization No. N/A
Same as #2 Expiration Date N/A
Address
4. Identification of System SAFETY INJECTION ASME CODE CLASS 1
5. (a) Applicable Const. Code ANSI B31.1 19 83Ed., N/A Add. N/A Code Case
(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1989
6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manuf. Serial No.	Nat. Board No.	Other Identification	Year Built	Repaired Replaced or Replacement	ASME Code Stamped (Yes or No)
1-ASI-R5302 1/2 HILTI BOLTS	M&E# 30-046271	ASP# 16478	N/A	HILTI KB II (90)	N/A	REPLACE-MENT	NO
1-ASI-R5302 1/2 CS PLATE	M&E# 30-153186	ASP# 17667	N/A	A36 (93)	N/A	REPLACE-MENT	NO
1-ASI-R5302 3 x 3 x 1/4 CS TUBE STEEL	M&E# 30-150169	ASP# 8034	N/A	A500 GR B (90)	N/A	REPLACE-MENT	NO
1-ASI-R5302 1/2 CS PLATE	M&E# 30-153251	ASP# 15485	N/A	A36 (93A)	N/A	REPLACE-MENT	NO
1-ASI-R5302 2 x 2 x 1/4 CS ANGLE	M&E# 30-153012	ASP# 17202	N/A	A36 (90)	N/A	REPLACE-MENT	NO
1-ASI-R5302 1/8 CS PLATE	M&E# 30-153156	ASP# 12890	N/A	A479 TYPE 304 (90)	N/A	REPLACE-MENT	NO

7. Description of Work MODIFIED SUPPORTS 1-GSI-R559, R576, R637 AND 1-ASI-R5302 PER DCP-0046.

8. Test Conducted: Hydrostatic ☐ Pneumatic ☐ Nominal Operating Pressure ☐
- N/A ☒ Other ☐ Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks

Applicable Manufacturer's Data Reports to be attached

Ref. JO: C34215-01

DESIGN CHANGE: DCP-0046

REPAIR/REPLACEMENT TRAVELER: RR96-1494

ISI CLASS: 1

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this REPLACEMENT conforms to the rules of the ASME Code, Section XI. repair or replacement

Type Code Symbol Stamp N/ACertificate of Authorization No. N/A Expiration Date N/A

Signed Wayne Cretsinger, Senior Planner
Owner or Owner's Designee, Title

Date APRIL 19, 1997

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Michigan and employed by ARKWRIGHT MUTUAL INS. CO. of WALTHAM, MASS. have inspected the components described in this Owner's Report during the period 4/26/96 to 5/16/97, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Mr. Whitehouse Commissions Mich 0055, END NPT
Inspector's Signature National Board, State, Province, Endorsements

Date May 16 1997

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

NIS-2-193-97

1. Owner AMERICAN ELECTRIC POWER COMPANY Date 05/05/97
 Name
P.O. Box 60 Fort Wayne, IN 46801 Sheet 1 of 2
 Address
2. Plant D.C. COOK NUCLEAR POWER PLANT Unit # 1
 Name
One Cook Place, Bridgman, MI 49106 C34216-07 R/R1498
 Address Repair Org. P.O. No., Job No., etc
 Type Code Symbol Stamp N/A
 Authorization No. N/A
 Expiration Date N/A
3. Work Performed by Installation Services
 Name
Same as #2
 Address
4. Identification of System SAFETY INJECTION ASME CODE CLASS 1
5. (a) Applicable Const. Code ANSI B31.1 19 83Ed., N/A Add. N/A Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1989

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manuf. Serial No.	Nat. Board No.	Other Identification	Year Built	Repaired Replaced or Replacement	ASME Code Stamped (Yes or No)
1-GSI-R575 FIG #211N SWAY STRUT	GRINNELL CORP. P.O. 01632-041-6Y	ASP# 18815	N/A	SIZE C, PO ITEM 02	N/A	REPLACE- MENT	NO

7. Description of Work MODIFIED SUPPORT 1-GSI-R575 PER DCP-0046.

8. Test Conducted: Hydrostatic ☐ Pneumatic ☐ Nominal Operating Pressure ☐
 N/A ☒ Other ☐ Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks

Applicable Manufacturer's Data Reports to be attached

Ref. JO: C34216-07

DESIGN CHANGE: DCP-0046

REPAIR/REPLACEMENT TRAVELER: RR96-1498

ISI CLASS: 2

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this
REPLACEMENT conforms to the rules of the ASME Code, Section XI.
 repair or replacement

Type Code Symbol Stamp N/ACertificate of Authorization No. N/A Expiration Date N/A

Signed Wayne Cretsinger, Senior Planner
 Owner or Owner's Designee, Title

Date MAY 05, 1997

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Michigan and employed by ARKWRIGHT MUTUAL INS. CO. of WALTHAM, MASS. have inspected the components described in this Owner's Report during the period 4/20/96 to 5/5/97, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Inspector's Signature

Commissions

National Board, State, Province, Endorsements

Date JUNE 5, 1997



FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS

As Required by the Provisions of the ASME Code Section XI

NIS-2-185-97

1. Owner AMERICAN ELECTRIC POWER COMPANY Date 05/01/97
 Name
P.O. Box 60 Fort Wayne, IN 46801 Sheet 2 of 3
 Address
2. Plant D.C. COOK NUCLEAR POWER PLANT Unit # 1
 Name
One Cook Place, Bridgman, MI 49106 C34216-01 R/R1526 THRU 1529
 Address Repair Org. P.O. No., Job No., etc
3. Work Performed by Installation Services Type Code Symbol Stamp N/A
 Name
Same as #2 Authorization No. N/A
 Address Expiration Date N/A
4. Identification of System SAFETY INJECTION ASME CODE CLASS 1
5. (a) Applicable Const. Code ANSI B31.1 19 83Ed., N/A Add. N/A Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1989

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manuf. Serial No.	Nat. Board No.	Other Identification	Year Built	Repaired Replaced or Replacement	ASME Code Stamped (Yes or No)
1-GSI-R705 1/4 CS PLATE	M&E# 30-153133	ASP# 18784	N/A	A36 (91)	N/A	REPLACE- MENT	NO

7. Description of Work MODIFIED SUPPORTS 1-GSI-R660, R697, R703 AND R705 PER DCP-0046.

8. Test Conducted: Hydrostatic ☐ Pneumatic ☐ Nominal Operating Pressure ☐
 N/A ☒ Other ☐ Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

Remarks

Applicable Manufacturer's Data Reports to be attached

Ref. JO: C34216-01

DESIGN CHANGE: DCP-0046

REPAIR/REPLACEMENT TRAVELER: RR96-1526

ISI CLASS: 1

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this REPLACEMENT conforms to the rules of the ASME Code, Section XI. repair or replacement

Type Code Symbol Stamp N/ACertificate of Authorization No. N/A Expiration Date N/A

Signed Wayne Cretsinger, Senior Planner
Owner or Owner's Designee, Title

Date MAY 01, 1997

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Michigan and employed by ARKWRIGHT MUTUAL INS. CO. of WALTHAM, MASS. have inspected the components described in this Owner's Report during the period 4/26/96 to 5/15/97, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Inspector's Signature

Commissions

National Board, State, Province, Endorsements

Date May 15, 1997

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner AMERICAN ELECTRIC POWER Date 03-21-97
Name
P.O. Box 60 Fort Wayne, IN 46801 Sheet 1 of 2
Address
2. Plant D.C. COOK NUCLEAR POWER PLANT Unit # 1
Name
One Cook Place, Bridgman, MI 49106 C39642-03, 1-ICM-250, vlv
Address Repair Org. P.O. No., Job No., etc
3. Work Performed by Maintenance Department Type Code Symbol Stamp N/A
Name Authorization No. N/A
Same as #2 Expiration Date N/A
Address
4. Identification of System Boron Injection
5. (a) Applicable Const. Code ANSI B31.1 19 83Ed., N/A Add. N/A Code Case
(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1989,
No Addenda

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manuf. Serial No.	Nat. Board No.	Other Identification	Year Built	Repaired Replaced or Replacement	ASME Code Stamped (Yes or No)
1-ICM-250							
Bonnet/body Nuts SA-194 gr 8F				M&E#: 30212142 ASP#15702 16322	n/a	replace	NO

7. Description of Work (See Remarks)
8. Test Conducted: Hydrostatic _____ Pneumatic _____ Nominal Operating Pressure _____
N/A _____ Other VT-1 Pressure _____ psi Test Temp. _____ °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks Replaced Bonnet to Body Nuts

Applicable Manufacturer's Data Reports to be attached

Ref. JO:C39642-3 File: 1-ICM-250 ISI Class: 1

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this
replacement conforms to the rules of the ASME Code, Section XI.
 repair or replacement

Type Code Symbol Stamp N/ACertificate of Authorization No. N/A Expiration Date N/A

Signed Frank Pisarsky, Maint. Eng. Supervisor Date 4/9, 1997
 Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Michigan and employed by ARKWRIGHT MUTUAL INS. CO.* of NORWOOD, MASS. have inspected the components described in this Owner's Report during the period 3/21/97 to 4/10/97, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Inspector's Signature [Signature]Commissions Mich 0055, and NBE
National Board, State, Province, EndorsementsDate April 10 19 97

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner AMERICAN ELECTRIC POWER Date 02-02-96
Name
P.O. Box 60 Fort Wayne, IN 46801 Sheet 1 of 2
Address
2. Plant D.C. COOK NUCLEAR POWER PLANT Unit # 1
Name
One Cook Place, Bridgman, MI 49106 C33810-7, 1-IRV-311, vlv
Address Repair Org. P.O. No., Job No., etc
3. Work Performed by Maintenance Department Type Code Symbol Stamp N/A
Name Authorization No. N/A
Same as #2 Expiration Date N/A
Address
4. Identification of System Residual Heat Removal
5. (a) Applicable Const. Code ANSI B31.1 19 83Ed., N/A Add. N/A Code Case
(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1983 Ed.
Summer Addenda 1983
6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manuf. Serial No.	Nat. Board No.	Other Identification	Year Built	Repaired Replaced or Replacement	ASME Code Stamped (Yes or No)
1- IRV-311							
Seal protector ring CF8M 316	Fisher Controls			M&E#: 30-039730 ASP#16658 (complete valve)	n/a	replace	NO

7. Description of Work (See Remarks)
8. Test Conducted: Hydrostatic _____ Pneumatic _____ Nominal Operating Pressure _____
N/A ☒ Other _____ Pressure _____ psi Test Temp. _____ °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

NIS-2-178-97

1. Owner AMERICAN ELECTRIC POWER Date 04/25/97
Name
P.O. Box 60 Fort Wayne, IN 46801 Sheet 1 of 2
Address
2. Plant D.C. COOK NUCLEAR POWER PLANT Unit # 1
Name
One Cook Place, Bridgman, MI 49106 R52239-16 R/R1621
Address Repair Org. P.O. No., Job No., etc
3. Work Performed by Installation Services Type Code Symbol Stamp N/A
Name
Same as #2 Authorization No. N/A
Address Expiration Date N/A
4. Identification of System REACTOR COOLANT ASME CODE CLASS 1
5. (a) Applicable Const. Code ANSI B31.1 19 83Ed., "N/A" Add: N/A Code Case
(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1989
6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manuf. Serial No.	Nat. Board No.	Other Identification	Year Built	Repaired Replaced or Replacement	ASME Code Stamped (Yes or No)
1-SV-45C	M&E# 30-036392 ASP# 18997	RV-1- 8010A	N/A	STYLE HB-86-BP-E	N/A	REPLACED	NO

7. Description of Work REMOVED AND REPLACED VALVE 1-SV-45C FOR SETPOINT TESTING.

8. Test Conducted: Hydrostatic ☐ Pneumatic ☐ Nominal Operating Pressure ☒
N/A ☐ Other ☐ Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.



FORM NIS-2 (Back)

9. Remarks

Applicable Manufacturer's Data Reports to be attached

Ref. JO: R52037-06 & 16 DESIGN CHANGE: N/A

REPAIR/REPLACEMENT TRAVELLER: RR96-1620 ISI CLASS: 1

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this
REPLACEMENT conforms to the rules of the ASME Code, Section XI.
 repair or replacement

Type Code Symbol Stamp N/ACertificate of Authorization No. N/A Expiration Date N/A

Signed Wayne Cretsinger, Senior Planner Date APRIL 25, 19 97
 Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of MICHIGAN and employed by ARKWRIGHT MUTUAL INS. CO. of WALTHAM, MASS. have inspected the components described in this Owner's Report during the period 12-10-96 to 5-5-97, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

W. H. Thompson Commissions MICHIGAN, END NBT
 Inspector's Signature National Board, State, Province, Endorsements

Date May 5, 19 97

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

NIS-2-177-97

1. Owner AMERICAN ELECTRIC POWER Date 04/25/97
 Name
 P.O. Box 60 Fort Wayne, IN 46801 Sheet 1 of 2
 Address
2. Plant D.C. COOK NUCLEAR POWER PLANT Unit # 1
 Name
 One Cook Place, Bridgman, MI 49106 R52037-16 R/R1620
 Address Repair Org. P.O. No., Job No., etc
3. Work Performed by Installation Services Type Code Symbol Stamp N/A
 Name Authorization No. N/A
 Same as #2 Expiration Date N/A
 Address

4. Identification of System REACTOR COOLANT ASME CODE CLASS 1
5. (a) Applicable Const. Code ANSI B31.1 19 83Ed., N/A Add. N/A Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1989
6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manuf. Serial No.	Nat. Board No.	Other Identification	Year Built	Repaired Replaced or Replacement	ASME Code Stamped (Yes or No)
1-SV-45B	M&E# 30-036392 ASP# 18997	RV-2- 8010B	N/A	STYLE HB-86-BP-E	N/A	REPLACED	NO
1 3/8" SS H.H. NUT	M&E# 30-047350	ASP# 12362	N/A	SA194 GR 8F	N/A	REPLACE- MENT	NO

7. Description of Work REMOVED AND REPLACED VALVE 1-SV-45B FOR SETPOINT TESTING AND REPLACED INLET NUT THAT WAS MISPLACED.

8. Test Conducted: Hydrostatic ☐ Pneumatic ☐ Nominal Operating Pressure ☒ ☐
- N/A ☐ Other ☐ Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks

Applicable Manufacturer's Data Reports to be attached

Ref. JO: R52036-06 & 16 DESIGN CHANGE: N/A

REPAIR/REPLACEMENT TRAVELLER: RR96-1619 ISI CLASS: 1

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this
REPLACEMENT conforms to the rules of the ASME Code, Section XI.
 repair or replacement

Type Code Symbol Stamp N/ACertificate of Authorization No. N/A Expiration Date N/A

Signed Wayne Cretsinger, Senior Planner
 Owner or Owner's Designee, Title

Date APRIL 15, 1997

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of MICHIGAN and employed by ARKWRIGHT MUTUAL INS. CO. of WALTHAM, MASS. have inspected the components described in this Owner's Report during the period 12/10/96 to 5/5/97, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

W. E. Waters
 Inspector's Signature

Commissions Michigan, ENI, NFI
 National Board, State, Province, Endorsements

Date May 5, 1997

1. Owner AMERICAN ELECTRIC POWER Date 04/25/97
 Name
P.O. Box 60 Fort Wayne, IN 46801 Sheet 1 of 2
 Address
2. Plant D.C. COOK NUCLEAR POWER PLANT Unit # 1
 Name
One Cook Place, Bridgman, MI 49106 R52036-16 R/R1619
 Address Repair Org. P.O. No., Job No., etc
3. Work Performed by Installation Services Type Code Symbol Stamp N/A
 Name
Same as #2 Authorization No. N/A
 Address Expiration Date N/A
4. Identification of System REACTOR COOLANT ASME CODE CLASS 1
5. (a) Applicable Const. Code ANSI B31.1 19 83Ed., N/A Add. N/A Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1989
6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manuf. Serial No.	Nat. Board No.	Other Identification	Year Built	Repaired Replaced or Replacement	ASME Code Stamped (Yes or No)
1-SV-45A	M&E# 30-036392 ASP# 18997	RV-1- 8010B	N/A	STYLE HB-86-BP-E	N/A	REPLACED	NO
1 3/8" SS H.H. NUT	M&E# 30-046895	ASP# 8006	N/A	SA194 GR 8F	N/A	REPLACE- MENT	NO
1 3/8" SS H.H. NUT	M&E# 30-046895	ASP# 10340	N/A	SA194 GR 8F	N/A	REPLACE- MENT	NO

7. Description of Work REMOVED AND REPLACED VALVE 1-SV-45A FOR SETPOINT TESTING AND REPLACED INLET NUTS THAT WERE MISPLACED.

8. Test Conducted: Hydrostatic ☐ Pneumatic ☐ Nominal Operating Pressure ☒
 N/A ☐ Other ☐ Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks Replaced the valve's disc
Applicable Manufacturer's Data Reports to be attached

Ref. JO:R61020-1 File: 1-SV-2B-3 ISI Class: 2

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this
replacement conforms to the rules of the ASME Code, Section XI.
 repair or replacement

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed Frank Pisarsky, Maint. Eng. Supervisor Date 4/24, 1997
 Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Michigan and employed by ARKWRIGHT MUTUAL INS. CO.* of NORWOOD, MASS. have inspected the components described in this Owner's Report during the period 3/5/97 to 5/27/97, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

M.R. Whittemore Commissions M.A. 10055, END N.P. I
 Inspector's Signature National Board, State, Province, Endorsements

Date May 27 19 97

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner AMERICAN ELECTRIC POWER Date 03-08-97
Name
P.O. Box 60 Fort Wayne, IN 46801 Sheet 1 of 2
Address
2. Plant D.C. COOK NUCLEAR POWER PLANT Unit # 1
Name
One Cook Place, Bridgman, MI 49106 R61020, 1-SV-2B-3, Vlv.
Address Repair Org. P.O. No., Job No., etc
3. Work Performed by Maintenance Department Type Code Symbol Stamp N/A
Name Authorization No. N/A
Same as #2 Expiration Date N/A
Address
4. Identification of System Mainj Steam
5. (a) Applicable Const. Code ANSI B31.1 19 83Ed., N/A Add. N/A Code Case
(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1989,
No Addenda
6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manuf. Serial No.	Nat. Board No.	Other Identification	Year Built	Repaired Replaced or Replacement	ASME Code Stamped (Yes or No)
1-SV-2B-3							
Disc ASME SB637				M&E#: 30-037699 ASP#19528	n/a	replace	NO

7. Description of Work (See Remarks)
8. Test Conducted: Hydrostatic _____ Pneumatic _____ Nominal Operating Pressure _____
N/A ☒ Other _____ Pressure _____ psi Test Temp. _____ °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks Replaced the entire valve (mechanical installation)
Applicable Manufacturer's Data Reports to be attached

Ref. JO:R56435-01 File: 1-SV-2B-2 ISI Class: 2

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this
replacement conforms to the rules of the ASME Code, Section XI.
 repair or replacement

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed Frank Pisarsky, Maint. Eng. Supervisor Date 7/2, 1997
 Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Michigan and employed by ARKWRIGHT MUTUAL INS. CO.* of NORWOOD, MASS. have inspected the components described in this Owner's Report during the period 3/17/97 to 7/15/97, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

M. J. Winterspaul Commissions Mich 0055, GND N&I
 Inspector's Signature National Board, State, Province, Endorsements

Date July 15 19 97

1. Owner AMERICAN ELECTRIC POWER Date 03-20-97
Name
P.O. Box 60 Fort Wayne, IN 46801 Sheet 1 of 2
Address
2. Plant D.C. COOK NUCLEAR POWER PLANT Unit # 1
Name
One Cook Place, Bridgman, MI 49106 R56435-01, 1-SV-2B-2
Address Repair Org. P.O. No., Job No., etc
3. Work Performed by Maintenance Department Type Code Symbol Stamp N/A
Name Authorization No. N/A
Same as #2 Expiration Date N/A
Address
4. Identification of System Steam Generator (Main Steam)
5. (a) Applicable Const. Code ANSI B31.1 19 83Ed., N/A Add. N/A Code Case
(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1989,
No Addenda

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manuf. Serial No.	Nat. Board No.	Other Identification	Year Built	Repaired Replaced or Replacement	ASME Code Stamped (Yes or No)
1-SV-2B-2							
Valve	Dresser			M&E#: 30037161 ASP#19438	n/a	replace	NO

7. Description of Work (See Remarks)
8. Test Conducted: Hydrostatic _____ Pneumatic _____ Nominal Operating Pressure _____
N/A _____ Other VT-2 Pressure _____ psi Test Temp. _____ °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks Mechanically replaced the valve's disc and yoke nuts
Applicable Manufacturer's Data Reports to be attached

Ref. JO:C39824-01 File: 1-SV-2A-3 ISI Class: 2

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this
replacement conforms to the rules of the ASME Code, Section XI.
 repair or replacement

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed Frank Pisarsky, Maint. Eng. Supervisor Date 4/23, 19 97
 Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Michigan and employed by ARKWRIGHT MUTUAL INS. CO.* of NORWOOD, MASS. have inspected the components described in this Owner's Report during the period 3-3-97 to 5-28-97, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Inspector's Signature [Signature]

Commissions McLooss, GND NFI
 National Board, State, Province, Endorsements

Date May 28 19 97

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner AMERICAN ELECTRIC POWER Date 03-11-97
Name
P.O. Box 60 Fort Wayne, IN 46801 Sheet 1 of 2
Address
2. Plant D.C. COOK NUCLEAR POWER PLANT Unit # 1
Name
One Cook Place, Bridgman, MI 49106 C39824-01, 1-SV-2A-3, vlv
Address Repair Org. P.O. No., Job No., etc
3. Work Performed by Maintenance Department Type Code Symbol Stamp N/A
Name Authorization No. N/A
Same as #2 Expiration Date N/A
Address
4. Identification of System Main Steam
5. (a) Applicable Const. Code ANSI B31.1 1983Ed., N/A Add. N/A Code Case.
(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1989, No Addenda
6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manuf. Serial No.	Nat. Board No.	Other Identification	Year Built	Repaired Replaced or Replacement	ASME Code Stamped (Yes or No)
1-SV-2A-3							
Disc ASME-SB637				M&E#: 30-037699 ASP#19528	n/a	replace	NO
Yoke Nuts: SA-194 gr.2H				M&E#: 30-211970 ASP#18093	n/a	replace	NO

7. Description of Work (See Remarks)
8. Test Conducted: Hydrostatic _____ Pneumatic _____ Nominal Operating Pressure _____
N/A ☒ Other _____ Pressure _____ psi Test Temp. _____ °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.



FORM NIS-2 (Back)

9. Remarks Replaced the valve's disc and yoke nutsApplicable Manufacturer's Data Reports to be attachedRef. JO:R62233-01 File: 1-SV-2A-1ISI Class: 2

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this
replacement conforms to the rules of the ASME Code, Section XI.
 repair or replacement

Type Code Symbol Stamp N/ACertificate of Authorization No. N/A Expiration Date N/A

Signed Frank Pisarsky, Maint. Eng. Supervisor Date 4/24, 1997
 Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Michigan and employed by ARKWRIGHT MUTUAL INS. CO.* of NORWOOD, MASS. have inspected the components described in this Owner's Report during the period 3-11-97 to 5/27/97, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

MT Waterman Commissions MA 0055, END N 56
 Inspector's Signature National Board, State, Province, Endorsements

Date May 27 19 97

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner AMERICAN ELECTRIC POWER Date 03-11-97
Name
P.O. Box 60 Fort Wayne, IN 46801 Sheet 1 of 2
Address
2. Plant D.C. COOK NUCLEAR POWER PLANT Unit # 1
Name
One Cook Place, Bridgman, MI 49106 R62233-1, 1-SV-2A-1, vlv
Address Repair Org. P.O. No., Job No., etc
3. Work Performed by Maintenance Department Type Code Symbol Stamp N/A
Name Authorization No. N/A
Same as #2 Expiration Date N/A
Address
4. Identification of System Main Steam
5. (a) Applicable Const. Code ANSI B31.1 19 83Ed., N/A Add. N/A Code Case
(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1989,
No Addenda
6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manuf. Serial No.	Nat. Board No.	Other Identification	Year Built	Repaired Replaced or Replacement	ASME Code Stamped (Yes or No)
1-SV-2A-1							
Yoke Nut SA-194 gr 2H				M&E#: 30-211970 ASP#18093	n/a	replace	NO
Disc ASME SB637				M&E#: 30-037699 ASP#19528	n/a	replace	NO

7. Description of Work (See Remarks)
8. Test Conducted: Hydrostatic _____ Pneumatic _____ Nominal Operating Pressure _____
N/A ✓ Other _____ Pressure _____ psi Test Temp. _____ °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks Replaced the valve's disc
Applicable Manufacturer's Data Reports to be attached

Ref. JO:R61019-1 File: 1-SV-1B-2 ISI Class: 2

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this
replacement conforms to the rules of the ASME Code, Section XI.
 repair or replacement

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed Frank Pisersky, Maint. Eng. Supervisor Date 4/24, 1997
 Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Michigan and employed by ARKWRIGHT MUTUAL INS. CO.* of NORWOOD, MASS. have inspected the components described in this Owner's Report during the period 2-27-97 to 5-27-97, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Michael J. Pisersky Commissions Mich. Code, ENR NSE
 Inspector's Signature National Board, State, Province, Endorsements

Date May 27 19 97

1. Owner AMERICAN ELECTRIC POWER Date 03-15-97
Name
P.O. Box 60 Fort Wayne, IN 46801 Sheet 1 of 2
Address
2. Plant D.C. COOK NUCLEAR POWER PLANT Unit # 1
Name
One Cook Place, Bridgman, MI 49106 R61019-1, 1-SV-1B-2, vlv.
Address Repair Org. P.O. No., Job No., etc
3. Work Performed by Maintenance Department Type Code Symbol Stamp N/A
Name Authorization No. N/A
Same as #2 Expiration Date N/A
Address
4. Identification of System _____
5. (a) Applicable Const. Code ANSI B31.1 19 83Ed., N/A Add. N/A Code Case
(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1989;
No Addenda

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manuf. Serial No.	Nat. Board No.	Other Identification	Year Built	Repaired Replaced or Replacement	ASME Code Stamped (Yes or No)
1-SV-1B-2							
Disc ASME SB637				M&E#: 30-037699 ASP#19528	n/a	replace	NO

7. Description of Work (See Remarks)
8. Test Conducted: Hydrostatic _____ Pneumatic _____ Nominal Operating Pressure _____
N/A ☒ Other _____ Pressure _____ psi Test Temp. _____ °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks Replaced the valve disc
Applicable Manufacturer's Data Reports to be attached

Ref. JO:R59811-01 File: 1-SV-1A-3 ISI Class: 2

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this
replacement conforms to the rules of the ASME Code, Section XI.
 repair or replacement

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed Frank Pisarsky, Maint. Eng. Supervisor Date 4/9, 1997
 Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Michigan and employed by ARKWRIGHT MUTUAL INS. CO.* of NORWOOD, MASS. have inspected the components described in this Owner's Report during the period 2-27-97 to 4/10/97, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

M. R. Mataspaugh
 Inspector's Signature

Commissions Michigan, EN, D, N, E, I
 National Board, State, Province, Endorsements

Date April 10 1997

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner AMERICAN ELECTRIC POWER Date 03-10-97
Name
P.O. Box 60 Fort Wayne, IN 46801 Sheet 1 of 2
Address
2. Plant D.C. COOK NUCLEAR POWER PLANT Unit # 1
Name
One Cook Place, Bridgman, MI 49106 R59811-01, 1-SV-1A-3, vlv
Address Repair Org. P.O. No., Job No., etc
3. Work Performed by Maintenance Department Type Code Symbol Stamp N/A
Name Authorization No. N/A
Same as #2 Expiration Date N/A
Address
4. Identification of System Main Steam
5. (a) Applicable Const. Code ANSI B31.1 19 83Ed., N/A Add. N/A Code Case
(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1989,
No Addenda

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manuf. Serial No.	Nat. Board No.	Other Identification	Year Built	Repaired Replaced or Replacement	ASME Code Stamped (Yes or No)
1-SV-1A-3							
Disc: ASME SB637				M&E#: 30-037728 ASP#19528	n/a	replace	NO

7. Description of Work (See Remarks)
8. Test Conducted: Hydrostatic _____ Pneumatic _____ Nominal Operating Pressure _____
N/A ☒ Other _____ Pressure _____ psi Test Temp. _____ °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks Replaced Stud on body/bonnet connection to valve
 Applicable Manufacturer's Data Reports to be attached

Ref. JO:R49141-4 File: 1-SI-170-3 ISI Class: 1

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this
replacement conforms to the rules of the ASME Code, Section XI.
 repair or replacement

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed Frank Pisarsky Maint. Eng. Supervisor Date 7/3, 1997
 Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Michigan and employed by ARKWRIGHT MUTUAL INS. CO.* of NORWOOD, MASS. have inspected the components described in this Owner's Report during the period 7/3/97 to 7/15/97, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

MR. [Signature] Commissions Michoos, END N.E.T.
 Inspector's Signature National Board, State, Province, Endorsements

Date July 15 1997

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner AMERICAN ELECTRIC POWER Date 03-19-97
Name
P.O. Box 60 Fort Wayne, IN 46801 Sheet 1 of 2
Address
2. Plant D.C. COOK NUCLEAR POWER PLANT Unit # 1
Name
One Cook Place, Bridgman, MI 49106 R49141-04, 1-SI-170-3,
Address Repair Org. P.O. No., Job No., etc
3. Work Performed by Maintenance Department Type Code Symbol Stamp N/A
Name Authorization No. N/A
Same as #2 Expiration Date N/A
Address
4. Identification of System Safety Injection
5. (a) Applicable Const. Code ANSI B31.1 19 83Ed., N/A Add. N/A Code Case
(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1989,
No Addenda

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manuf. Serial No.	Nat. Board No.	Other Identification	Year Built	Repaired Replaced or Replacement	ASME Code Stamped (Yes or No)
<u>1-SI-170-3</u>							
<u>Stud: SA-453 gr660</u>				<u>M&E# 30047100 ASP#10356</u>	<u>n/a</u>	<u>replace</u>	<u>NO</u>

7. Description of Work (See Remarks)
8. Test Conducted: Hydrostatic _____ Pneumatic _____ Nominal Operating Pressure _____
N/A _____ Other VT-1/2 Pressure _____ psi Test Temp. _____ °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks Used a Conval manual seat cutter to clean cut the inbody
Applicable Manufacturer's Data Reports to be attached
seating surface of the valve

Ref. JQ:c33542-2 File: 1-SI-169-L4 ISI Class: 2

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this repair conforms to the rules of the ASME Code, Section XI.
repair or replacement

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed Frank Pisarsky, Maint. Eng. Supervisor Date 4/9, 1997
 Owner or Owner's Designee, Title _____

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Michigan and employed by ARKWRIGHT MUTUAL INS. CO.* of NORWOOD, MASS. have inspected the components described in this Owner's Report during the period 3/17/97 to 4/10/97, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Inspector's Signature Mr. M. H. Doss, End N & E Commissions National Board, State, Province, Endorsements

Date April 10 19 97

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner AMERICAN ELECTRIC POWER Date 03-18-97
Name _____
P.O. Box 60 Fort Wayne, IN 46801 Sheet 1 of 2
Address _____
2. Plant D.C. COOK NUCLEAR POWER PLANT Unit _____ # 1
Name _____
One Cook Place, Bridgman, MI 49106 C33542-2, 1-SI-169-L4, vlv
Address _____ Repair Org. P.O. No., Job No., etc _____
3. Work Performed by Maintenance Department Type Code Symbol Stamp N/A
Name _____ Authorization No. N/A
Same as #2 Expiration Date N/A
Address _____
4. Identification of System Safety Injection
5. (a) Applicable Const. Code ANSI B31.1 19 83Ed., N/A Add. N/A Code Case _____
(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1989,
No Addenda

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manuf. Serial No.	Nat. Board No.	Other Identification	Year Built	Repaired Replaced. or Replacement	ASME Code Stamped (Yes or No)
<u>1-SI-169-L4</u>							
<u>Inbody seating surface</u>				<u>existing SA-479-XM19H Nitronic 50</u>	<u>n/a</u>	<u>repair</u>	<u>NO</u>

7. Description of Work (See Remarks)
8. Test Conducted: Hydrostatic _____ Pneumatic _____ Nominal Operating Pressure _____
N/A ☒ Other _____ Pressure _____ psi Test Temp. _____ °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.



FORM NIS-2 (Back)

9. Remarks Replaced the valve's cover studsApplicable Manufacturer's Data Reports to be attachedRef. JO:C39888-3 File: 1-SI-166-L1 ISI Class: 1

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this
replacement conforms to the rules of the ASME Code, Section XI.
 repair or replacement

Type Code Symbol Stamp N/ACertificate of Authorization No. N/A Expiration Date N/A

Signed Frank Pisarsky Date 6/30, 1977
 Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Michigan and employed by ARKWRIGHT MUTUAL INS. CO.* of NORWOOD, MASS. have inspected the components described in this Owner's Report during the period 3-12-77 to 7-2-77, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

W. H. Whitcomb Commissions Mich 0033, ENDR
 Inspector's Signature National Board, State, Province, Endorsements

Date July 2 1977

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner AMERICAN ELECTRIC POWER Date 03-19-97
Name
P.O. Box 60 Fort Wayne, IN 46801 Sheet 1 of 2
Address
2. Plant D.C. COOK NUCLEAR POWER PLANT Unit # 1
Name
One Cook Place, Bridgman, MI 49106 C39888-03, 1-SI-161-L1, valve
Address Repair Org. P.O. No., Job No., etc
3. Work Performed by Maintenance Department Type Code Symbol Stamp N/A
Name Authorization No. N/A
Same as #2 Expiration Date N/A
Address
4. Identification of System Safety Injection
5. (a) Applicable Const. Code ANSI B31.1 19 83Ed., N/A Add. N/A Code Case
(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1989, No Addenda
6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manuf. Serial No.	Nat. Board No.	Other Identification	Year Built	Repaired Replaced or Replacement	ASME Code Stamped (Yes or No)
1-SI-166-L1							
Studs: SA-564 gr-630SS	n/a	n/a	n/a	M&E#: 30044105 ASP#19580	n/a	replace	NO

7. Description of Work (See Remarks)
8. Test Conducted: Hydrostatic _____ Pneumatic _____ Nominal Operating Pressure _____
N/A _____ Other VT-1/2 Pressure _____ psi Test Temp. _____ °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks Replaced Bonnet/body studs. The VT-2 was completed on job order
Applicable Manufacturer's Data Reports to be attached

R65911-2

Ref. JO:R52451-3 File: 1-SI-166-3

ISI Class: 1

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this
replacement conforms to the rules of the ASME Code, Section XI.
 repair or replacement

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed Frank Pisarsky, Maint. Eng. Supervisor Date 6/30, 19 97
 Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Michigan and employed by ARKWRIGHT MUTUAL INS. CO.* of NORWOOD, MASS. have inspected the components described in this Owner's Report during the period 3-18-97 to 7-3-97, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Michael J. McCross Commissions Michigan, and N.B.I.
 Inspector's Signature National Board, State, Province, Endorsements

Date July 3, 19 97

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner AMERICAN ELECTRIC POWER Date 03-18-97
Name
P.O. Box 60 Fort Wayne, IN 46801 Sheet 1 of 2
Address
2. Plant D.C. COOK NUCLEAR POWER PLANT Unit # 1
Name
One Cook Place, Bridgman, MI 49106 R52451-3, 1-SI-166-3, vlv
Address Repair Org. P.O. No., Job No., etc
3. Work Performed by Maintenance Department Type Code Symbol Stamp N/A
Name Authorization No. N/A
Same as #2 Expiration Date N/A
Address
4. Identification of System Safety Injection
5. (a) Applicable Const. Code ANSI B31.1 19 83Ed., N/A Add. N/A Code Case
(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1989
No Addenda

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manuf. Serial No.	Nat. Board No.	Other Identification	Year Built	Repaired Replaced or Replacement	ASME Code Stamped (Yes or No)
1-SI-166-3							
Bonnet/body Studs: SA-453 gr660				M&E#: 30-047100 ASP#10636	n/a	replace	NO

7. Description of Work (See Remarks)
8. Test Conducted: Hydrostatic _____ Pneumatic _____ Nominal Operating Pressure _____
N/A _____ Other VT-1/2 Pressure _____ psi Test Temp. _____ °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks Replaced the valve's cover studs

Applicable Manufacturer's Data Reports to be attached

Ref. JO:C39895-3 File: 1-SI-161-L4 ISI Class: 1

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this
replacement conforms to the rules of the ASME Code, Section XI.
 repair or replacement

Type Code Symbol Stamp N/ACertificate of Authorization No. N/A Expiration Date N/A

Signed Frank Pisarsky, Maint. Eng. Supervisor Date 6/30, 1997
 Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Michigan and employed by ARKWRIGHT MUTUAL INS. CO.* of NORWOOD, MASS. have inspected the components described in this Owner's Report during the period 3-15-97 to 7-2-97, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Mr. Waterhouse Commissions Mich 0055, ENR N#1
 Inspector's Signature National Board, State, Province, Endorsements

Date July 2 19 97

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner AMERICAN ELECTRIC POWER Date 03-21-97
Name
P.O. Box 60 Fort Wayne, IN 46801 Sheet 1 of 2
Address
2. Plant D.C. COOK NUCLEAR POWER PLANT Unit # 1
Name
One Cook Place, Bridgman, MI 49106 C39895-03, 1-SI-161-L4, valve
Address Repair Org. P.O. No., Job No., etc
3. Work Performed by Maintenance Department Type Code Symbol Stamp N/A
Name Authorization No. N/A
Same as #2 Expiration Date N/A
Address
4. Identification of System Safety Injection
5. (a) Applicable Const. Code ANSI B31.1 19 83Ed., N/A Add. N/A Code Case
(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1989,
No Addenda
6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manuf. Serial No.	Nat. Board No.	Other Identification	Year Built	Repaired Replaced or Replacement	ASME Code Stamped (Yes or No)
1-SI-161-L4							
Studs: SA-564 gr-630SS	n/a	n/a	n/a	M&E#: 30044105 ASP#19580	n/a	replace	NO

7. Description of Work (See Remarks)
8. Test Conducted: Hydrostatic _____ Pneumatic _____ Nominal Operating Pressure _____
N/A _____ Other VT-1/2 Pressure _____ psi Test Temp. _____ °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks Replaced the cage and fasteners

Applicable Manufacturer's Data Reports to be attached

Ref. JO:C40547-04 File: 1-SI-158-L3 ISI Class: 1

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code, Section XI.
 repair or replacement

Type Code Symbol Stamp N/ACertificate of Authorization No. N/A Expiration Date N/A

Signed Frank Pisarsky, Maint. Eng. Supervisor Date 7/2, 1987
 Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Michigan and employed by ARKWRIGHT MUTUAL INS. CO.* of NORWOOD, MASS. have inspected the components described in this Owner's Report during the period 7/14/87 to 7/15/87, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

212 [Signature] Commissions Michigan, ENDSI
 Inspector's Signature National Board, State, Province, Endorsements

Date July 15 19 87



1. Owner AMERICAN ELECTRIC POWER Date 04-16-97
 Name
P.O. Box 60 Fort Wayne, IN 46801 Sheet 1 of 2
 Address
2. Plant D.C. COOK NUCLEAR POWER PLANT Unit # 1
 Name
One Cook Place, Bridgman, MI 49106 C40547-04, 1-SI-158-L3
 Address Repair Org. P.O. No., Job No., etc
3. Work Performed by Maintenance Department Type Code Symbol Stamp N/A
 Name Authorization No. N/A
Same as #2 Expiration Date N/A
 Address
4. Identification of System Safety Injection
5. (a) Applicable Const. Code ANSI B31.1 19 83Ed., N/A Add. N/A Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1989
No Addenda

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manuf. Serial No.	Nat. Board No.	Other Identification	Year Built	Repaired Replaced or Replacement	ASME Code Stamped (Yes or No)
1-SI-158-L3							
cage				M&E#: 30043794 ASP#16711	n/a	replace	NO
Studs: SA-564-gr630				M&E# 30044105 ASP# 19669	n/a	replace	NO
Nuts				M&E# 30044090 ASP19669	n/a	replace	NO

7. Description of Work (See Remarks)
8. Test Conducted: Hydrostatic _____ Pneumatic _____ Nominal Operating Pressure _____
N/A Other VT-1/2/3 Pressure _____ psi Test Temp. _____ °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided
 (1) size is 8-1/2 in. x 11 in., (2) information in items 1 through 6 on this report is

FORM NIS-2 (Back)

9. Remarks Replaced the cage, fasteners and disc
Applicable Manufacturer's Data Reports to be attached

Ref. JO:C40545-04 File: 1-SI-158-L2 ISI Class: 1

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code, Section XI.
 repair or replacement

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed Frank Pisarsky, Maint. Eng. Supervisor Date 7/2, 1997
 Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Michigan and employed by ARKWRIGHT MUTUAL INS. CO.* of NORWOOD, MASS. have inspected the components described in this Owner's Report during the period 4/15/97 to 7/15/97, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Michael J. Matasavage Commissions Michigan, ENR NSE
 Inspector's Signature National Board, State, Province, Endorsements

Date July 15 19 97



1. Owner AMERICAN ELECTRIC POWER Date 04-16-97
 Name
P.O. Box 60 Fort Wayne, IN 46801 Sheet 1 of 2
 Address
2. Plant D.C. COOK NUCLEAR POWER PLANT Unit # 1
 Name
One Cook Place, Bridgman, MI 49106 C40545-04, 1-SI-158-L2
 Address Repair Org. P.O. No., Job No., etc
3. Work Performed by Maintenance Department Type Code Symbol Stamp N/A
 Name Authorization No. N/A
Same as #2 Expiration Date N/A
 Address
4. Identification of System Safety Injection
5. (a) Applicable Const. Code ANSI B31.1 19 83Ed., N/A Add. N/A Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1989,
No Addenda
6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manuf. Serial No.	Nat. Board No.	Other Identification	Year Built	Repaired Replaced or Replacement	ASME Code Stamped (Yes or No)
1-SI-158-L2							
cage				M&E#: 30043794 ASP#16711	n/a	replace	NO
Studs: SA-564-gr630				M&E# 30044105 ASP# 19669	n/a	replace	NO
Nuts				M&E# 30044090 ASP19669	n/a	replace	NO
Disc				M&E# 30043955 ASP#17244	n/a	replace	NO

7. Description of Work (See Remarks)
8. Test Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
 N/A Other VT-1/2/3 Pressure psi Test Temp. °F



FORM NIS-2 (Back)

9. Remarks Replaced studs and nuts at body to bonnet joint
 Applicable Manufacturer's Data Reports to be attached

Ref. JO:C40031-04 File: 1-SI-158-L1 ISI Class: 1

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this
replacement conforms to the rules of the ASME Code, Section XI.
 repair or replacement

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed Frank Pisarsky Maint. Eng. Supervisor Date 6/30, 19 91
 Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Michigan and employed by ARKWRIGHT MUTUAL INS. CO.* of NORWOOD, MASS. have inspected the components described in this Owner's Report during the period 3-17-97 to 7-3-97, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Mr. [Signature] Commissions Mich. 2011 ENR NIT
 Inspector's Signature National Board, State, Province, Endorsements

Date July 3 19 97

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner AMERICAN ELECTRIC POWER Date 03-18-97
Name
P.O. Box 60 Fort Wayne, IN 46801 Sheet 1 of 2
Address
2. Plant D.C. COOK NUCLEAR POWER PLANT Unit # 1
Name
One Cook Place, Bridgman, MI 49106 C40031-04, 1-SI-158-L1, VLV
Address Repair Org. P.O. No., Job No., etc
3. Work Performed by Maintenance Department Type Code Symbol Stamp N/A
Name Authorization No. N/A
Same as #2 Expiration Date N/A
Address
4. Identification of System Safety Injection
5. (a) Applicable Const. Code ANSI B31.1 19 83Ed., N/A Add. N/A Code Case
(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1989,
No Addenda
6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manuf. Serial No.	Nat. Board No.	Other Identification	Year Built	Repaired Replaced or Replacement	ASME Code Stamped (Yes or No)
<u>1-SI-158-L1</u>							
Studs: SA-564 gr630	n/a	n/a	n/a	M&E# 30044105 ASP# 5562	n/a	replace	NO
Nuts: SA-194 gr.8M	N/a	n/a	n/a	M&E# 30044090 ASP# 5562	n/a	replace	NO

7. Description of Work (See Remarks)
8. Test Conducted: Hydrostatic _____ Pneumatic _____ Nominal Operating Pressure _____
N/A _____ Other VT-1/2/3 Pressure _____ psi Test Temp. _____ °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks Recondition inbody seating surface in order to achieve a
Applicable Manufacturer's Data Reports to be attached
satisfactory bluing of seating surface with Conval manual
seat cutting tool
 Ref. JO:C24126-3 File: 1-SI-130 ISI Class: 2

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this
repair conforms to the rules of the ASME Code, Section XI.
 repair or replacement

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed Frank Pisarsky Maint. Eng. Supervisor Date 4/9, 1997
 Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Michigan and employed by ARKWRIGHT MUTUAL INS. CO.* of NORWOOD, MASS. have inspected the components described in this Owner's Report during the period 3/17/97 to 4/10/97, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

M. R. Muterbaugh Commissions Mich 0055, EOD NIT
 Inspector's Signature National Board, State, Province, Endorsements

Date April 10 1997



FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner AMERICAN ELECTRIC POWER Date 03-17-97
Name
P.O. Box 60 Fort Wayne, IN 46801 Sheet 1 of 2
Address
2. Plant D.C. COOK NUCLEAR POWER PLANT Unit # 1
Name
One Cook Place, Bridgman, MI 49106 C24126-3, 1-SI-130, vlv
Address Repair Org. P.O. No., Job No., etc
3. Work Performed by Maintenance Department Type Code Symbol Stamp N/A
Name Authorization No. N/A
Same as #2 Expiration Date N/A
Address
4. Identification of System Safety Injection
5. (a) Applicable Const. Code ANSI B31.1 19 83Ed., N/A Add. N/A Code Case
(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1989,
No Addenda

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manuf. Serial No.	Nat. Board No.	Other Identification	Year Built	Repaired Replaced or Replacement	ASME Code Stamped (Yes or No)
1-SI-130							
vlv body w/ integral seat ring sA-182 f316 w/Stell.# 21	VELAN			existing	n/a	repair	NO

7. Description of Work (See Remarks)
8. Test Conducted: Hydrostatic _____ Pneumatic _____ Nominal Operating Pressure _____
N/A ☒ Other _____ Pressure _____ psi Test Temp. _____ °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.



FORM NIS-2 (Back)

9. Remarks Replaced the existing carbon steel body/bonnet stud with
Applicable Manufacturer's Data Reports to be attached
new stainless steel stud

Ref. JO:C34037-01 File: 1-RH-128E ISI Class: 2

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this
replacement conforms to the rules of the ASME Code, Section XI.
 repair or replacement

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed Frank Pisarsky, Maint. Eng. Supervisor Date 6/30, 1997
 Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Michigan and employed by ARKWRIGHT MUTUAL INS. CO.* of NORWOOD, MASS. have inspected the components described in this Owner's Report during the period 4-2-97 to 7-3-97, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Mike Matras Commissions MICH 0055, END N&I
 Inspector's Signature National Board, State, Province, Endorsements

Date July 3 19 97

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner AMERICAN ELECTRIC POWER Date 03-22-97
Name
P.O. Box 60 Fort Wayne, IN 46801 Sheet 1 of 2
Address
2. Plant D.C. COOK NUCLEAR POWER PLANT Unit # 1
Name
One Cook Place, Bridgman, MI 49106 C34037-01, 1-RH-128E, valve
Address Repair Org. P.O. No., Job No., etc
3. Work Performed by Maintenance Department Type Code Symbol Stamp N/A
Name Authorization No. N/A
Same as #2 Expiration Date N/A
Address
4. Identification of System Residual Heat Removal
5. (a) Applicable Const. Code ANSI B31.1 19 83Ed., N/A Add. N/A Code Case
(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1989,
No Addenda
6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manuf. Serial No.	Nat. Board No.	Other Identification	Year Built	Repaired Replaced or Replacement	ASME Code Stamped (Yes or No)
1-RH-128E							
Stud: SA-453 gr.660	n/a	n/a	n/a	M&E# 30212295 ASP#16300	n/a	replace	NO

7. Description of Work (See Remarks)
8. Test Conducted: Hydrostatic _____ Pneumatic _____ Nominal Operating Pressure _____
N/A _____ Other VT-1/2/3 Pressure _____ psi Test Temp. _____ °F.

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.



FORM NIS-2 (Back)

Remarks

Applicable Manufacturer's Data Reports to be attached

Ref. JO: C38052-04 DESIGN CHANGE: MM-404

REPAIR/REPLACEMENT TRAVELLER: C38052-04 ISI CLASS: 1

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this
REPLACEMENT conforms to the rules of the ASME Code, Section XI.
 repair or replacement

Type Code Symbol Stamp N/ACertificate of Authorization No. N/A Expiration Date N/A

Signed Wayne Cretsinger, Senior Planner
 Owner or Owner's Designee, Title

Date APRIL 15, 1997

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Michigan and employed by ARKWRIGHT MUTUAL INS. CO. of WALTHAM, MASS. have inspected the components described in this Owner's Report during the period 3-15-97 to 5-5-97, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Inspector's Signature

Commissions MICHOOS, END N.Y.I.
 National Board, State, Province, Endorsements

Date May 5, 1997

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

NIS-2-133-97

1. Owner AMERICAN ELECTRIC POWER COMPANY Date 04/15/97
 Name
P.O. Box 60 Fort Wayne, IN 46801 Sheet 1 of 2
 Address
2. Plant D.C. COOK NUCLEAR POWER PLANT Unit # 1
 Name
One Cook Place, Bridgman, MI 49106 C38052-04
 Address Repair Org. P.O. No., Job No., etc
3. Work Performed by Installation Services Type Code Symbol Stamp N/A
 Name Authorization No. N/A
Same as #2 Expiration Date N/A
 Address
4. Identification of System REACTOR COOLANT ASME CODE CLASS 1
5. (a) Applicable Const. Code ANSI B31.1 19 83Ed., N/A Add. N-416-1 Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1989
6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manuf. Serial No.	Nat. Board No.	Other Identification	Year Built	Repaired Replaced or Replacement	ASME Code Stamped (Yes or No)
1-RC-111-L1 2" SCH 160 PIPE	M&E# 30-019310	ASP# 14295	N/A	SA376 TP 304	N/A	REPLACE- MENT	NO
1-RC-111-L1 2" 1500# SS GLOBE VALVE	M&E# 30-035209	ASP# 14119	N/A	SA182 F 316	N/A	REPLACE- MENT	NO

7. Description of Work REPLACE GLOBE VALVE 1-RC-111-L1 PER MM-404

8. Test Conducted: Hydrostatic ☐ Pneumatic ☐ Nominal Operating Pressure ☒
 N/A ☐ Other ☐ Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks Remove forging indication by grinding/buffing. No welding
Applicable Manufacturer's Data Reports to be attached
required

Ref. JO:C40235-1 File: 1-RC-108-L2 ISI Class: 1

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this
repair conforms to the rules of the ASME Code, Section XI.
 repair or replacement

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed Frank Pisarsky, Maint. Eng. Supervisor Date 4/3, 19 97
 Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Michigan and employed by ARKWRIGHT MUTUAL INS. CO.* of NORWOOD, MASS. have inspected the components described in this Owner's Report during the period 3/25/97 to 4/11/97, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

MR. Matuszewska Commissions MA 0055, END N&I
 Inspector's Signature National Board, State, Province, Endorsements

Date April 11 19 97

~~*FACTORY MUTUAL SYSTEM~~

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner INDIANA MICHIGAN POWER COMPANY Date 03-25-97
Name
P.O. Box 60 Fort Wayne, IN 46801 Sheet 1 of 2
Address
2. Plant D.C. COOK NUCLEAR POWER PLANT Unit # 1
Name
One Cook Place, Bridgman, MI 49106 C40235-1, 1-RC-108-L2
Address Repair Org. P.O. No., Job No., etc
3. Work Performed by Maintenance Department Type Code Symbol Stamp N/A
Name Authorization No. N/A
Same as #2 Expiration Date N/A
Address
4. Identification of System Reactor Coolant
5. (a) Applicable Const. Code ANSI B31.1 19 83Ed., N/A Add. N/A Code Case
(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1989, No Addenda

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manuf. Serial No.	Nat. Board No.	Other Identification	Year Built	Repaired Replaced or Replacement	ASME Code Stamped (Yes or No)
1-RC-108-L2 Body:SA-182 F316				existing	n/a	repair	NO

7. Description of Work (See Remarks)
8. Test Conducted: Hydrostatic _____ Pneumatic _____ Nominal Operating Pressure _____
N/A Other PT Pressure _____ psi Test Temp. _____ °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks Replaced the pilot valve and stem assembly mechanically
Applicable Manufacturer's Data Reports to be attached

Ref. JO: R35549-17 File:1-OT-506

ISI Class: 2

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this
replacement conforms to the rules of the ASME Code, Section XI.
 repair or replacement

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed Frank Pisarsky, Maint. Eng. Supervisor Date 4/3, 19 97
 Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Michigan and employed by ARKWRIGHT MUTUAL INS. CO.* of NORWOOD, MASS. have inspected the components described in this Owner's Report during the period 10/19/97 to 4/9/97, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

W. A. Whitcomb Commissions Mich 0053, ENR N.Y.I
 Inspector's Signature National Board, State, Province, Endorsements

*FACTORY MUTUAL SYSTEM

Date April 9 19 97



FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner INDIANA MICHIGAN POWER COMPANY Date 10-19-95
Name
P.O. Box 60 Fort Wayne, IN 46801 Sheet 1 of 2
Address
2. Plant D.C. COOK NUCLEAR POWER PLANT Unit # 1
Name
One Cook Place, Bridgman, MI 49106 R35549-17, 1-OT-506, 4"vly
Address Repair Org. P.O. No., Job No., etc
3. Work Performed by Maintenance Department Type Code Symbol Stamp N/A
Name Authorization No. N/A
Same as #2 Expiration Date N/A
Address
4. Identification of System Auxiliary Feedwater system
5. (a) Applicable Const. Code ANSI B31.1 19 83Ed., N/A Add. N/A Code Case
(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1983 Ed.,
Summer Addenda, 1983

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manuf. Serial No.	Nat. Board No.	Other Identification	Year Built	Repaired Replaced or Replacement	ASME Code Stamped (Yes or No)
1-OT-506							
Pilot valve & stem assy	Dresser Rand			M&E#: 30-049395 ASP#12672	N/A	REPLACE	NO

7. Description of Work (See Remarks)
8. Test Conducted: Hydrostatic _____ Pneumatic _____ Nominal Operating Pressure _____
N/A ☒ Other _____ Pressure _____ psi Test Temp. _____ °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks Mechanically replaced the valve's plug
 Applicable Manufacturer's Data Reports to be attached

Ref. JO:R36017-2 File: 1-ORV-62 ISI Class: 1

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this
replacement conforms to the rules of the ASME Code, Section XI.
 repair or replacement

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed Frank Pisarsky Date 4/3, 1997
 Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Michigan and employed by ARKWRIGHT MUTUAL INS. CO.* of NORWOOD, MASS. have inspected the components described in this Owner's Report during the period 11/2/94 to 4/14/97, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

W.R. Waterhouse Commissions Mich 0035, ENR N&E
 Inspector's Signature National Board, State, Province, Endorsements

Date April 14 19 97

*FACTORY MUTUAL SYSTEM

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner AMERICAN ELECTRIC POWER Date 03-23-97
Name
P.O. Box 60 Fort Wayne, IN 46801 Sheet 1 of 2
Address
2. Plant D.C. COOK NUCLEAR POWER PLANT Unit # 1
Name
One Cook Place, Bridgman, MI 49106 R36017-2, 1-ORV-62, vlv.
Address Repair Org. P.O. No., Job No., etc
3. Work Performed by Maintenance Department Type Code Symbol Stamp N/A
Name Authorization No. N/A
Same as #2 Expiration Date N/A
Address
4. Identification of System Chemical Volume & Control System
5. (a) Applicable Const. Code ANSI B31.1 19 83Ed., N/A Add. N/A Code Case
(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1989
No Addenda
6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manuf. Serial No.	Nat. Board No.	Other Identification	Year Built	Repaired Replaced or Replacement	ASME Code Stamped (Yes or No)
1-QRV-62							
plug ASTM A-276 Gr. 316	Masoneilan			M&E# 30-042060 ASP#17085	n/a	replace	NO

7. Description of Work (See Remarks)
8. Test Conducted: Hydrostatic _____ Pneumatic _____ Nominal Operating Pressure _____
N/A ☒ Other _____ Pressure _____ psi Test Temp. _____ °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks Mechanically replaced the valve plug
Applicable Manufacturer's Data Reports to be attached

Ref. JO:R36016-2 File: 1-ORV-61 ISI Class: 1

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code, Section XI.
 repair or replacement

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed Frank Pisarsky Maint. Eng. Supervisor Date 4/3, 1997
 Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Michigan and employed by ARKWRIGHT MUTUAL INS. CO.* of NORWOOD, MASS. have inspected the components described in this Owner's Report during the period 11/2/94 to 4/11/97, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

W. H. Matson Commissions MICHOSS, ENDRSE
 Inspector's Signature National Board, State, Province, Endorsements

*FACTORY MUTUAL SYSTEM

Date April 11 1997

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner INDIANA MICHIGAN POWER COMPANY Date 03-18-97
Name
P.O. Box 60 Fort Wayne, IN 46801 Sheet 1 of 2
Address
2. Plant D.C. COOK NUCLEAR POWER PLANT Unit # 1
Name
One Cook Place, Bridgman, MI 49106 R36016-2, 1-ORV-61, vlv
Address Repair Org. P.O. No., Job No., etc
3. Work Performed by Maintenance Department Type Code Symbol Stamp N/A
Name Authorization No. N/A
Same as #2 Expiration Date N/A
Address
4. Identification of System Chemical Volume & Control System
5. (a) Applicable Const. Code ANSI B31.1 19 83Ed., N/A Add. N/A Code Case
(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1989, No Addenda
6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manuf. Serial No.	Nat. Board No.	Other Identification	Year Built	Repaired Replaced or Replacement	ASME Code Stamped (Yes or No)
1-ORV-61							
Plug ASTM A-276	Mssoneilan			M&e#: 30-042060 ASP#9933	n/a	replace	NO

7. Description of Work (See Remarks)
8. Test Conducted: Hydrostatic _____ Pneumatic _____ Nominal Operating Pressure _____
N/A ☒ Other _____ Pressure _____ psi Test Temp. _____ °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks Replaced valve and adjacent piping by welding per DCP-7
Applicable Manufacturer's Data Reports to be attached

Ref. JO:C28397-2 File: 1-OMO-226 ISI Class: 2

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this
replacement conforms to the rules of the ASME Code, Section XI.
 repair or replacement

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed Frank Pisarsky, Maint. Eng. Supervisor Date 7/8, 1997
 Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Michigan and employed by ARKWRIGHT MUTUAL INS. CO.* of NORWOOD, MASS. have inspected the components described in this Owner's Report during the period 8/11/95 to 7/16/97, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

MR. [Signature] Commissions MICHIGAN, ENDSIDE
 Inspector's Signature National Board, State, Province, Endorsements

Date July 16 1997

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner AMERICAN ELECTRIC POWER Date 03-17-97
Name _____
- P.O. Box 60 Fort Wayne, IN 46801 Sheet 1 of 2
Address _____
2. Plant D.C. COOK NUCLEAR POWER PLANT Unit _____ # 1
Name _____
- One Cook Place, Bridgman, MI 49106 C28397-2.11, 1-OMO-226, DCP-7
Address _____ Repair Org. P.O. No., Job No., etc _____
3. Work Performed by Maintenance Department Type Code Symbol Stamp N/A
Name _____ Authorization No. N/A
Same as #2 Expiration Date N/A
Address _____
4. Identification of System _____ Chemical Volume & Control
5. (a) Applicable Const. Code ANSI B31.1 19 83Ed., N/A Add. N-416-1 Code Case
(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1989,
No Addenda

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manuf. Serial No.	Nat. Board No.	Other Identification	Year Built	Repaired Replaced or Replacement	ASME Code Stamped (Yes or No)
1-QMO-226 SA-351 CF8M				P.O.# 02011- 041-5 ASP#18583	n/a	replace	NO
pipe 2" SA-376 tp304				M&E# 30016812 ASP#7762	n/a	replace	NO
2" elbow SA-403 wp304				P.O.# 66128- 042-3 ASP#18196	n/a	replace	NO
2"elbow SA-182 F304				M&E# 30017265 ASP#7559	n/a	replace	NO

7. Description of Work _____ (See Remarks)
8. Test Conducted: Hydrostatic _____ Pneumatic _____ Nominal Operating Pressure _____
VT/Surface Other VT-2/RT Pressure _____ psi Test Temp. _____ °F

7. Description of Work (See Remarks)
8. Test Conducted: Hydrostatic _____ Pneumatic _____ Nominal Operating Pressure _____
 VT/Surface Other VT-2/RT Pressure _____ psi Test Temp. _____ °F
 FORM NIS-2 (Back)
9. Remarks Replaced valve and adjacent piping by welding per DCP-7
Applicable Manufacturer's Data Reports to be attached
Repairs were performed on the prefabbed assembly to remove
surface indications.
Ref. JO:C28393-2 File: 1-OMO-225 ISI Class: 2

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement/repair conforms to the rules of the ASME Code, Section XI.
 repair or replacement

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed Frank Pisarsky Maint. Eng. Supervisor Date 7/8, 19 77
 Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Michigan and employed by ARKWRIGHT MUTUAL INS. CO.* of NORWOOD, MASS. have inspected the components described in this Owner's Report during the period 8/11/75 to 7/16/77, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Michael J. McCross Commissions Michigan, ENR, NFI
 Inspector's Signature National Board, State, Province, Endorsements

Date July 16 19 77

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner AMERICAN ELECTRIC POWER Date 03-08-97
Name
P.O. Box 60 Fort Wayne, IN 46801 Sheet 1 of 2
Address
2. Plant D.C. COOK NUCLEAR POWER PLANT Unit # 1
Name
One Cook Place, Bridgman, MI 49106 C28393-2.11. 1-OMO-225, DCP-7
Address Repair Org. P.O. No., Job No., etc
3. Work Performed by Maintenance Department Type Code Symbol Stamp N/A
Name Authorization No. N/A
Same as #2 Expiration Date N/A
Address
4. Identification of System Chemical Volume & Control
5. (a) Applicable Const. Code ANSI B31.1 19 83Ed., N/A Add. N-416-1 Code Case
(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1989,
No Addenda

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manuf. Serial No.	Nat. Board No.	Other Identification	Year Built	Repaired Replaced or Replacement	ASME Code Stamped (Yes or No)
1-QMO-225 SA-351 CF8M				P.O.# 02011- 041-5 ASP#18583	n/a	replace	NO
2" tee SA-182 F304				M&E# 30017321 ASP#1127	n/a	replace	NO
insert SA-182 F304 2"x 3/4"				M&E# 30017287 ASP#1127	n/a	replace	NO
pipe 2" SA-376 tp304				M&E# 30016812 ASP#7762	n/a	replace	NO
2" elbow SA-403 wp304				P.O.# 66128- 042-3 ASP#18196	n/a	replace	NO
2"elbow SA-182 F304				M&E# 30017265 ASP#7559	n/a	replace	NO

FORM NIS-2 (Back)

9. Remarks Replaced valve per DCP-7 and MM-526 by welding
Applicable Manufacturer's Data Reports to be attached

Ref. JO:C28167-2 File: 1-OMO-201 ISI Class: 1

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this
replacement conforms to the rules of the ASME Code, Section XI.
 repair or replacement

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed Frank Pisarsky Maint. Eng. Supervisor Date 7/8, 19 97
 Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Michigan and employed by ARKWRIGHT MUTUAL INS. CO.* of NORWOOD, MASS. have inspected the components described in this Owner's Report during the period 8-11-95 to 7-16-97, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

M. R. Whitcomb Commissions Mich, MASS, END NBT
 Inspector's Signature National Board, State, Province, Endorsements

Date July 16 19 97

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner AMERICAN ELECTRIC POWER Date 03-20-97
Name _____
P.O. Box 60 Fort Wayne, IN 46801 Sheet 1 of 2
Address _____
2. Plant D.C. COOK NUCLEAR POWER PLANT Unit _____ # 1
Name _____
One Cook Place, Bridgman, MI 49106 C28167-02, 1-OMO-201, DCP-7
Address _____ Repair Org. P.O. No., Job No., etc _____
3. Work Performed by Maintenance Department Type Code Symbol Stamp N/A
Name _____ Authorization No. N/A
Same as #2 Expiration Date N/A
Address _____
4. Identification of System _____ Chemical Volume & Control _____
5. (a) Applicable Const. Code ANSI B31.1 19 83Ed., N/A Add. N-416-1 Code Case _____
(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1989,
No Addenda

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manuf. Serial No.	Nat. Board No.	Other Identification	Year Built	Repaired Replaced or Replacement	ASME Code Stamped (Yes or No)
1-QMO-201 SA-351 CF8M				P.O.# 02011- 041-5 ASP#18583	n/a	replace	NO

7. Description of Work _____ (See Remarks) _____
8. Test Conducted: Hydrostatic _____ Pneumatic _____ Nominal Operating Pressure _____
Vt/RT Other VT-2 Pressure _____ psi Test Temp. _____ °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.



FORM NIS-2 (Back)

9. Remarks Replaced valve & adjacent piping and remove/replace parts of
Applicable Manufacturer's Data Reports to be attached
adjacent support 1-GCS-R83 per DCP-7/MM-526

Ref. JO:C28166-2 File: 1-OMO-200 ISI Class: 1

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this
replacement conforms to the rules of the ASME Code, Section XI.
 repair or replacement

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed Frank Pisarsky, Maint. Eng. Supervisor Date 7/8, 1997
 Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Michigan and employed by ARKWRIGHT MUTUAL INS. CO.* of NORWOOD, MASS. have inspected the components described in this Owner's Report during the period 1-17-96 to 7-16-97, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

HR Hultquist Commissions Mich 0055, End N&I
 Inspector's Signature National Board, State, Province, Endorsements

Date July 16 19 97

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner AMERICAN ELECTRIC POWER Date 03-19-97
Name _____
- P.O. Box 60 Fort Wayne, IN 46801 Sheet 1 of 2
Address _____
2. Plant D.C. COOK NUCLEAR POWER PLANT Unit _____ # 1
Name _____
- One Cook Place, Bridgman, MI 49106 C28166-2.19 1-OMO-200 DCP-7
Address _____ Repair Org. P.O. No., Job No., etc _____
3. Work Performed by Maintenance Department Type Code Symbol Stamp N/A
Name _____ Authorization No. N/A
Same as #2 Expiration Date N/A
Address _____
4. Identification of System _____ Chemical Volume and Control _____
5. (a) Applicable Const. Code ANSI B31.1 19 83Ed., N/A Add. N-416-1 Code Case
(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1989
No Addenda
6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manuf. Serial No.	Nat. Board No.	Other Identification	Year Built	Repaired Replaced or Replacement	ASME Code Stamped (Yes or No)
1-QMO-200 SA-351 grCF8M				P.O.# 02011- 041-5 ASP#18583	n/a	replaced	NO
Plate SA-36 for support 1-GCS-R83				M&E# 30153192 ASP#15485	n/a	replace	NO
pipe 2" for support SA-312 tp304				M&E# 30016711 ASP#12983	n/a	replace	NO
pipe 3" SA-376 tp304				P.O.# 66174- 042-5 ASP#18075	n/a	replace	NO

7. Description of Work _____ (See Remarks) _____
8. Test Conducted: Hydrostatic _____ Pneumatic _____ Nominal Operating Pressure _____
RT/VT/VT-3 Other VT-2 Pressure _____ psi Test Temp. _____ °F

FORM NIS-2 (Back)

9. Remarks Replace # 2 seal housing boltingApplicable Manufacturer's Data Reports to be attachedRef. JO:R51617-06 File: 1-PP-45-3 ISI Class: 1

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this
replacement conforms to the rules of the ASME Code, Section XI.
 repair or replacement

Type Code Symbol Stamp N/ACertificate of Authorization No. N/A Expiration Date N/A

Signed Frank Pisarsky, Maint. Eng. Supervisor Date 6/22/97, 19 97
 Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Michigan and employed by ARKWRIGHT MUTUAL INS. CO.* of NORWOOD, MASS. have inspected the components described in this Owner's Report during the period 3-15-97 to 6-27-97, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

MR. [Signature] Commissions MICH 0055, END NBI
 Inspector's Signature National Board, State, Province, Endorsements

Date JUNE 27 19 97

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS

As Required by the Provisions of the ASME Code Section XI

1. Owner AMERICAN ELECTRIC POWER Date 03-25-97
 Name
P.O. Box 60 Fort Wayne, IN 46801 Sheet 1 of 2
 Address
2. Plant D.C. COOK NUCLEAR POWER PLANT Unit # 1
 Name
One Cook Place, Bridgman, MI 49106 R51617-06, 1-PP-45-3, pump
 Address Repair Org. P.O. No., Job No., etc
3. Work Performed by Maintenance Department Type Code Symbol Stamp N/A
 Name Authorization No. N/A
Same as #2 Expiration Date N/A
 Address
4. Identification of System Reactor Coolant
5. (a) Applicable Const. Code ANSI B31.1 19 83Ed., N/A Add. N/A Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1989,
No Addenda

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manuf. Serial No.	Nat. Board No.	Other Identification	Year Built	Repaired Replaced or Replacement	ASME Code Stamped (Yes or No)
1-PP-45-3							
bolting				30-09262 ASP#16265	n/a	replace	NO

7. Description of Work (See Remarks)
8. Test Conducted: Hydrostatic _____ Pneumatic _____ Nominal Operating Pressure _____
 N/A _____ Other VT-1/2 Pressure _____ psi Test Temp. _____ °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks Replaced three (3) core barrel/former plate bolts at the A4,
Applicable Manufacturer's Data Reports to be attached

A5, and A6 locations. Design Change Package DCP-125

ISI Class: 1 (Reactor)

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this
replacement conforms to the rules of the ASME Code, Section XI.
 repair or replacement

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed Frank Pisarsky, Maint. Eng. Supervisor Date 7/17, 1997

Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Michigan and employed by ARKWRIGHT MUTUAL INS. CO.* of NORWOOD, MASS. have inspected the components described in this Owner's Report during the period 3/22/97 to 7/17/97, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

M. J. Winterpach Commissions Mich 0055, ENOPII
 Inspector's Signature National Board, State, Province, Endorsements

Date July 17 19 97

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner INDIANA MICHIGAN POWER Date 3-30-97
Name _____
- P.O. Box 60 Fort Wayne, IN 46801 Sheet 1 of 2
Address _____
2. Plant D.C. COOK NUCLEAR POWER PLANT Unit _____ # 1
Name _____
- One Cook Place, Bridgman, MI 49106 J.O. C38602-07
Address _____ Repair Org. P.O. No., Job No., etc _____
3. Work Performed by WESTINGHOUSE CORP. Type Code Symbol Stamp N/A
Name _____ Authorization No. N/A
Address _____ Expiration Date N/A
4. Identification of System Reactor Vessel Lower Internals
(Fabricated to Westinghouse Specification)
5. (a) Applicable Const. Code N/A 19 _____ Ed., _____ Add. _____ Code Case
(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1989
No Addenda
6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manuf. Serial No.	Nat. Board No.	Other Identification	Year Built	Repaired Replaced or Replacement	ASME Code Stamped (Yes or No)
1-OME-1 Lower Internals	Westinghouse	CONV: 000086 69 (RV)	N/A	—	1971	—	No
Former bolts	Westinghouse		N/A	SA 479 Type 316	1997	Replacement	No
	Westinghouse		N/A	A193 Gr.B8C Type 347		Replaced	No

7. Description of Work (See Remarks)
8. Test Conducted: Hydrostatic _____ Pneumatic _____ Nominal Operating Pressure _____
N/A X Other _____ Pressure _____ psi Test Temp. _____ °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks Replaced the trim assembly which included the plug which was
Applicable Manufacturer's Data Reports to be attached
also machined and replaced the bonnet/body/extension fasteners
Also machined the Bonnet Extension.
Ref. JO:R20880-3 File: 1-NRV-164 ISI Class: 1

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this
replacement conforms to the rules of the ASME Code, Section XI.
 repair or replacement

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed Frank Pisarsky Date 6/30, 1997
 Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Michigan and employed by ARKWRIGHT MUTUAL INS. CO.* of NORWOOD, MASS. have inspected the components described in this Owner's Report during the period 3-18-97 to 7-3-97, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

MR. [Signature] Commissions Michigan, END NIT
 Inspector's Signature National Board, State, Province, Endorsements

Date July 3 19 97



7. Description of work _____ (see remarks)
8. Test Conducted: Hydrostatic _____ Pneumatic _____ Nominal Operating Pressure 200
PT Other VT-1/2 Pressure _____ psi Test Temp. _____ °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner AMERICAN ELECTRIC POWER Date 03-22-97
Name
P.O. Box 60 Fort Wayne, IN 46801 Sheet 1 of 2 3 440
Address
2. Plant D.C. COOK NUCLEAR POWER PLANT Unit # 1
Name
One Cook Place, Bridgman, MI 49106 R20880-3.10:1-NRV-164, vlv
Address Repair Org. P.O. No., Job No., etc.
3. Work Performed by Maintenance Department Type Code Symbol Stamp N/A
Name Authorization No. N/A
Same as #2 Expiration Date N/A
Address
4. Identification of System Reactor Coolant
5. (a) Applicable Const. Code ANSI B31.1 19 83Ed., N/A Add. N/A Code Case
(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1989,
No Addenda

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manuf. Serial No.	Nat. Board No.	Other Identification	Year Built	Repaired Replaced or Replacement	ASME Code Stamped (Yes or No)
1-NRV-164							
Plug:SA-564 gr. 630 (trim assy)				M&E#: 30-036074 ASP#15939	n/a	replace machine	NO
All thread SA-453 gr660				M&E#: 30-212315 ASP#18021	n/a	replace	NO
All thread SA-453 gr660				M&E#: 30-030001 ASP#12987	n/a	replace	NO
Nuts				M&E# 30-212142 ASP#16322	n/a	replace	NO
Bonnet ext. SA-351 grCF8M				existing	n/a	repair	NO
Nuts				M&E# 30-046906 ASP#12915	n/a	replace	NO

FORM NIS-2 (Back)

9. Remarks Machined the existing Bonnet extension and replaced fastener
Applicable Manufacturer's Data Reports to be attached
material and internals.

Ref. JO:C34692-3 File: 1-NRV-163 ISI Class: 1

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement/repair conforms to the rules of the ASME Code, Section XI.
 repair or replacement

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed Frank Pisarsky, Maint. Eng. Supervisor Date 7/17, 1997
 Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Michigan and employed by ARKWRIGHT MUTUAL INS. CO.* of NORWOOD, MASS. have inspected the components described in this Owner's Report during the period 2/29/96 to 7/17/97, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

MR. Winterpough Commissions MICH 0055, END NFI
 Inspector's Signature National Board, State, Province, Endorsements

Date July 17 1997

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner AMERICAN ELECTRIC POWER Date 03-24-97
Name
P.O. Box 60 Fort Wayne, IN 46801 Sheet 1 of 2
Address
2. Plant D.C. COOK NUCLEAR POWER PLANT Unit # 1
Name
One Cook Place, Bridgman, MI 49106 C34692-03.10. 1-NRV-163 vlv.
Address Repair Org. P.O. No., Job No., etc
3. Work Performed by Maintenance Department Type Code Symbol Stamp N/A
Name Authorization No. N/A
Same as #2 Expiration Date N/A
Address
4. Identification of System Reactor Coolant
5. (a) Applicable Const. Code ANSI B31.1 19 83Ed., N/A Add. N/A Code Case
(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1989
No Addenda
6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manuf. Serial No.	Nat. Board No.	Other Identification	Year Built	Repaired Replaced or Replacement	ASME Code Stamped (Yes or No)
1-NRV-163							
Bonnet ext. ASTM A351 gr. CF8M	Copes Vulcan			existing	n/a	repair	NO
Cage/plug ASTM A654 gr.630 tp17-4	Copes Vulcan			M&E# 30036074 ASP#15939	n/a	replace	NO
All thread ASTM A453 GR. 660				M&E#: 30030001 ASP#12987	n/a	replace	NO

7. Description of Work (See Remarks)
8. Test Conducted: Hydrostatic _____ Pneumatic _____ Nominal Operating Pressure _____
VT-1/ PT/ VT-2 Other _____ Pressure _____ psi Test Temp. _____ °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in. x 11 in., (2) information in items 1 through 6 on this report is m.

FORM NIS-2 (Back)

Remarks mechanically replaced the seat ring, stem plug and plug on valve
 Applicable Manufacturer's Data Reports to be attached

Ref. JO:C33536-4 File: 1-NRV-153 ISI Class: 1

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this
replacement conforms to the rules of the ASME Code, Section XI.
 repair or replacement

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed Frank Pisarsky, Maint. Eng. Supervisor Date 7/8, 1997
 Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Michigan and employed by ARKWRIGHT MUTUAL INS. CO.* of NORWOOD, MASS. have inspected the components described in this Owner's Report during the period 12-15-95 to 7-16-97, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

[Signature] Commissions Michoors, EN & NJ-I
 Inspector's Signature National Board, State, Province, Endorsements

*FACTORY MUTUAL SYSTEM

Date July 16 1997

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner INDIANA MICHIGAN POWER COMPANY Date 11-07-95
Name
P.O. Box 60 Fort Wayne, IN 46801 Sheet 1 of 2
Address
2. Plant D.C. COOK NUCLEAR POWER PLANT Unit # 1
Name
One Cook Place, Bridgman, MI 49106 C33537-4 1-NRV-153
Address Repair Org. P.O. No., Job No., etc
3. Work Performed by Maintenance Department Type Code Symbol Stamp N/A
Name Authorization No. N/A
Same as #2 Expiration Date N/A
Address
4. Identification of System Pressurizer system
5. (a) Applicable Const. Code ANSI B31.1 19 83Ed., N/A Add. N/A Code Case
(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1983, Summer 1983 Addenda

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manuf. Serial No.	Nat. Board No.	Other Identification	Year Built	Repaired Replaced or Replacement	ASME Code Stamped (Yes or No)
1-NRV-153							
stem plug	Masoneilan	n/a	n/a	M&E#: 30042250 ASP#17999	n/a	replace	NO
Plug	Masoneilan	n/a	n/a	M&E# 30042070 ASP#17675	n/a	replace	NO
Seat Ring	Masoneilan	n/a	n/a	M&E# 30042230 ASP#15268	n/a	relace	NO

7. Description of Work (See Remarks)
8. Test Conducted: Hydrostatic _____ Pneumatic _____ Nominal Operating Pressure _____
N/A ☒ Other _____ Pressure _____ psi Test Temp. _____ °F

FORM NIS-2 (Back)

9. Remarks Replaced the seat ring mechanically

Applicable Manufacturer's Data Reports to be attached

Ref. JO:C33535-04File: 1-NRV-152

ISI Class: 1

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this
replacement conforms to the rules of the ASME Code, Section XI.
 repair or replacement

Type Code Symbol Stamp N/ACertificate of Authorization No. N/A Expiration Date N/A

Signed Frank Pisarsky Maint. Eng. Supervisor Date 7/3, 1997
 Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Michigan and employed by ARKWRIGHT MUTUAL INS. CO.* of NORWOOD, MASS. have inspected the components described in this Owner's Report during the period 10/30/95 to 7/16/97, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

MR. Matusz Commissions MICHIGAN, END N & I
 Inspector's Signature National Board, State, Province, Endorsements

Date July 16 1997

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner AMERICAN ELECTRIC POWER Date 11-04-95
Name _____
- P.O. Box 60 Fort Wayne, IN 46801 Sheet 1 of 2
Address _____
2. Plant D.C. COOK NUCLEAR POWER PLANT Unit _____ # 1
Name _____
- One Cook Place, Bridgman, MI 49106 C33535-04, 1-NRV-152, 3" vlv.
Address _____ Repair Org. P.O. No., Job No., etc _____
3. Work Performed by Maintenance Department Type Code Symbol Stamp N/A
Name _____ Authorization No. N/A
Same as #2 Expiration Date N/A
Address _____
4. Identification of System Pressurizer
5. (a) Applicable Const. Code ANSI B31.1 19 83Ed., N/A Add. N/A Code Case _____
(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1983 Ed.
Summer Addenda, 1983
6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manuf. Serial No.	Nat. Board No.	Other Identification	Year Built	Repaired Replaced or Replacement	ASME Code Stamped (Yes or No)
1-NRV-152							
Seat Ring ASTM A276 tp 316	Masoneilan			M&E# 30042230 ASP#16010	n/a	replace	NO

7. Description of Work (See Remarks)
8. Test Conducted: Hydrostatic _____ Pneumatic _____ Nominal Operating Pressure _____
N/A ☒ Other _____ Pressure _____ psi Test Temp. _____ °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks Installed a new wedge and machined the hardfacing material
Applicable Manufacturer's Data Reports to be attached
on the valve's wedge.

Ref. JO:C28174-2 File: 1-NMO-153 ISI Class: 1

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this
replacement/repair conforms to the rules of the ASME Code, Section XI.
 repair or replacement

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed Frank Pisarsky Maint. Eng. Supervisor Date 4/9, 1997
 Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Michigan and employed by ARKWRIGHT MUTUAL INS. CO.* of NORWOOD, MASS. have inspected the components described in this Owner's Report during the period 5/18/95 to 7/10/97, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Michael J. McCross Commissions Mich 0055, END NIT
 Inspector's Signature National Board, State, Province, Endorsements

Date April 10 19 97

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner AMERICAN ELECTRIC POWER Date 03-20-97
Name
P.O. Box 60 Fort Wayne, IN 46801 Sheet 1 of 2
Address
2. Plant D.C. COOK NUCLEAR POWER PLANT Unit # 1
Name
One Cook Place, Bridgman, MI 49106 C28174-2, 1-NMO-153, vlv
Address Repair Org. P.O. No., Job No., etc
3. Work Performed by Maintenance Department Type Code Symbol Stamp N/A
Name Authorization No. N/A
Same as #2 Expiration Date N/A
Address
4. Identification of System Pressurizer
5. (a) Applicable Const. Code ANSI B31.1 19 83Ed., N/A Add. N/A Code Case
(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1989
No Addenda

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manuf. Serial No.	Nat. Board No.	Other Identification	Year Built	Repaired Replaced or Replacement	ASME Code Stamped (Yes or No)
1-NMO=153							
Wedge SA-182 gr.FXM-19 w/Stellit#6				P.O.# 02072-041-5 ASP#18947	n/a	replace repair.	NO

7. Description of Work (See Remarks)
8. Test Conducted: Hydrostatic _____ Pneumatic _____ Nominal Operating Pressure _____
N/A ☒ Other _____ Pressure _____ psi Test Temp. _____ °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks Installed a new wedge and machined the hardfacing material
Applicable Manufacturer's Data Reports to be attached
on the valve's wedge.

Ref. JO:C28173-2 File: 1-NMO-152 ISI Class: 1

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this
replacement/repair conforms to the rules of the ASME Code, Section XI.
 repair or replacement

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed Frank Pisarsky Maint. Eng. Supervisor Date 4/9, 19 97
 Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Michigan and employed by ARKWRIGHT MUTUAL INS. CO.* of NORWOOD, MASS. have inspected the components described in this Owner's Report during the period 5/18/95 to 4/10/97, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

M. J. Winter Commissions Mich DOSS, and NBT
 Inspector's Signature National Board, State, Province, Endorsements

Date April 10, 19 97

1. Owner AMERICAN ELECTRIC POWER Date 03-20-97
 Name _____
P.O. Box 60 Fort Wayne, IN 46801 Sheet 1 of 2
 Address _____
2. Plant D.C. COOK NUCLEAR POWER PLANT Unit # 1
 Name _____
One Cook Place, Bridgman, MI 49106 C28173-2, 1-NMO-152, vlv
 Address _____ Repair Org. P.O. No., Job No., etc _____
3. Work Performed by Maintenance Department Type Code Symbol Stamp N/A
 Name _____ Authorization No. N/A
Same as #2 Expiration Date N/A
 Address _____
4. Identification of System Pressurizer
5. (a) Applicable Const. Code ANSI B31.1 19 83Ed. N/A Add. N/A Code Case _____
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1989
No Addenda

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manuf. Serial No.	Nat. Board No.	Other Identification	Year Built	Repaired Replaced or Replacement	ASME Code Stamped (Yes or No)
1-NMO=152							
Wedge SA-182 gr.FXM-19 w/Stellit#6				P.O.# 02072-041-5 ASP#18947	n/a	replace repair	NO

7. Description of Work (See Remarks)
8. Test Conducted: Hydrostatic _____ Pneumatic _____ Nominal Operating Pressure _____
 N/A ☒ Other _____ Pressure _____ psi Test Temp. _____ °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks Replaced the valve wedge and machined to fit
Applicable Manufacturer's Data Reports to be attached

Ref. JO:C28172-2 File: 1-NMO-151 ISI Class: 1

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this
replacement/repair conforms to the rules of the ASME Code, Section XI.
 repair or replacement

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed Frank Pisarsky, Maint. Eng. Supervisor Date 1/3, 1997
 Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Michigan and employed by ARKWRIGHT MUTUAL INS. CO.* of NORWOOD, MASS. have inspected the components described in this Owner's Report during the period 5-18-95 to 7-2-97, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Mr. Waterhouse Commissions Michigan, ENCL NIS-2
 Inspector's Signature National Board, State, Province, Endorsements

Date July 2 19 97

~~*FACTORY MUTUAL SYSTEM~~

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner AMERICAN ELECTRIC POWER Date 03-20-97
Name
P.O. Box 60 Fort Wayne, IN 46801 Sheet 1 of 2
Address
2. Plant D.C. COOK NUCLEAR POWER PLANT Unit # 1
Name
One Cook Place, Bridgman, MI 49106 C28172-2, 1-NMO-151, vlv
Address Repair Org. P.O. No., Job No., etc
3. Work Performed by Maintenance Department Type Code Symbol Stamp N/A
Name Authorization No. N/A
Same as #2 Expiration Date N/A
Address
4. Identification of System Pressurizer System
5. (a) Applicable Const. Code ANSI B31.1 19 83Ed., N/A Add. N/A Code Case
(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1989,
No Addenda
6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manuf. Serial No.	Nat. Board No.	Other Identification	Year Built	Repaired Replaced or Replacement	ASME Code Stamped (Yes or No)
1-NMO-151							
Wedge SA-182 grFXM-19 w/ hardfacing				P.O.# 020720415 ASP#18947	n/a	replace machine	NO

7. Description of Work (See Remarks)
8. Test Conducted: Hydrostatic _____ Pneumatic _____ Nominal Operating Pressure _____
N/A ✓ Other _____ Pressure _____ psi Test Temp. _____ °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks Replaced valve and adjacent piping by welding
Applicable Manufacturer's Data Reports to be attached

Ref. JO:C37089-8 File: 1-MS-135-2 ISI Class: 2

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this
replacement conforms to the rules of the ASME Code, Section XI.
 repair or replacement

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed Frank Pisarsky, Maint. Eng. Supervisor Date 7/8, 1997
 Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Michigan and employed by ARKWRIGHT MUTUAL INS. CO.* of NORWOOD, MASS. have inspected the components described in this Owner's Report during the period 3/7/97 to 7/15/97, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

MR. M. W. [Signature] Commissions Mich 0055, ENDNIT
 Inspector's Signature National Board, State, Province, Endorsements

Date July 15 1997

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner AMERICAN ELECTRIC POWER Date 03-14-97
 Name _____
P.O. Box 60 Fort Wayne, IN 46801 Sheet 1 of 2
 Address _____
2. Plant D.C. COOK NUCLEAR POWER PLANT Unit _____ # 1
 Name _____
One Cook Place, Bridgman, MI 49106 C37089-08, 1-MS-135-2, vlv
 Address _____ Repair Org. P.O. No., Job No., etc _____
3. Work Performed by Maintenance Department Type Code Symbol Stamp N/A
 Name _____ Authorization No. N/A
Same as #2 Expiration Date N/A
 Address _____
4. Identification of System _____ Main Steam
5. (a) Applicable Const. Code ANSI B31.1 19 83Ed., N/A Add. N-416-1 Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1989
No Addenda
6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manuf. Serial No.	Nat. Board No.	Other Identification	Year Built	Repaired Replaced or Replacement	ASME Code Stamped (Yes or No)
1-MS-135-2 SA-105	Vogt			M&E#: 30044248 ASP#15781	n/a	replace	NO
pipe, 2" SA-106 gr.B				M&E#: 30014981 ASP#18702	n/a	replace	NO

7. Description of Work _____ (See Remarks)
8. Test Conducted: Hydrostatic _____ Pneumatic _____ Nominal Operating Pressure _____
 VT/MT _____ Other VT-2 Pressure _____ psi Test Temp. _____ °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks Mechanically installed seat ring and plug
 Applicable Manufacturer's Data Reports to be attached

Ref. JO:C40682-02 File: 1-MRV-241 ISI Class: 2

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this
replacement conforms to the rules of the ASME Code, Section XI.
 repair or replacement

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed Frank Pisarsky for Maint. Eng. Supervisor Date 6/22, 1997
 Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Michigan and employed by ARKWRIGHT MUTUAL INS. CO.* of NORWOOD, MASS. have inspected the components described in this Owner's Report during the period 4/21/97 to 6/27/97, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

MR. [Signature] Commissions Michigan, EN & I
 Inspector's Signature National Board, State, Province, Endorsements

Date June 27 19 97

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner AMERICAN ELECTRIC POWER Date 04-22-97
Name _____
P.O. Box 60 Fort Wayne, IN 46801 Sheet 1 of 2
Address _____
2. Plant D.C. COOK NUCLEAR POWER PLANT Unit _____ # 1
Name _____
One Cook Place, Bridgman, MI 49106 C40682-02, 1-MRV-241, vlv
Address _____ Repair Org. P.O. No., Job No., etc. _____
3. Work Performed by Maintenance Department Type Code Symbol Stamp N/A
Name _____ Authorization No. N/A
Same as #2 Expiration Date N/A
Address _____
4. Identification of System _____ Main Steam
5. (a) Applicable Const. Code ANSI B31.1 19 83Ed., N/A Add. N/A Code Case _____
(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1989, _____
No Addenda

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manuf. Serial No.	Nat. Board No.	Other Identification	Year Built	Repaired Replaced or Replacement	ASME Code Stamped (Yes or No)
1-MRV-241							
Plug:ASTM A582 416s/s				MSE#: 30-040005 ASP#15863	n/a	replace	NO
Seat Ring ASTM A582 416 s/s				M&E#: 30-040120 ASP#16247	n/a	replace	NO

7. Description of Work _____ (See Remarks)
8. Test Conducted: Hydrostatic _____ Pneumatic _____ Nominal Operating Pressure _____
N/A ☒ Other _____ Pressure _____ psi Test Temp. _____ °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks Mechanically installed seat ring and plug
Applicable Manufacturer's Data Reports to be attached

Ref. JO:R36164-02 File: 1-MRV-231 ISI Class: 2

. CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this
replacement conforms to the rules of the ASME Code, Section XI.
 repair or replacement

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed Frank Pisarsky, Maint. Eng. Supervisor Date 4/24, 1997
 Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Michigan and employed by ARKWRIGHT MUTUAL INS. CO.* of NORWOOD, MASS. have inspected the components described in this Owner's Report during the period 2-9-96 to 5-27-97, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

W. E. Waterhouse Commissions Michoors, ENDNGT
 Inspector's Signature National Board, State, Province, Endorsements

Date May 27 19 97

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner AMERICAN ELECTRIC POWER Date 03-06-97
Name _____
P.O. Box 60 Fort Wayne, IN 46801 Sheet 1 of 2
Address _____
2. Plant D.C. COOK NUCLEAR POWER PLANT Unit _____ # 1
Name _____
One Cook Place, Bridgman, MI 49106 R36164-02, 1-MRV-231, vlv
Address _____ Repair Org. P.O. No., Job No., etc _____
3. Work Performed by Maintenance Department Type Code Symbol Stamp N/A
Name _____ Authorization No. N/A
Same as #2 Expiration Date N/A
Address _____
4. Identification of System Main Steam
5. (a) Applicable Const. Code ANSI B31.1 19 83Ed., N/A Add. N/A Code Case _____
(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1989,
No Addenda

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manuf. Serial No.	Nat. Board No.	Other Identification	Year Built	Repaired Replaced or Replacement	ASME Code Stamped (Yes or No)
1-MRV-231							
Plug:ASTM A582 416s/s				M&E#: 30-040005 ASP#15863	n/a	replace	NO
Seat Ring ASTM A582 416 s/s				M&E#: 30-040120 ASP#16247	n/a	replace	NO

7. Description of Work (See Remarks)
8. Test Conducted: Hydrostatic _____ Pneumatic _____ Nominal Operating Pressure _____
N/A ☒ Other _____ Pressure _____ psi Test Temp. _____ °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks Mechanically installed seat ring and plug
 Applicable Manufacturer's Data Reports to be attached

Ref. JO:R36163-02 File: 1-MRV-221 ISI Class: 2

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this
replacement conforms to the rules of the ASME Code, Section XI.
 repair or replacement

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed Frank Pisarsky, Maint. Eng. Supervisor Date 4/24, 1997
 Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Michigan and employed by ARKWRIGHT MUTUAL INS. CO.* of NORWOOD, MASS. have inspected the components described in this Owner's Report during the period 2/9/94 to 5/21/97, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Michael J. Doss Commissions MICH, MASS, END, NBT
 Inspector's Signature National Board, State, Province, Endorsements

Date May 27 1997

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner AMERICAN ELECTRIC POWER Date 03-06-97
Name _____
P.O. Box 60 Fort Wayne, IN 46801
Address _____
2. Plant D.C. COOK NUCLEAR POWER PLANT Unit _____ # 1
Name _____
One Cook Place, Bridgman, MI 49106 R36163-02, 1-MRV-221, vlv
Address _____ Repair Org. P.O. No., Job No., etc _____
3. Work Performed by Maintenance Department Type Code Symbol Stamp N/A
Name _____ Authorization No. N/A
Same as #2 Expiration Date N/A
Address _____
4. Identification of System Main Steam
5. (a) Applicable Const. Code ANSI B31.1 19 83Ed., N/A Add. N/A Code Case _____
(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1989, No Addenda

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manuf. Serial No.	Nat. Board No.	Other Identification	Year Built	Repaired Replaced or Replacement	ASME Code Stamped (Yes or No)
1-MRV-221							
Plug: ASTM A582 416s/s				M&E#: 30-040005 ASP#17496	n/a	replace	NO
Seat Ring ASTM A582 416 s/s				M&E#: 30-040120 ASP#16247	n/a	replace	NO

7. Description of Work (See Remarks)
8. Test Conducted: Hydrostatic _____ Pneumatic _____ Nominal Operating Pressure _____
N/A ☒ Other _____ Pressure _____ psi Test Temp. _____ °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks Mechanically installed seat ring and plug
 Applicable Manufacturer's Data Reports to be attached

Ref. JO:R36162-02 File: 1-MRV-211 ISI Class: 2

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this
replacement conforms to the rules of the ASME Code, Section XI.
 repair or replacement

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed Frank Pisarsky, Maint. Eng. Supervisor Date 4/24, 1997
 Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Michigan and employed by ARKWRIGHT MUTUAL INS. CO.* of NORWOOD, MASS. have inspected the components described in this Owner's Report during the period 2-29-96 to 5-22-97, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Frank Pisarsky
 Inspector's Signature

Commissions Mich. Code, ENR NFI
 National Board, State, Province, Endorsements

Date May 28 19 97

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner AMERICAN ELECTRIC POWER Date 03-06-97
Name _____
P.O. Box 60 Fort Wayne, IN 46801 Sheet 1 of 2
Address _____
2. Plant D.C. COOK NUCLEAR POWER PLANT Unit _____ # 1
Name _____
One Cook Place, Bridgman, MI 49106 R36162-02, 1-MRV-211, vlv
Address _____ Repair Org. P.O. No., Job No., etc _____
3. Work Performed by Maintenance Department Type Code Symbol Stamp N/A
Name _____ Authorization No. N/A
Same as #2 Expiration Date N/A
Address _____
4. Identification of System Main Steam
5. (a) Applicable Const. Code ANSI B31.1 1983Ed., N/A Add. N/A Code Case _____
(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1989, No Addenda
6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manuf. Serial No.	Nat. Board No.	Other Identification	Year Built	Repaired Replaced or Replacement	ASME Code Stamped (Yes or No)
1-MRV-211							
Plug:ASTM A582 416s/s				MSE#: 30-040005 ASP#15863	n/a	replace	NO
Seat Ring ASTM A582 416 s/s				M&E#: 30-040120 ASP#16247	n/a	replace	NO

7. Description of Work (See Remarks)
8. Test Conducted: Hydrostatic _____ Pneumatic _____ Nominal Operating Pressure _____
N/A ☒ Other _____ Pressure _____ psi Test Temp. _____ °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks Replaced the seal protector ring
Applicable Manufacturer's Data Reports to be attached

Ref. JO:C33810-7 File: 1- IRV-311 ISI Class: 2

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this
replacement conforms to the rules of the ASME Code, Section XI.
 repair or replacement

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed Frank Pisarsky, Maint. Eng. Supervisor Date 6/22, 1997
 Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Michigan and employed by ARKWRIGHT MUTUAL INS. CO.* of NORWOOD, MASS. have inspected the components described in this Owner's Report during the period 3-26-96 to 6-27-97, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

M. F. Whitman Commissions Mich 0055, ENR N.E.I.
 Inspector's Signature National Board, State, Province, Endorsements

Date June 27 19 97

FORM NIS-2 (Back)

9. Remarks

Applicable Manufacturer's Data Reports to be attached

Ref. JO: R52239-06 & 16 DESIGN CHANGE: N/A

REPAIR/REPLACEMENT TRAVELLER: RR96-1621 ISI CLASS: 1

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this
REPLACEMENT conforms to the rules of the ASME Code, Section XI.
 repair or replacement

Type Code Symbol Stamp N/ACertificate of Authorization No. N/A Expiration Date N/A

Signed Wayne Cretsinger, Senior Planner
 Owner or Owner's Designee, Title

Date April 25, 1997

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of MICHIGAN and employed by ARKWRIGHT MUTUAL INS. CO. of WALTHAM, MASS. have inspected the components described in this Owner's Report during the period 12/10/96 to 5/5/97, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Inspector's Signature

Commissions

National Board, State, Province, Endorsements

Date May 5, 1997

1. Owner AMERICAN ELECTRIC POWER COMPANY Date 04/04/97
 Name
P.O. Box 60 Fort Wayne, IN 46801 Sheet 1 of 2
 Address
2. Plant D.C. COOK NUCLEAR POWER PLANT Unit # 1
 Name
One Cook Place, Bridgman, MI 49106 R22763-06 R/R 1040
 Address Repair Org. P.O. No., Job No., etc
3. Work Performed by Installation Services Type Code Symbol Stamp N/A
 Name Authorization No. N/A
Same as #2 Expiration Date N/A
 Address
4. Identification of System REFUELING WATER STORAGE TANK ASME CODE CLASS 3
5. (a) Applicable Const. Code ASME SEC VIII 19 86 Ed., N/A Add. N/A Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1989
6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manuf. Serial No.	Nat. Board No.	Other Identification	Year Built	Repaired Replaced or Replacement	ASME Code Stamped (Yes or No)
1-TK-33 3/4" H.H. NUT	NOVA	ASP# 14876	N/A	SA194 GR.8F	N/A	REPLACE- MENT	NO
							NO

7. Description of Work REPLACED 3/4" NUT THAT WAS LOST

8. Test Conducted: Hydrostatic ☐ Pneumatic ☐ Nominal Operating Pressure ☐

N/A ☒ Other ☐ Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks

Applicable Manufacturer's Data Reports to be attached

Ref. JO: R22763-06 DESIGN CHANGE: N/A ISI CLASS: 2

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this REPLACEMENT conforms to the rules of the ASME Code, Section XI.
repair or replacement

Type Code Symbol Stamp N/ACertificate of Authorization No. N/A Expiration Date N/A

Signed Wayne Cretsinger, Senior Planner
Owner or Owner's Designee, Title

Date APRIL 10, 1997

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Michigan and employed by ARKWRIGHT MUTUAL INS. CO. of WALTHAM, MASS. have inspected the components described in this Owner's Report during the period 4/3/97 to 4/14/97, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

N.R. Mutzgruehl
Inspector's Signature

Commissions Mich 0055, END NFI

National Board, State, Province, Endorsements

Date April 14 1997

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner INDIANA MICHIGAN POWER COMPANY Date 07-03-96
Name
P.O. Box 60 Fort Wayne, IN 46801 Sheet 1 of 2
Address
2. Plant D.C. COOK NUCLEAR POWER PLANT Unit # 1
Name
One Cook Place, Bridgman, MI 49106 C13407-01, 1-WCR-933, 3" vlv
Address Repair Org. P.O. No., Job No., etc
3. Work Performed by Maintenance Department Type Code Symbol Stamp N/A
Name Authorization No. N/A
Same as #2 Expiration Date N/A
Address
4. Identification of System Non Essential Service Water system
5. (a) Applicable Const. Code ANSI B31.1 19 83Ed., N/A Add. N/A Code Case
(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1989,
No Addenda

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manuf. Serial No.	Nat. Board No.	Other Identification	Year Built	Repaired Replaced or Replacement	ASME Code Stamped (Yes or No)
1-WCR-933							
Nuts 5/8" SA-194 gr 8F (4)				M&E#: 30-212090 ASP#16473 30-040589 ASP#18382	n/a	replace	NO

7. Description of Work (See Remarks)
8. Test Conducted: Hydrostatic _____ Pneumatic _____ Nominal Operating Pressure _____
N/A ☒ Other _____ Pressure _____ psi Test Temp. _____ °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.



FORM NIS-2 (Back)

9. Remarks Replace existing fasteners with new material
 Applicable Manufacturer's Data Reports to be attached

Ref. JO:C13407-1 File:1-WCR-933

ISI Class: 2

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this
replacement conforms to the rules of the ASME Code, Section XI.
 repair or replacement

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed Frank Pisarsky, Maint. Eng. Supervisor Date 3/17, 19 97
 Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Michigan and employed by ARKWRIGHT MUTUAL INS. CO. of NORWOOD, MASS. have inspected the components described in this Owner's Report during the period 12-11-92 to 4-2-97, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

M. J. Matusz Commissions MICH 0055, ENDORSEMENTS NFI
 Inspector's Signature National Board, State, Province, Endorsements

Date 4-2 19 97

~~FACTORY MUTUAL SYSTEM~~

DONALD G. COOK NUCLEAR PLANT (COOK NUCLEAR PLANT)

UNIT NUMBERS 1 AND 2

DOCKET NOS. 50-135 AND 50-316

LICENSE NOS. DPR-58 AND DPR-74

UPDATED QUALITY ASSURANCE PROGRAM DESCRIPTION

FOR THE

COOK NUCLEAR PLANT

JULY 1997

Concurred by:

Paul G. Boardman
Performance Assurance Director

Date: 7-30-97

Approved by :

EE Jitpatrik
Vice President - Indiana Michigan Power Company

Date: 8/1/97



STATEMENT OF POLICY
FOR THE DONALD C. COOK NUCLEAR PLANT
QUALITY ASSURANCE PROGRAM

POLICY

American Electric Power [REDACTED] recognizes the fundamental importance of controlling the design, modification, and operation of Indiana Michigan Power Company's Donald C. Cook Nuclear Plant by implementing a planned and documented quality assurance program, including quality control, that complies with applicable regulations, codes, and standards.)

The quality assurance program has been established to control activities affecting safety-related functions of structures, systems, and components in Cook Nuclear Plant. The quality assurance program supports the goal of maintaining the safety and reliability of Cook Nuclear Plant at the highest level through a systematic program designed to assure that activities affecting safety-related functions are conducted in compliance with applicable regulations, codes, standards, and established corporate policies and practices.

As chairman of the board, president, and chief executive officer of American Electric Power Company [REDACTED], I maintain the ultimate responsibility for the quality assurance program associated with Cook Nuclear Plant. I have delegated responsibilities for implementation of, and compliance with, the quality assurance program, as outlined in this statement.

IMPLEMENTATION

The [REDACTED] performance assurance director, under the direction of the [REDACTED] executive vice president nuclear generation, has been assigned the overall responsibility for specifying the quality assurance program requirements for Cook Nuclear Plant and verifying their implementation. The [REDACTED] performance assurance director has authority to stop work on any activity affecting safety-related items that does not meet applicable administrative, technical, and/or regulatory requirements. The

Statement of Policy for the
Donald C. Cook Nuclear Plant

performance assurance director does not have the authority to stop unit operations, but shall notify appropriate plant and/or corporate management of conditions not meeting the aforementioned criteria and recommend that unit operations be terminated.

The executive vice president nuclear generation, under my direction, has been delegated responsibility for effectively implementing the quality assurance program. All other AEP divisions and departments having a supporting role for Cook Nuclear Plant are functionally responsible to the executive vice president nuclear generation.

The site vice president , under the direction of the executive vice president nuclear generation, is delegated the responsibility for implementing the quality assurance program at Cook Nuclear Plant.

The performance assurance director is responsible for establishing a quality control program at Cook Nuclear Plant.

The performance assurance director is responsible for providing technical direction to the site vice president for matters relating to the quality assurance program at Cook Nuclear Plant. The performance assurance director is responsible for maintaining a quality assurance group at Cook Nuclear Plant to perform required reviews, audits, and surveillances, and to provide technical liaison services to the site vice president .

The requirements for implementation of the quality assurance program are described in the nuclear generation group policies and procedures.

Each nuclear generation group involved in activities affecting safety-related functions of structures, systems, and components in Cook Nuclear Plant has the responsibility to implement the applicable policies and requirements of the quality assurance program. This responsibility includes being familiar with, and complying with, the applicable quality assurance program requirements.

Statement of Policy for the
Donald C. Cook Nuclear Plant

COMPLIANCE

The performance assurance director shall monitor compliance with the established quality assurance program. Audit programs shall be established to ensure that nuclear generation group activities comply with established program requirements, identify deficiencies or noncompliances, and obtain effective and timely corrective actions.

Any employee engaged in activities affecting safety-related functions of structures, systems, and components in Cook Nuclear Plant who believes the quality assurance program is not being complied with, or that a deficiency in quality exists, should notify his/her supervisor, the performance assurance director, and/or the site vice president. If the notification does not, in the employee's opinion, receive prompt or appropriate attention, the employee should contact successively higher levels of management. An employee reporting such conditions shall not be discriminated against by companies of the American Electric Power System, nor shall any supplier under contract with any of the companies of the American Electric Power System discriminate against any employee of the supplier for reporting such conditions. Discrimination includes discharge or other actions relative to compensation, terms, conditions, or privileges of employment.



E. Linn Draper, Jr.
Chairman of the Board, President,
and Chief Executive Officer

1.7.1 ORGANIZATION

1.7.1.1 SCOPE

Indiana Michigan Power Company's nuclear generation group (I&M) is responsible for establishing and implementing the Quality Assurance (QA) program for the operational phase of the Donald C. Cook Nuclear Plant. Although authority for development and execution of various portions of the program may be delegated to others, such as contractors, agents or consultants, I&M retains overall responsibility. I&M shall evaluate work delegated to such organizations. Evaluations shall be based on the status of safety importance of the activity being performed and shall be initiated early enough to assure effective quality assurance during the performance of the delegated activity.

This section of the Quality Assurance Program Description (QAPD) identifies the organizational responsibilities for activities affecting the quality of safety-related nuclear power plant structures, systems, and components, and describes the authority and duties assigned to them. It addresses responsibilities for both attaining quality objectives and for the functions of establishing the QA program, and verifying that activities affecting the quality of safety-related items are performed effectively in accordance with QA program requirements.

1.7.1.2 IMPLEMENTATION

1.7.1.2.1 Source of Authority

The chairman of the board, president, and chief executive officer of American Electric Power Company, Inc. (AEP), through its wholly owned subsidiary I&M is responsible for safe operation of the Cook Nuclear Plant. Authority and responsibility for effectively implementing the QA program

for plant modifications, operations and maintenance are delegated to the I&M vice president responsible for nuclear generation. The I&M vice president also serves as the American Electric Power Service Corporation (AEPSC) executive vice president nuclear generation.

In the operation of a nuclear power plant, the licensee is required to establish clear and direct lines of responsibility, authority and accountability. This requirement is applicable to the organization providing support to the plant, as well as to the plant staff.

The responsibility for the support of Cook Nuclear Plant rests with I&M which includes the onsite and offsite AEP organizations that administer, operate, maintain, and modify the plant. The I&M vice president has primary responsibility for Cook Nuclear Plant. All I&M nuclear generation group organizations are functionally responsible to the I&M vice president (reference Figure 1.7-1).

In order to facilitate a more thorough understanding of the support functions, some of the responsibilities, authorities, and accountabilities within the organization are as follows:

- 1) The responsibilities of the I&M vice president shall be dedicated to the area of Cook Nuclear Plant operations and support.
- 2) The I&M vice president shall be responsible for, and has the authority to direct, all Cook Nuclear Plant operational and support matters and shall make, or concur, in all final decisions regarding significant nuclear safety matters.

- 3) I&M managers shall be familiar with activities within their scope of responsibility that affect plant safety and reliability. They shall be cognizant of, and sensitive to, internal and external factors that might affect the operations of Cook Nuclear Plant.
- 4) I&M managers have a commitment to seek and identify problem areas and take corrective action to eliminate unsafe conditions, or to improve trends that will upgrade plant safety and reliability.
- 5) The I&M vice president [redacted] shall ensure that Cook Nuclear Plant personnel are not requested to perform inappropriate work or tasks by corporate personnel, and shall control assignments and requests that have the potential for diverting the attention of the site vice president [redacted] from the primary responsibility for safe and reliable plant operation.
- 6) I&M managers shall be familiar with the policy statements from higher management concerning nuclear safety and operational priorities. They shall be responsible for ensuring that activities under their direction are performed in accordance with these policies.

1.7.1.2.2 Responsibility for Attaining Quality Objectives in I&M Nuclear Generation

The AEP chairman of the board, president, and chief executive officer has assigned the overall responsibility for specifying QA program requirements and verifying their implementation to the performance assurance director.

The I&M vice president under the direction of the AEP chairman of the board, president, and chief executive officer, is responsible for effectively implementing the QA program.

The performance assurance director, under the direction of the vice president is responsible for establishing the Cook Nuclear Plant quality control program.

Each I&M manager involved in activities affecting safety-related functions of structures, systems, and components in Cook Nuclear Plant, has the responsibility to implement the applicable policies and requirements of the quality assurance program. This responsibility includes being familiar with and complying with, the applicable quality assurance program requirements.

I&M has an independent off-site Nuclear Safety and Design Review Committee (NSDRG) which has been established pursuant to the requirements of the Technical Specifications, Appendix C to this QAPD for the Cook Nuclear Plant. The function of the NSDRG is to oversee the engineering, design, operation, and maintenance of the Cook Nuclear Plant by performing audits and independent reviews of activities which are specified in the facility Technical Specifications.

The Cook Nuclear Plant on-site review group is the Plant Nuclear Safety Review Committee (PNSRC). This committee has also been established pursuant to the requirements of the Cook Nuclear Plant Technical Specifications, Appendix C to this QAPD. The function of

the PNSRC is to review plant operations on a continuing basis and advise the site vice president [REDACTED] on matters related to nuclear safety.

1.7.1.2.3 Corporate Organization

American Electric Power Company

AEP, the parent holding company, wholly owns the common stock of all AEP System subsidiary (operating) companies. [REDACTED] The chairman of the board, president, and chief executive officer of AEP is the chief executive officer of [REDACTED] and all operating companies. The responsibility for the functional management of the major operating companies is vested in the president of each operating company reporting to the [REDACTED] chairman of the board, president, and chief executive officer.

Operating Companies

The operating facilities of the AEP System are owned and operated by the respective operating companies. The responsibility for executing the engineering, design, construction, specialized technical training, and certain operations' supervision is vested in AEPSC, while all, or part, of the administrative functional responsibility is assigned to the operating companies. In the case of Cook Nuclear Plant, [REDACTED] provides [REDACTED] public affairs, accounting, and industrial safety direction.

The Cook Nuclear Plant is owned and operated by I&M which is part of the AEP System.

1.7.1.2.4 Quality Assurance Responsibility of I&M

- 1) I&M provides the technical direction for the Cook Nuclear Plant, and as such makes the final decisions pertinent to safety-related changes in plant design. Further, I&M reviews Nuclear Regulatory Commission (NRC) letters, bulletins, notices, etc., for impact on plant design, and the need for design changes or modifications.
- 2) I&M furnishes quality assurance, engineering, design, construction, licensing, NRC correspondence, fuel management and radiological support activities.
- 3) I&M provides additional service in matters such as supplier qualification, procurement of original equipment and replacement parts, and the process of dedicating commercial grade items or services to safety-related applications.
- 4) The performance assurance organization provides technical direction in quality assurance matters to the nuclear organization and oversees the adequacy, effectiveness and implementation of the QA Program through review and audit activities.
- 5) Cognizant engineer (e.g., system engineer, equipment engineer, lead engineer, responsible engineer, procurement engineer etc.) is that individual who provides the engineering/design expertise for a particular area of responsibility. This

responsibility includes the implementation of the quality assurance and quality control measures for systems, equipment, structures, or functional areas included in that individual's responsibility. The various titles used for the identification of an individual's responsibility and assignment shall be understood to mean the same as cognizant engineer in the respective areas of responsibility.

Quality Assurance Responsibility - Cook Nuclear Plant

The Cook Nuclear Plant staff operates the Cook Nuclear Plant in accordance with licensing requirements, including the Technical Specifications and such other commitments as established by the operating licenses. The categories of procedures identified in section 1.7.5.2.2 describe the means by which compliance is achieved and responsibilities are assigned. Figure 1.7-1 indicates the organizations pertaining to the operation and support of the Cook Nuclear Plant.

1.7.1.2.5 Organization

The chairman of the board, president, and chief executive officer is ultimately responsible for the QA program associated with the Cook Nuclear Plant. This responsibility is administered through the I&M vice president responsible for nuclear generation.

Nuclear Generation

Nuclear generation is comprised of regulatory affairs, nuclear engineering, performance assurance, business performance, site operations, and human resources.

Performance Assurance

The performance assurance director, reporting to the I&M vice president is responsible for the performance assurance organization. Performance Assurance consists of the following organizations (Figure 1.7-2):

- Performance engineering and analysis
- Plant and supplier performance

Performance Assurance is organizationally independent and is responsible to perform the following:

- Specify QA program requirements.
- Identify quality problems.
- Initiate, recommend, or provide solutions through designated channels.
- Verify implementation of solutions, as appropriate.
- Prepare, issue and maintain QA program documents, as required.
- Verify the implementation of the QA program through scheduled audits and surveillances.
- Verify the implementation of computer software quality assurance through reviews, surveillances and audits.

- Audit engineering, design, procurement, construction and operational documents for incorporation of, and compliance with, applicable quality assurance requirements to the extent specified by the [REDACTED] management-approved QA program.
- Organize and conduct the QA auditor orientation, training, certification and qualification of [REDACTED] audit personnel.
- Provide direction for the collection, storage, maintenance, and retention of quality assurance records.
- Maintain, on data base, a list of suppliers of nuclear (N) items and services, plus other selected categories of suppliers.
- Identify noncompliances of the established QA program to the responsible organizations for corrective actions, and report significant occurrences that jeopardize quality to senior [REDACTED] management.
- Follow up on selected corrective actions, taken in response to adverse conditions, to confirm effectiveness.
- Review the disposition of selected conditions adverse to quality to assure that action taken will preclude recurrence.
- Conduct in-process QA audits or surveillances at supplier's facilities, as required.
- Assist and advise other [REDACTED] groups in matters related to the QA program.
- Conduct audits as directed by the NSDRC.
- Maintain cognizance of industry and governmental quality assurance requirements such that the QA program is compatible with requirements, as necessary.
- Recommend for revision to, or improvements in, the established QA Program to senior [REDACTED] management.

- Audit dedication plans for commercial grade items and services.
- Issue "Stop Work" orders when significant conditions adverse to safety-related items are identified to prevent unsafe conditions from occurring and/or continuing.
- Provide management with periodic reports concerning the status, adequacy and implementation of the QA program.
- Prepare and conduct special verification and/or surveillance programs on in-house activities, as required or requested.
- Routinely attend, and participate in, daily plant work schedule and status meetings.
- Provide adequate QA coverage relative to procedural and inspection controls, acceptance criteria, and QA staffing and qualification of personnel to carry out QA assignments.
- Determine the acceptability of vendors to supply products and services for safety-related applications.
- Provide working-level coordination with the Institute of Nuclear Power Operations (INPO) in the areas of INPO training, seminars, and workshops. This effort includes providing the nuclear generation organization access to INPO resources, such as NUCLEAR NETWORK, and effectively considering the use of INPO recommendations contained in operating experience reports to improve Cook Nuclear Plant performance.
- Develop and implement an effective Quality Control (QC) Program. This encompasses, but is not limited to, the planning and directing of quality control activities to assure that industry codes, NRC regulations, and company instructions and policies

regarding quality control for Cook Nuclear Plant are implemented, qualified personnel perform the work, and that these activities are properly documented.

- Direct the activities of contractor QC/nondestructive examination (NDE) personnel assigned to the plant performance assurance department and provide oversight of work performed.
- Qualification and certification of I&M personnel performing inspections or tests of major modifications and non-routine maintenance to the requirements of Regulatory Guide 1.58 and ANSI N45.2.6, except as noted in Appendix B hereto, item 9.
- Proper certification of contractor inspection, test and examination personnel in accordance with Regulatory Guide 1.58, ANSI N45.2.6, ASME B&PV Code and/or SNT-TC-1A, as applicable.
- Selection of a qualification and certification administrator (NDE administrator) to certify personnel in accordance with ANSI N45.2.6 and SNT-TC-1A, as applicable.

Amplification of Specific Responsibilities

- Qualification of the performance assurance director
The performance assurance director shall possess the following position requirements:
 - Bachelor's degree in engineering, scientific, or related discipline.
 - Ten (10) years experience in one of, or a combination of, the following areas:
engineering, design, construction, operations, maintenance of fossil or nuclear power generation facilities' or utility facilities' QA, of which at least four (4) years must be

experience in nuclear quality assurance related activities.

- Knowledge of QA regulations, policies, practices and standards.
- The same, or higher,, organization reporting level as the highest line manager directly responsible for performing activities affecting the quality of safety-related items, such as engineering, procurement, construction and operation, and is sufficiently independent from cost and schedule.
- Effective communication channels with other senior management positions.
- Responsibility for approval of QA Manual(s).
- Performance of no other duties or responsibilities unrelated to QA that would prevent full attention to QA matters.

Stop Work Orders

The ~~performance assurance organization~~ is responsible for ensuring that activities affecting the quality of safety-related items are performed in a manner that meets applicable administrative, technical, and regulatory requirements. In order to carry out this responsibility, the AEP chairman of the board, president, and chief executive officer has given the ~~performance assurance director~~ the authority to stop work on any activity affecting the quality of safety-related items that does not meet the aforementioned requirements. Stop work authority has been further delegated by the ~~performance assurance director~~ to the ~~manager of performance engineering and analysis~~, and the ~~manager of plant and supplier performance~~.

The performance assurance director and the subordinate managers do not have the authority to stop unit operations, but will notify appropriate management of conditions which do not meet the aforementioned criteria, and recommend that unit operations be terminated.

- QA Auditor, Qualification and Certification Program
- ICM has established and maintains a QA auditor training and certification program for all QA auditors.
- Condition Identification, Reporting and Escalation
- ICM has established mechanisms for the identification, reporting and escalation of conditions affecting the quality of safety-related items to a level of management whereby satisfactory resolutions can be obtained.

Regulatory Affairs

The regulatory affairs director, reporting to the ICM vice president is responsible for the following:

- Formulate policies and practices relative to licensing, fuel management, and radiological support.
- Maintain liaison with the performance assurance director.
- Implement the requirements of the QA program.

- Maintain knowledge of the latest [REDACTED] licensing, and regulatory requirements, codes, standards, and federal regulations applicable to the operation of Cook Nuclear Plant.
- Accomplish the procurement, economic, technical, licensing and quality assurance activities dealing with the reactor core and its related fuel assemblies and components.
- Prepare bid specifications, evaluate bids, and negotiate and administer contracts for the procurement of all nuclear fuel and related components and services.
- Maintain a special nuclear material accountability system.
- Provide analyses to support nuclear steam supply system operation, including [REDACTED] fuel economics, fuel mechanical behavior, [REDACTED] furnish plant Technical Specification changes and other licensing work, and participate in NRC and NSDRG meetings as required by these analyses.
- Perform reactor core operation follow-up activities and other reactor core technical support activities as requested, and arrange for support from the fuel fabricator, when needed.
- Contract for, and provide technical support for, disposal of both high level and low level radioactive waste.
- [REDACTED]
- [REDACTED]
- Obtain and maintain the NRC Operating Licenses and Technical Specifications for the Cook Nuclear Plant.
- Act as the communication link between the NRC and I&M.

- Perform and coordinate efforts involved in gathering information, performing calculations and generic studies; preparing criteria, reports, and responses; reviewing items affecting safety; and interpreting regulations.
- [REDACTED] The preparation of changes to, and appropriate interpretation of, the plant Technical Specification submittals of license amendments; and the analysis of plant compliance with regulatory requirements.
- Primary corporate contact for most oral and written communication with the NRC.

[REDACTED]

[REDACTED]

- Review, evaluate, and respond to NRC requests for information and NRC notifications of regulatory changes resulting in plant modifications or new facilities. Such responses are generated in accordance with appropriate administrative procedures.

[REDACTED]

[REDACTED]

- Review, on a conceptual basis, plant reports, to the extent that they are related to the ultimate safe operation of the plant, for compliance with safety regulations, plant Technical Specifications, the Updated FSAR design basis, and with any other requirements under the Operating License, to determine if there are any unreviewed safety questions as defined in 10CFR50.59.
- Perform reviews of Condition Reports and 10CFR21 reviews in accordance with corporate requirements.
- Operate the Action Item Tracking (AIT) system for [REDACTED] internal commitment tracking.

[REDACTED]

- [REDACTED]
- Contribute to the annual FSAR updates through reviews of Licensee Event Reports, and the Annual Operating Report.

- [REDACTED]
- [REDACTED]
- Serve as technical advisors on plant audits.
 - Remain cognizant of current decommissioning practices and developments.

[REDACTED] Nuclear Engineering [REDACTED]


The [REDACTED] chief nuclear engineer, [REDACTED] reporting to the I&M vice president [REDACTED] is responsible for certain engineering, design, procurement, and construction functions. [REDACTED] Nuclear engineering [REDACTED] is comprised of plant engineering, [REDACTED] design engineering, [REDACTED] and production engineering, [REDACTED]

Certain organizations within the AEP power generation group and energy delivery provide occasional technical assistance for the Cook Nuclear Plant. The administrative and quality assurance controls for this assistance are controlled through documented interface agreements. [REDACTED]

[REDACTED] Nuclear engineering [REDACTED] is responsible for the following:

- Provide planning, engineering and design of the electrical facilities inside Cook Nuclear Plant up to the high voltage (HV) bushings of the main generator transformers and mechanical facilities inside Cook Nuclear Plant including:
 - * determination of general layout and design;
 - * selection of equipment;

- * preparation of one-line and flow diagrams; and,
- * coordination of inside and outside plant facilities.
- Provide engineering and design of all controls for operation and protection of nuclear steam supply, steam generator, turbine generator, auxiliary equipment and general plant protection, including checking and approving elementary, one-line, and flow drawings.
- Ensure that all purchased equipment conforms to accepted standards and fulfills the desired function.
- Closely follow manufacturer's engineering and design processes to assure provision of adequate and reliable equipment upon which depend the safety, reliability, and performance of the unit and plant.
- Prepare, review and/or approve design changes, sketches, drawings, calculations, and design verifications, as required.
- Perform safety reviews of design changes pursuant to 10CFR50.59.
- Prepare and/or approve dedication plans, specifications and procurement documents.
- Perform drawing review of equipment, as appropriate.
- Develop, review and/or approve procedures or correspondence as appropriate.
- Obtain, review and perform engineering and design evaluations, including environmental equipment qualification (EQ).
- Establish and maintain a central file for equipment environmental qualification documentation.
- Coordinate operations [REDACTED] that support the Cook Nuclear Plant Facility Data Base (FDB).

- Perform calculations for proper application of equipment.
- Perform and evaluate investigations, analyses and reports for facilities pertaining to the engineering design, operation and maintenance of the Cook Nuclear Plant.
- Assist field personnel in installation, start-up, and subsequent locating of problems in equipment, and in determining proper operation of equipment, during normal or after emergency operations.
- Maintain a constant awareness for improvements and more reliable design of equipment and facilities, maintenance and operating methods or procedures.
- Maintain a constant awareness of activities to ensure compliance with all applicable policies and procedures, initiating, when required, training or retraining programs.
- Participate, as assigned, on the NSDRC and NSDRC subcommittees, and participate in matters covered in the committee's charter.
- Provide responses to NRC correspondence, as required.
- Participate in the evaluation and remedy of any situation requiring activation of the Emergency Response Organization.
- Provide support personnel for the Emergency Response Organization.
- Provide technical support in areas of operation and maintenance, including: the Inservice Inspection (ISI) program; the QA program; the fire protection QA program; the  ALARA program covering radiation protection; and, the corporate and plant industrial safety program.

- Provide technical direction and assistance in the layout and arrangement of equipment piping, systems, controls, etc., for the development of drawings.
- [REDACTED]
- Develop System Descriptions..
- Provide analytical support in engineering and design disciplines (e.g., heat transfer, thermodynamics, fluid dynamics).
- Provide engineering and design evaluations for CRs, LERs, INPO SOERs, and NRC Bulletins.
- Participate, as assigned, on the [REDACTED] Condition Assessment Group (CAG).
- Make recommendations and assist in the formulation of policies and practices relating to the design and engineering of office and service buildings, miscellaneous structures and material handling equipment, and provide the general supervision of the engineering of such facilities, structures and equipment.
- Initiate and/or review, approve and control laboratory and field investigations and feasibility studies.
- [REDACTED]
- Arrange for outside engineering, design and consulting assistance, as required.
- Perform shop and field surveillance on equipment being manufactured, fabricated, or installed.
- Provide field services to the Cook Nuclear Plant, including the assigning of personnel, as are required, during construction, normal or forced outages, or as requested.

- Assist in the planning and execution of maintenance work on equipment, facilities, buildings and other structures.
- Supervise maintenance and repairs of all masonry and concrete work at Cook Nuclear Plant, including supplying qualified inspection personnel.
- Direct testing of materials used in concrete and testing of soils to be used in work at the Cook Nuclear Plant.
- Review and recommend concrete mix formulations for all new construction.
- Implement the corrective action program, with regard to activities affecting the quality of safety-related items and services, that controls and documents items, services or activities which do not conform to requirements.
- Assist in the preparation of applications for federal, state and local permits relative to installations being made which require such permits.
- Conduct periodic management reviews of the activities of the department to ensure compliance with the objectives of the QA Program, and external technical surveillance, as necessary, of consultants, outside organizations and vendors over which the department is cognizant.
- Establish and maintain a file for QA records.
- Develop, review and approve designs and drawings for mechanical, electrical and structural systems, equipment and facilities of the Cook Nuclear Plant.
- Perform required calculations and analyses, including pipe stress, pipe support design, cable sizing, conduit and cable tray support and structural steel and concrete.

- Assist field personnel in the resolution of problems stemming from the installation of design changes, or from as-found plant conditions, including assigning personnel to the plant.
- Formulate, administer, and implement policies and practices relating to the engineering, and design of the Cook Nuclear Plant.
- Conduct functions so as to be in conformance with the operating licenses of the Cook Nuclear Plant.
- Investigate evaluate and correct problems.
- Coordinate special projects and studies, as required.
- Coordinate the development and maintenance of the Vendor Drawing Control (VDC) programs which include coordinating the programs with interfacing divisions/departments.
- Control the issuance and distribution of drawings for the Cook Nuclear Plant, including monitoring of the Aperture Card Microfilm Program.
- Supervise and control the work of consultants, architect/engineers and outside engineering and design agencies supplying services to ~~ICM~~ in their discipline and process notification of defects in accordance with company requirements. Also perform detailed reviews of engineering and design work submitted by outside agencies.
- Review and update applicable sections of Cook Nuclear Plant Updated FSAR as assigned.
- Participate, as members and as assigned, on committees and ad hoc task forces that review nuclear activities.

- Coordinate Cook Nuclear Plant activities associated with the initiation, review, approval, engineering, design, production, examination, inspection, test, turnover, and close out of design changes.
- [REDACTED]
- [REDACTED]
- Administer and implement job orders issued by the Cook Nuclear Plant organization for major modifications, replacement and maintenance work with outside contractors.
- Administer and monitor contractor's industrial safety programs and performance.
- [REDACTED]
- Manage construction labor relations with the international building and construction trades unions.
- [REDACTED]
- Plan, organize and control major construction projects, as assigned by the I&M vice president [REDACTED].
- Maintain cognizance on matters pertaining to the Cook Nuclear Plant [REDACTED] emergency response organization.
- Prepare labor estimates.
- Provide constructability guidance when requested in support of engineering and design changes.
- Formulate Policies and practices relative to nuclear safety
- Maintain knowledge of the latest safety requirements, codes, standards, and federal regulations applicable to the operation of Cook Nuclear Plant
- Provide analysis to support reactor physics, core thermal hydraulic and LOCA and non-LOCA transient safety analysis

- and other analysis activities as requested, and participate in NRC and NSDRG meetings related to these analyses.
- Coordinate the development of neutronics and thermal hydraulic safety codes and conduct safety analysis.
 - Coordinate computer code development, and provide the interface control for AEPSC information systems and I&M nuclear generation.
 - Review, coordinate, and resolve all matters pertaining to nuclear safety for Cook Nuclear Plant. This includes, but is not limited to: the preparation of safety evaluations, or reviews, for a designated subject.
 - Provide support in key areas of expertise, such as nuclear engineering, probabilistic analysis, thermohydraulic analysis, chemical engineering, mechanical engineering, electrical engineering, and technical writing.
 - Interface with vendors and other outside organizations on matters connected with the nuclear steam supply system and other areas affecting the safe design and operation of nuclear plants.
 - Participate, as appropriate, in the review of nuclear plant operating experiences, and relate those experiences to the design and safe operation of Cook Nuclear Plant.

- Develop, specify, and/or review conceptual nuclear safety criteria for Cook Nuclear Plant in accordance with established regulations. This includes all information contained in the FSAR, as well as specialized information such as environmental qualification and seismic criteria.
- Review and evaluate performance requirements for systems, equipment and materials for compliance with specified safety criteria.
- Coordinate Equipment Performance and Information Exchange (EPIX) with INPO.
- Recommend facility engineering modification and initiate and approve plant improvement requisitions.
- Plan and direct engineering and technical studies, equipment performance, and instrument and control maintenance for Cook Nuclear Plant.
- Direct programs related to on-site fuel management and reactor core physics testing, and ensure satisfactory completion.
- Coordinate the maintenance of design drawings.

Site Operations

The [redacted] site vice president [redacted] reports [redacted] to the [redacted] vice president [redacted] and is responsible for the Cook Nuclear Plant activities (Figure 1.7-~~1~~).

The site operations organization is responsible for the following:

- Ensure the safety of all facility employees and the general public relative to general plant safety, as well as radiological safety, by maintaining strict compliance with plant Technical Specifications, procedures and instructions.
- Recommend facility engineering modification and initiate and approve plant improvement requisitions.
- Ensure that work practices in all site operations organizations are consistent with regulatory standards, safety, approved procedures, and plant Technical Specifications.
- Provide membership, as required, on the PNSRC.
- Maintain close working relationships with the NRC, as well as local, state, and federal government regulatory officials regarding conditions which could affect, or are affected, by Cook Nuclear Plant activities.
- Set up plant load schedules and arrange for equipment outages.
- Develop and efficiently implement all site centralized training activities.

- Administer the centralized facility training complex, simulator, and programs ensuring that program development is consistent with the systematic approach to training, maintain INPO accreditations, regulatory and corporate requirements.
- Ensure that human resource activities include employee support programs (i.e., fitness for duty) consistent with INPO/NUMARC guidelines, company policies, and regulatory requirements and standards.
- Administer the NRC approved physical Security Program in compliance with regulatory standards, Modified Amended Security Plan (MASP), and company policy.
- Supervise, plan, and direct the activities related to the maintenance and installation of all Cook Nuclear Plant equipment, structures, grounds, and yards.
- Prepare and maintain records and reports pertinent to equipment maintenance and regulatory agency requirements.
- [REDACTED]
- Enforce and coordinate Cook Nuclear Plant regulations, procedures, policies, and objectives to assure safety, efficiency, and continuity in the operation of the Cook Nuclear Plant within the limits of the operating license and the Technical Specifications and formulation of related policies and procedures.
- Plan, schedule, and direct activities relating to the operation of the Cook Nuclear Plant and associated switchyards; cooperate in planning and scheduling of work and procedures for refueling and maintenance of the Cook Nuclear Plant; and direct and coordinate fuel loading operations.

- Review reports and records, direct general inspection of operating conditions of plant equipment, and investigate any abnormal conditions, making recommendations for repairs. Establish and administer equipment clearance procedures consistent with company, plant, and radiation protection standards; authorize and arrange for equipment outages to meet normal or emergency conditions. Provide the shift operating crews with appropriate procedures and instructions to assist them in operating the Cook Nuclear Plant safely and efficiently.
- Approve operator training programs administered by the Cook nuclear plant training department designed to provide operating personnel with the knowledge and skill required for safe operation of the facility, and for obtaining and holding NRC operator licenses. Coordinate training programs in plant safety and emergency procedures for Cook Nuclear Plant operating department personnel to ensure that each shift group will function properly in the event of injury of personnel, fire, nuclear incident, or civil disorder.
- Advance planning and overall conduct of scheduled and forced outages, including the scheduling and coordination of all plant activities associated with refueling, preventive maintenance, corrective maintenance, equipment overhaul, Technical Specification surveillance, and design change installations.
- Prepare reports of reportable events which are mandated by the NRC and the Technical Specifications.
- Prepare statistical reports utilized in NRC Appraisal Meetings and Enforcement Conference. .

- Coordinate the efforts of outside agencies, such as American Nuclear Insurers (ANI), INPO, and third-party inspector programs.
- Maintain knowledge of developments and changes in NRC requirements, industry standards and codes, regulatory compliance activities, and quality control disciplines and techniques.
- Stop plant operation, as appropriate, in the event that conditions are found which are in violation of the Technical Specifications or adverse to quality.
- Maintain and renew accreditation of training programs.
- Qualification of I&M personnel performing inspection of normal operating activities to ANSI N18.1.
- Perform peer inspections of work completed by I&M personnel by independent persons qualified to ANSI N18.7.
- Conduct of the Inservice Inspection (ISI) Program.
- Plan and direct on-site computer systems, Shift Technical Advisors, and emergency planning. These activities support daily on-site operations in a safe, reliable, and efficient manner in accordance with all corporate policies, applicable laws, regulations, licenses, and Technical Specification requirements.
- Implement station performance testing and monitor programs to ensure optimum plant efficiency.
-
- Establish testing and preventive maintenance programs related to station instrumentation, electrical systems, and computers.
- Recommend alternatives to Cook Nuclear Plant operation, technical or emergency procedures, and

design of equipment to improve safety of operations and overall plant efficiency.

- Implement the [REDACTED] Emergency Plan as it pertains to the Cook Nuclear Plant site.
- Provide technical and engineering services in the fields of chemistry, radiation protection, ALARA, and environmental in support of the safe operation of the plant and the health and safety of the employees and the public.
- Plan and schedule the activities of the radiation protection department of the Cook Nuclear Plant in support of operations and maintenance.
- Establish chemistry, radiochemistry, and health physics criteria which ensure maximum equipment life, and the protection of the health and safety of the workers and the public.
- Establish sampling and analysis programs which ensure the chemistry, radiochemistry, and health physics criteria are within the established criteria.
- Establish and direct investigations, responses, and corrective actions when outside the established criteria.
- Administer and direct the Cook Nuclear Plant's radioactive waste programs, including volume reduction, packaging and shipping.
- [REDACTED]
- Maintain the Cook Nuclear Plant Facility Data Base.
- [REDACTED]
- Procurement, receiving, quality control receipt inspection, storage, handling, issue, stock level maintenance, and overall control of stores items.

- Provide material service and support in accordance with policies and procedures required by AEPSO purchasing and materials management, QA, and the NRC, which are administered and enforced in a total effort to ensure safety and plant reliability.
- Provide nuclear General Employee Training (GET) for nuclear generation personnel.

Business Performance

The business performance director, reporting to the I&M vice president, is responsible for the following:

- Prepare and administer equipment, labor and service contracts.
- Administer contracts and schedule outside contractors' work forces
- Administration of the QA records program
- Scope, bid, recommend awards and administer construction labor and service contracts.
- Process incoming vendor information.

Purchasing and Materials Management (not charted)

The AEP executive vice president administration and chief accounting officer, reporting to the AEP chairman of the board, president, and chief executive officer, is responsible for purchasing and materials management through the vice president - procurement & supply chain services

Procurement & supply chain services is responsible for the following:

- Procurement of safety-related items from only qualified and approved suppliers.
- Provide supervision to Cook Nuclear Plant purchasing organization
- Provide ordering and stocking descriptions (Material & Equipment database) for safety-related items and include these descriptions in the Cook Nuclear Plant inventory catalog, including necessary communications with suppliers, cognizant engineers, the Cook Nuclear Plant stores supervisor and other appropriate personnel.
- Establish computerized inventory status reports, on line inventory and purchase order inquiry capabilities and other procedures to order, track and control materials.
- Coordinate procurement activities with I&M.
- Prepare and issue requests for quotations, contracts, service orders, blanket orders, and purchase orders for safety-related items.
- Implement corrective action as described in the I&M procedures for Cook Nuclear Plant.

- Establish a system of document keeping and transmittal.
- Establish a system of document control for controlled procedures, instructions, and purchasing documents for safety-related items.
- The maintenance and control of selected procurement document standard phrases as identified by the ~~performance~~ assurance ~~director~~ or designee.
- Conduct training sessions involving purchasing personnel and others on an annual basis, or more frequently, as required, and ascertain that training sessions include complete responsibilities associated with the purchase of safety-related items and services.

1.7.2 QUALITY ASSURANCE PROGRAM

1.7.2.1 SCOPE

Policies that define and establish the Cook Nuclear Plant QA Program are summarized in the individual sections of this document. The program is implemented through procedures and instructions responsive to provisions of the QAPD, and will be carried out for the life of the Cook Nuclear Plant.

Quality assurance controls apply to activities affecting the quality of safety-related structures, systems and components to an extent based on the importance of those structures, systems, components, etc., (items) to safety. Such activities are performed under controlled conditions, including the use of appropriate equipment; environmental conditions, assignment of qualified personnel, and assurance that all applicable prerequisites have been met.

Safety-related items are defined as items:

- Which are associated with the safe shutdown (hot) of the reactor; or isolation of the reactor; or maintenance of the integrity of the reactor coolant system pressure boundary.
- OR
- Whose failure might cause or increase the severity of a design basis accident as described in the Updated FSAR; or lead to a release of radioactivity in excess of 10CFR100 guidelines.

In general, safety-related items are those which are classified Seismic Class I, or Electrical Class 1E; or associated with the Engineered Safety Features Actuation System (ESFAS); or associated with the Reactor Protection System (RPS). Note: Some nonsafety-related items have been designed to Seismic Class I and/or Electrical Class 1E requirements. For example: post accident monitoring instrumentation is not safety-related but is qualified Seismic Class 1 and Electrical Class 1E to meet the requirements of Reg. Guide 1.97.

A special QA Program has been implemented for Fire Protection items (Section 1.7.19 herein).

The QA Program also includes provision for Radwaste QA in accordance with the requirements of 10CFR71, Subpart H.

QA Program status, scope, adequacy, and compliance with 10CFR50, Appendix B, are regularly reviewed by [REDACTED] management through reports, meetings, and review of audit results.

The implementation of the QA program may be accomplished by [REDACTED] or delegated in whole or in part to other AEP System companies or outside parties. However, [REDACTED] retains full responsibility for all activities affecting safety-related items. The performance of the delegated organization is evaluated by audit or surveillances on a frequency commensurate with their scope and importance of assigned work.

1.7.2.2 IMPLEMENTATION

1.7.2.2.1

The chairman of the board, president, and chief executive officer of [REDACTED] has stated in a signed, formal "Statement of Policy", that it is the corporate policy to comply with the provisions of applicable codes, standards and regulations pertaining to quality assurance for nuclear power plants as required by the Cook Nuclear Plant operating licenses.

The statement makes this QAPD and the associated implementing procedures and instructions mandatory, and requires compliance by all responsible organizations and

individuals. The statement also identifies the management positions within the companies vested with responsibility and authority for implementing the program and assuring its effectiveness:

1.7.2.2.2

The QA program at ~~ICM~~ consists of controls exercised by organizations responsible for attaining quality objectives, and by organizations responsible for assurance functions.

The QA Program effectiveness is continually assessed through management review of various reports, NSDRC review of the QA audit program, and shall also be periodically reviewed by independent outside parties as deemed necessary by management.

The QA program described in this QAPD is intended to apply for the life of the Cook Nuclear Plant.

The QA program applies to activities affecting the quality of safety-related structures, components, and related consumables during plant operation, maintenance, testing, and all design changes. Safety-related structures, systems and components are identified in the Facility Data Base and other documents which are developed and maintained for the plant.

As deemed necessary by ~~management~~ management, applicable portions of the QA program controls will be applied to nonsafety-related activities associated with the implementation of the QA program to ensure that commitments are met (e.g., off-site records storage, training services, etc.).

1.7.2.2.3

This QAPD, organized to present the QA Program for the Cook Nuclear Plant in the order of the 18 criteria of 10CFR50, Appendix B, states ~~the~~ policy for each of the criteria and describes how the controls pertinent to each are carried out. Any changes made to this QAPD that do not reduce the commitments previously accepted by the NRC must be submitted to the NRC at least annually. Any changes made to this QAPD that do reduce the commitments previously accepted by the NRC must be submitted to the NRC and receive NRC approval prior to implementation. The submittal of the changes described above shall be made in accordance with the requirements of 10CFR50.54.

The program described in this QAPD will not be intentionally changed in any way that would prevent it from meeting the criteria of 10CFR50, Appendix B and other applicable operating license requirements.

1.7.2.2.4

Documents used for implementing the provisions of this QAPD include the following:

Plant Manager Instructions (PMIs) establish the policy at the plant for compliance with specified criteria, and assign responsibility to the various departments, as required, for implementation. Plant Manager Procedures (PMPs), Department Head Procedures (DHPs), and in some cases Department Head Instructions (DHIs), have been

prepared to describe the detailed activities required to support safe and effective plant operation as per the PMIs.

The PMIs are reviewed by performance assurance for concurrence that they will satisfactorily implement regulatory requirements and commitments. PMIs and PMPs are reviewed by the PNSRC prior to approval by the site vice president [REDACTED].

DHPs and DHIs are reviewed within the departments prior to approval by the department head of origination. DHPs and DHIs that might involve an unreviewed safety question as defined in 10CFR50.59 are reviewed by PNSRC prior to approval by the department head of origination.

AEP Nuclear Organization Policy & Procedure Manual and [REDACTED] General Procedures (GPs) are utilized to define policies and requirements for quality assurance, and to implement certain QA program requirements. [REDACTED] Division/department and/or section procedures are also used to implement QA program requirements.

When contractors perform work on-site under their own quality assurance programs, the programs are audited for compliance and consistency with the applicable requirements of the Cook Nuclear Plant's QA Program and the contract, and are approved by performance assurance prior to the start of work. Implementation of on-site contractor's QA programs, will be audited to assure that the contractor's programs are effective.

1.7.2.2.5

Provisions of the QA program for the Cook Nuclear Plant apply to activities affecting the quality of safety-related items. Appendix A to this QAPD lists the Regulatory/Safety Guides and ANSI Standards that identify ~~TCM~~ commitment. Appendix B describes necessary exceptions and clarifications to the requirements of those documents. The scope of the program, and the extent to which its controls are applied, are established as follows:

- a) ~~TCM~~ uses the criteria specified in the Cook Nuclear Plant Updated FSAR for identifying structures, systems and components to which the QA program applies.
- b) This identification process results in the Facility Data Base for the Cook Nuclear Plant. This Facility Data Base is controlled by authorized personnel. Facility Data Base items are determined by engineering analysis of the function(s) of plant items in relation to safe operation and shutdown.
- c) The extent to which controls specified in the QA program are applied to Facility Data Base items is determined for each item considering its relative importance to safety. Such determinations are based on data in such documents as the Cook Nuclear Plant Technical Specifications and the Updated FSAR.

Appendix C to this QAPD identifies administrative controls, such as onsite and offsite review committee activities, which either supplement or complement the quality assurance program described herein

1.7.2.2.6

Activities affecting safety-related items are accomplished under controlled conditions. Preparations for such activities include consideration of the following:

- a) Assigned personnel are qualified.
- b) Work has been planned to applicable engineering and/or Technical Specifications.
- c) Specified equipment and/or tools are available.
- d) Items are in an acceptable status.
- e) Items on which work is to be performed are in the proper condition for the task.
- f) Proper approved instructions/procedures for the work are available for use.
- g) Items and facilities that could be damaged by the work have been protected, as required.
- h) Provisions have been made for special controls, processes, tests and verification methods.

1.7.2.2.7

Responsibility and authority for planning and implementing indoctrination and training of I&M personnel are specifically designated, as follows:

- a) The training and indoctrination program provides for on-going training and periodic familiarization with the QA program for the Cook Nuclear Plant.

- b) Personnel who perform inspection and examination functions are qualified in accordance with requirements of Regulatory Guide 1.8, ANSI N18.1, Regulatory Guide 1.58, ANSI N45.2.6, the ASME B&PV Code, or SNT-TC-1A, as applicable, and with exceptions as noted in Appendix B hereto.
- c) ~~Performance assurance~~ auditors are qualified in accordance with Regulatory Guide 1.146 and ANSI N45.2.23.
- d) Personnel assigned duties such as special cleaning processes, welding, etc., are qualified in accordance with applicable codes, standards, regulatory guides and/or plant procedures.
- e) The training, qualification and certification program includes, as applicable, provisions for retraining, reexamination and recertification to ensure that proficiency is maintained.
- f) Training, qualification, and certification records including documentation of objectives, waivers/exceptions, attendees and dates of attendance, are maintained at least as long as the personnel involved are performing activities to which the training, qualification and certification is relevant.
- g) Personnel responsible for performing activities that affect safety-related items are instructed as to the purpose, scope and implementation of the applicable manuals, instructions and procedures.

Management/supervisory personnel receive functional training to the level necessary to plan, coordinate and administer the day-to-day verification activities of the QA Program for which they are responsible.

Training of I&M personnel is performed employing the following techniques, as applicable: 1) on the job and formal training administered by the department or section the individual works for; 2) formal training conducted by qualified instructors from the training department or other entities (internal and external to the AEP System); and 3) formal, INPO accredited training conducted by the training department. Records of training sessions for such training are maintained. Where personnel qualifications or certifications are required, these certifications are performed on a scheduled basis (consistent with the appropriate code or standard).

Cook Nuclear Plant employees receive introductory training in quality assurance usually within the first two weeks of employment. In addition, I&M personnel receive training prior to being allowed unescorted access to the plant. This training includes management's policy for implementation of the QA program through plant manager and department head instructions and procedures. These instructions also include a description of the QA program, the use of instructions and procedures, personnel requirements for procedure compliance and the systems and components controlled by the QA program.

1.7.3 DESIGN CONTROL

1.7.3.1 SCOPE

Design changes are accomplished in accordance with approved design. Activities to develop such designs are controlled. Depending on the scope of the design change, these activities include design and field engineering; the performance of physics, seismic, stress, thermal, hydraulic and radiation evaluations; update of the FSAR; review of accident analyses; the development and control of associated computer programs; studies of material compatibility; accessibility for inservice inspection and maintenance; determination of quality standards; and requirement for equipment qualification. The controls apply to preparation and review of design documents, including the correct translation of applicable regulatory requirements and design bases into design, procurement and procedural documents.

1.7.3.2 IMPLEMENTATION

1.7.3.2.1

Design changes are controlled by procedures and instructions and are reviewed as required by 10CFR50.59 and the Technical Specifications.

Safety related and non-safety related design changes are implemented via Design Change Packages (DCPS).

1.7.3.2.2

Design changes are reviewed to determine their impact on nuclear safety and to determine if the proposed changes

involve an unreviewed safety question as defined by 10CFR50.59. If a design change were to involve an unreviewed safety question, it would not be approved for implementation until the required NRC approval was received.

Design Change Packages (DCPs) are reviewed and approved prior to implementation, [REDACTED] by the DCP team members and cognizant [REDACTED] managers. The PNSRC also reviews those DCPs, for which safety evaluations are deemed necessary, pursuant to 10CFR50.59 and Technical Specification 6.5.1.6.

1.7.3.2.3

When DCPs involve design interfaces between internal or external design organizations, or across technical disciplines, these interfaces are controlled. Procedures are used for the review, approval, release, distribution and revision of documents involving design interfaces to ensure that structures, systems and components are compatible geometrically and functionally with processes and the environment. Lines of communication are established for controlling the flow of needed design information across design interfaces, including changes to the information as work progresses. Decisions and problem resolutions involving design interfaces are made by the [REDACTED] organization having responsibility for engineering direction of the design effort.

1.7.3.2.4

Checks are performed and documented to verify the dimensional accuracy and completeness of design drawings and specifications.

1.7.3.2.5

Design change document packages are audited by performance assurance to assure that the documents have been prepared, verified, reviewed and approved in accordance with company procedures.

1.7.3.2.6

The extent of, and methods for, design verification are documented. The extent of design verification performed is a function of the importance of the item to safety, design complexity, degree of standardization, the state-of-the-art, and similarity with previously proven designs. Methods for design verification include evaluation of the applicability of standardized or previously proven designs, alternate calculations, qualification testing and design reviews. These methods may be used singly or in combination, depending on the needs for the design under consideration.

When design verification is done by evaluating standardized or previously proven designs, the applicability of such designs is confirmed. Any differences from the proven design are documented and evaluated for the intended application.

Qualification testing of prototypes, components, or features is used when the ability of an item to perform an essential safety function cannot otherwise be adequately substantiated. This testing is performed before plant equipment installation, where possible, but always before reliance upon the item to perform a safety-related function. Qualification testing is performed under conditions that simulate the most adverse design conditions, considering all relevant operating modes. Test requirements, procedures and results are documented. Results are evaluated to assure that test requirements have been satisfied. Design changes shown to be necessary through testing are made, and any necessary retesting or other verification is performed. Test configurations are clearly documented.

Design reviews are performed by multi-organizational or interdisciplinary groups, or by single individuals. Criteria are established to determine when a formal group review is required, and when review by an individual is sufficient.

1.7.3.2.7

Persons representing applicable technical disciplines are assigned to perform design verifications. These persons are qualified by appropriate education or experience, but are not directly responsible for the design. The designer's immediate supervisor may perform the verification, provided that:

- 1) The supervisor is the only technically qualified individual.

or

2) The supervisor has not specified a singular design approach, ruled out design considerations, nor established the design inputs.

and

3) The need is documented and approved by the supervisor's management.

Regularly scheduled QA audits verify conformance to previous items 1 through 3.

Design verification of safety-related design changes shall be completed prior to declaring a design change, or portions thereof, operational.

1.7.3.2.8

Implementation of design changes is coordinated on site by ~~nuclear engineering~~. Material to perform the design change must meet the specifications established for the original system, or as specified by the DCP. For those design changes where testing after completion is required, the testing documentation is reviewed by the organization performing the test and, when specified, by the DCP. Further, completed design changes are audited/surveilled by ~~performance assurance~~ following installation and testing.

1.7.3.2.9

Changes to design documents, including field changes, are reviewed, approved and controlled in a manner commensurate with that used for the original design. Such changes are

evaluated for impact. Information on approved changes is transmitted to all affected organizations.

1.7.3.2.10

Error and deficiencies in, and deviations from, approved design documents are identified and dispositioned in accordance with established design control and/or corrective action procedures.

1.7.3.2.11

Established design control procedures provide for:

- 1) controlled submission of design changes,
- 2) engineering evaluation,
- 3) review for impact on nuclear safety,
- 4) audit by ~~performance assurance~~
- 5) design modification,
- 6) ~~managerial~~ managerial review, and
- 7) approval and record keeping for the implemented design change.

1.7.4 PROCUREMENT DOCUMENT CONTROL

1.7.4.1 SCOPE

Procurement documents define the characteristics of item(s) to be procured, identify applicable regulatory and industry codes/standards requirements, and specify supplier QA Program requirements to the extent necessary to assure adequate quality.

1.7.4.2 IMPLEMENTATION

1.7.4.2.1

Procurement control is established by instructions and procedures. These documents require that procurement documents be sufficiently detailed to ensure that purchased safety-related items and services are: 1) purchased to specification and code requirements equivalent to those of the original equipment or service (except when the Code of Federal Regulations requires upgrading), 2) properly documented to show compliance with the applicable specifications, codes and standards; and 3) purchased from vendors or contractors who have been evaluated and deemed qualified, or by the commercial grade dedication process.

Procedures establish the review of procurement documents to determine that: appropriate technical and quality requirements are correctly stated, inspectable and controllable; there are adequate acceptance criteria; and procurement documents have been prepared, reviewed and approved in accordance with established requirements.

The manager of the originating group, with support of the cognizant [REDACTED] engineering group, is responsible for assuring that applicable requirements are set forth in procurement documents.

The I&M may use cognizant engineers in any procurement activity.

1.7.4.2.2

The Facility Data Base, in conjunction with other sources, is used for equipment safety classification and procurement grade. ~~Engineering~~ specifications are used to determine requirements, codes or standards that items must fulfill, and define the documentation that must accompany the item to the plant.

Procurement documents for safety-related items and services are reviewed to ensure that: correct classification is made; the requirements are properly stated; and that measures have been, or will be, implemented to assure the requirements are met and adequately provided for.

Procurement documents for new safety-related items are initiated by the cognizant engineering group which establishes initial requirements.

Replacement/spares are purchased to requirements equivalent to the original unless upgrading is required by federal regulations, or deemed necessary by the cognizant engineering group.

1.7.4.2.3

The contents of procurement documents vary according to the item(s) being purchased and its function(s) in the Cook Nuclear Plant. Provisions of this QAPD are considered for application to service contractors, also. As applicable, procurement documents include:

- a) Scope of work to be performed.

- b) Technical requirements, with applicable drawings, specifications, codes and standards identified by title, document number, revision and date, with any required procedures, such as special process instructions identified in such a way as to indicate source and need. Imposition of guides/standards on I&M suppliers and subtier suppliers will be on a case-by-case basis depending upon the item or service to be supplied and upon the degree that I&M relies on suppliers to invoke guides/standards. I&M recognizes that certain suppliers have acceptable 10CFR50, Appendix B QA programs, even though, the suppliers are not committed to Regulatory Guides or industry standards (e.g. ANSI N45.2.6.). In those cases, in which suppliers are not committed to the same guides/standards as I&M, I&M will assure that (1) the supplier's QA program provides adequate QA controls, regardless of the lack of specific commitment, or (2) controls will be invoked directly by I&M to assure adequate quality of items/services received by suppliers.
- c) Regulatory, administrative and reporting requirements.
- d) Quality requirements appropriate to the complexity and scope of the work, including necessary tests and/or inspections.
- e) A requirement for a documented QA Program, subject to QA review and written concurrence prior to the start of work.
- f) A requirement for the supplier to invoke applicable quality requirements on subtier suppliers.

- g) Provisions for access to supplier, and subtier suppliers', facilities and records for inspections, surveillances and audits.
- h) Identification of documentation to be provided by the supplier, the schedule of submittals and documents requiring I&M approval.

1.7.4.2.4

Performance assurance performs audits of procurement documents to assure that QA program requirements have been met. These audits are conducted in accordance with performance assurance procedures.

1.7.4.2.5

Changes to procurement documents are controlled in a manner commensurate with that used for the original documents.

1.7.5 INSTRUCTIONS, PROCEDURES, AND DRAWINGS

1.7.5.1 SCOPE

Activities affecting the quality of safety-related structures, systems and components are accomplished using instructions, procedures and drawings appropriate to the circumstances, including acceptance criteria for determining if an activity has been satisfactorily completed.

1.7.5.2 IMPLEMENTATION

1.7.5.2.1

Instructions and procedures incorporate: 1) a description of the activity to be accomplished, and 2) appropriate quantitative (such as tolerances and operating limits) and qualitative (such as workmanship and standards) acceptance criteria sufficient to determine that the activity has been satisfactorily accomplished. Hold points for inspection are established when required.

Instructions and procedures pertaining to the specification of, and/or implementation of, the QA Program receive multiple reviews for technical adequacy and inclusion of appropriate quality requirements. Top tier instructions and procedures that define the quality assurance program requirements are reviewed and/or approved by performance assurance. Lower tier documents are reviewed and approved, as a minimum, by management/supervisory personnel trained to the level necessary to plan, coordinate and administer those day-to-day verification activities of the QA Program for which they are responsible.

Special procedures may be issued for activities which have short-term applicability.

1.7.5.2.2

I&M activities are outlined by procedures which provide the controls for the implementation of these activities. I&M has the following categories of QA program implementation procedures:

- 1) General Procedures (GPs), Plant Manager's Instructions (PMI's) and American Electric Power Nuclear Organization Procedures (AEPNOs) which are applicable to part or all organizations involved with Cook Nuclear Plant.
- 2) Organization procedures which apply to the specific division, department or section involved.

1.7.5.2.3

The procedures controlling I&M activities are classified into the following series:

- 1000 Personnel Selection, PNSRC Procedures
- 2000 Administration - Document Control, Security, Training, Records, Emergency Plan, Fire Protection, Clearance Permits, Chemical Control, Internal Cleanliness, Spill Response, Standing orders, Corrective Maintenance.
- 3000 Procurement, Receiving, Shipping and Storage
- 4000 Operations, Fuel Handling, Surveillance Testing, Test Controls
- 5000 Maintenance, Repair, Modification, Special Processes, EQ and ISI Control of Contractors
- 6000 Technical - Chemistry/Radiological Controls, Radiation Protection, Performance/Engineering Testing, and Instrument and Control Maintenance and Calibration, Measuring and Test Equipment
- 7000 Quality Assurance, Quality Control Program and Condition/Problem Reporting

Instructions and procedures identify the regulatory requirements and commitments which pertain to the subject that it will control and establish responsibilities for implementation. Instructions and procedures may either provide the guidance necessary for the development of supplemental instructions and/or procedures to implement their requirements, or provide comprehensive guidance based on the subject matter.

1.7.5.2.4

Cook Nuclear Plant drawings are produced, controlled and distributed under the control of T&M. T&M design drawings are produced by, or under the control of, [REDACTED] nuclear engineering [REDACTED] under a set of procedures which direct their development and review. These procedures specify requirements for inclusion of quantitative and qualitative acceptance criteria. Specific drawings are reviewed and approved by the cognizant engineering organization.

T&M has stationed an on-site design staff to provide for the revision of certain types of design drawings to reflect as-built conditions.

1.7.5.2.5

Complex [REDACTED] procedures that are specific to Cook Nuclear Plant site are designated as "In Hand" procedures. Examples of "In Hand" procedures are those developed for extensive or complex jobs where reliance on memory cannot be trusted. Further, those procedures which describe a sequence which cannot be altered, or require the documentation of data during the course of the procedure,

are considered. "In Hand" procedures are designated as such by double asterisks (**) which precede the procedure number on the cover sheet, all pages and attachments of a procedure and the corresponding index.

1.7.6 DOCUMENT CONTROL

1.7.6.1 SCOPE

Documents controlling activities within the scope defined in 1.7.2 herein are issued and changed according to established procedures. Documents such as instructions, procedures and drawings, including changes thereto, are reviewed for adequacy, approved for release by authorized personnel, and are distributed and used at the location where a prescribed activity is performed.

Changes to controlled documents are reviewed and approved by the same organizations that performed the original review and approval, or by other qualified, responsible organizations specifically designated in accordance with the procedures governing these documents. Obsolete or superseded documents are controlled to prevent inadvertent use.

1.7.6.2 IMPLEMENTATION

1.7.6.2.1

Controls are established for approval, issue and change of documents in the following categories:

- a) Design documents (e.g., calculations, specifications, analyses)
- b) Drawings and related documents
- c) Procurement documents

- d) Instructions and procedures
- e) Updated Final Safety Analysis Report (UFSAR)
- f) Plant Technical Specifications
- g) Safeguards documents

1.7.6.2.2

The review, approval, issuance and change of documents are controlled by:

- a) Establishment of criteria to ensure that adequate technical and quality requirements are incorporated.
- b) Identification of the organization responsible for review, approval, issue and maintenance.
- c) Review of changes to documents by the organization that performed the initial review and approval, or by the organization designated in accordance with the procedure governing the review and approval of specific types of documents.

1.7.6.2.3

Documents are issued and controlled so that:

- a) The documents are available prior to commencing work.
- b) Obsolete documents are replaced by current documents in a timely manner.

1.7.6.2.4

Master lists, or equivalent controls, are used to identify the current revision of instructions, procedures, specifications and drawings. These control documents are updated and distributed to designated personnel who are responsible for maintaining current copies of the applicable documents. The distribution of controlled documents is performed under procedures requiring receipt acknowledgement and in accordance with established distribution lists.

1.7.6.2.5

In the event a drawing is developed on-site to reflect an as-built configuration, the marked-up drawing is maintained in the Master Plant File and all holders of the drawing are issued appropriate notification to inform them the revision they hold is not current, cannot be used and, if required, reference must be made to the Master Plant File drawing.

1.7.6.2.6

Documents prepared for use in training are appropriately marked to indicate that they cannot be used to operate or maintain the facility or to conduct activities affecting the quality of safety-related items. At the Cook Nuclear Plant, unless a document is identified as 'controlled' or 'working copy' only, it is automatically assumed that the document is for information use only.

1.7.7 CONTROL OF PURCHASED ITEMS AND SERVICES

1.7.7.1 SCOPE

Activities that implement approved procurement requests for items and services are controlled to assure conformance with procurement document requirements. Controls include a system of supplier evaluation and selection audits, acceptance of items and documentation upon delivery, and periodic assessment of supplier performance. Objective evidence of quality that demonstrates conformance with specified procurement document requirements is available to the Cook Nuclear Plant site prior to use of equipment, material, or services.

1.7.7.2 IMPLEMENTATION

1.7.7.2.1

I&M qualifies suppliers (including distributors to the extent they perform quality related activities) by performing a documented evaluation of their capability to provide items or services specified by procurement documents. Items and services designated as safety-related are purchased from suppliers whose QA programs have been accepted in accordance with I&M requirements, or from commercial grade suppliers through the I&M dedication program. Suppliers of other items/services are subject to evaluation and approval based on acceptance criteria applicable to those items/services.

Qualification of such suppliers is determined by performance assurance. In the discharge of this responsibility, performance assurance may use information generated by other utilities. The supplier must be

approved before procurement can be completed. I&M is a member of the Nuclear Procurement Issues Committee (NUPIC), participates in joint supplier audits, and shares audit information consistent with NUPIC requirements. The supplier must be acceptable, or acceptable subject to follow-up, before a procurement can be approved and processed. Additional audits will be conducted, as necessary, to meet requirements. Acceptance is not complete until it has been determined that the suppliers' QA program can meet the requirements for the item(s)/service(s) offered.

1.7.7.2.2

For items that are not unique to a nuclear power plant ("Commercial Grade") where application-specific requirements cannot be contractually imposed in a practical manner at the time of procurement, programs for dedication to safety-related standards are established by engineering personnel and accomplished prior to the item being accepted for safety-related use.

1.7.7.2.3

In-process audits of suppliers' activities during fabrication, inspection, testing and shipment of items are performed when deemed necessary, depending upon supplier qualification status, complexity of the item(s) being furnished, the items' importance to safety, and/or previous supplier history. These audits are performed by performance assurance. The cognizant engineer and/or responsible Cook Nuclear Plant personnel may also participate, if deemed necessary.

1.7.7.2.4

Spare and replacement parts are procured in such a manner that their performance and quality are at least equivalent to those of the parts that will be replaced.

- a) Specifications and codes referenced in procurement documents for spare or replacement items are at least equivalent to those for the original items or to properly reviewed and approved revisions.
- b) Parts intended as spares or replacement for "off-the-shelf" items, or other items for which quality requirements were not originally specified, are evaluated for performance at least equivalent to the original.
- c) Where quality requirements for the original items cannot be determined, requirements and controls are established by engineering evaluation performed by qualified individuals. The evaluation assures there is no adverse effect on interfaces, safety, interchangeability, fit, form, function, or compliance with applicable regulatory or code requirements. Evaluation results are documented.
- d) Any additional or modified design criteria, imposed after previous procurement of the item(s), are identified and incorporated.

1.7.7.2.5

Instructions and procedures address the requirements for supplier selection and control, as well as procurement document control. The program for receipt inspection of safety-related items addresses inspection of incoming items, including a review of the documentation required under the procurement. Receipt inspection personnel are qualified and certified in accordance with the requirements of ANSI N45.2.6. Provisions for receipt inspection apply regardless of where the procurement originates. Additional inspections may apply if required by the procurement document.

Items, which have special procurement requirements (such as nuclear fuel and nuclear fuel components), may involve detailed source evaluations or audits at the supplier's facility prior to shipment to supplement receipt inspection. Personnel performing these evaluations and audits will be qualified in accordance with ANSI N18.1 and/or ANSI N45.2.23. Receipt inspections at the site will be performed by personnel certified to ANSI N45.2.6. In addition, reviews of special procurement documents or shipping manifests will be performed by personnel trained in the procurement and qualified in accordance with ANSI N18.1.

Where items and/or services are safety-related and procurement is accomplished without assistance of ~~TCM~~, supplier selection is limited to those companies identified as being qualified.

1.7.7.2.6

Items received at the site are tagged with a "HOLD" tag and/or placed in a designated area (e.g., new nuclear fuel) until receipt inspected. During receipt inspection, designated material characteristics and attributes are checked, and documentation is checked against the procurement documents. When the receipt inspection of items is supplemented by source evaluations or audits at the vendor prior to shipment, appropriate visual and/or mechanical inspections will be completed to ensure that shipping damage has not occurred. If found acceptable, the "HOLD" tag is removed and replaced with an "ACCEPTED" tag and/or the item is placed in a designated area.

Item traceability to procurement documents and to end use is maintained through recording of identification numbers or, "HOLD" and "ACCEPTED" tag numbers on applicable documents.

Nonconforming items, or missing or questionable documentation results in items being placed on "HOLD" and maintained in a designated, controlled area. If the nonconformance cannot be cleared, the item is either scrapped, returned to manufacturer, or dispositioned through engineering analysis.

1.7.7.2.7

Contractors providing services (on-site) for safety-related components are required to have either a formal quality assurance program and procedures, or they must abide by the ~~ISM~~ QA Program and procedures. Prior to their working at

the Cook Nuclear Plant, contractors working under their own quality assurance programs must be audited and approved by ~~performance assurance~~. Contractor procedures must be reviewed and approved by the originating/sponsoring department head. Further, periodic audits of site contractor activities are conducted under the direction of ~~performance assurance~~.

1.7.7.2.8

To the extent prescribed in specific procurement documents, suppliers furnish quality records; documentary evidence that material and equipment either conforms to requirements or identifies any requirements that have not been met; and descriptions of those nonconformances from the procurement requirements, which have been dispositioned "use-as-is" or "repair." This evidence is retained by ~~16M~~.

To the extent prescribed in specific procurement agreements, suppliers are required to maintain additional (backup) documents in their record system.

In some cases, such as with NSSS, suppliers are designated primary record retention responsibility.

1.7.7.2.9

The capability of suppliers to furnish valid documentation is evaluated during procurement document reviews, annual supplier evaluations, and during audits.

1.7.8 IDENTIFICATION AND CONTROL OF ITEMS

1.7.8.1 SCOPE

Items are identified and controlled to prevent their inadvertent use. Identification of items is maintained either on the items, their storage areas or containers, or on records traceable to the items.

1.7.8.2 IMPLEMENTATION

1.7.8.2.1

Controls are established that provide for the identification and control of items (including partially fabricated assemblies).

1.7.8.2.2

Items are identified by physically marking the item or its container, and by maintaining records traceable to the item. The method of identification is such that the quality of the item is not degraded.

1.7.8.2.3

Items are traceable to applicable drawings, specifications, or other pertinent documents to ensure that only correct and acceptable items are used. Verification of traceability is performed and documented prior to release for fabrication, assembly, or installation.

1.7.8.2.4

Requirements for the identification by use of heat number, part number, serial number, etc., are included in ~~engineering~~ specifications and/or the procurement document.

1.7.8.2.5

Separate storage is provided for incorrect or defective items that are on hold and material which has been accepted for use. All safety-related items are appropriately tagged or identified (stamping, etc.) to provide easy identification as to the items' usage status. Records are maintained for the issue of items to provide traceability from storage to end use in the Cook Nuclear Plant.

1.7.8.2.6

When materials are subdivided, appropriate identification numbers are transferred to each section of the material, or traceability is maintained through documentation.

1.7.9 CONTROL OF SPECIAL PROCESSES

1.7.9.1 SCOPE

Special processes are controlled and accomplished by qualified personnel using approved procedures and equipment in accordance with applicable codes, standards, specifications, criteria and other special requirements.

1.7.9.2 IMPLEMENTATION

1.7.9.2.1

Processes subject to special process controls are those for which full verification or characterization by direct inspection is impossible or impractical. Such processes include welding, heat treating, chemical cleaning, application of protective coatings, concrete placement and NDE.

1.7.9.2.2

Special process requirements for chemical cleaning, application of protective coatings and concrete placement are set forth in engineering specifications, dedication plans and/or directives prepared by the responsible cognizant engineer. These documents are reviewed and approved by other personnel with the necessary technical competence.

Special process requirements for welding, heat treating and NDE are set forth in engineering specifications, the Welding Manual and plant procedures. Special process requirements for welding and heat treating are prepared by, or are reviewed and approved by, the cognizant engineer welding. Special process requirements for NDE are prepared by, or are reviewed and approved by, the NDE administrator and/or Cook Nuclear Plant NDE Level III personnel.

Special process procedures, with the exception of welding and heat treating, are prepared by Cook Nuclear Plant personnel with technical knowledge in the discipline

involved. These procedures, which are also reviewed by other personnel with the necessary technical competence, are qualified by testing.

Welding is performed in accordance with procedures contained in the [REDACTED] Welding Manual, or by approved contractor's procedures. These procedures are qualified in accordance with applicable codes, and Procedure Qualification Records are prepared. Weld procedure specifications are reviewed and approved by the [REDACTED] cognizant engineer - welding. Weld procedure qualification documentation is retained in the AEP Welding Manual, or the approved contractor's manual.

Contractor welding procedures are qualified by the contractor. These procedures and the qualification documentation are reviewed and approved by the [REDACTED] cognizant engineer welding. This documentation is retained by the contractor.

1.7.9.2.3

NDE personnel are qualified and certified by a Cook Nuclear Plant NDE Level III who has been qualified and certified by the designated [REDACTED] NDE administrator. Certification is by examination. Personnel qualification is kept current by re-examination at time intervals specified in qualification/ certification procedures which are in accordance with the ASME Code.

Cook Nuclear Plant welders are qualified by the maintenance organization, and/or the project management and installation services organization using approved welding procedure specifications. Administration of Cook Nuclear Plant welder qualifications is performed by the maintenance, and/or the project management and installation service organizations. Examination of qualification specimens is performed under the supervision of the performance assurance organization in accordance with the Welding Manual and nuclear engineering specifications covering welder qualification. Cook Nuclear Plant welder qualification records are maintained for maintenance and contractor welders by nuclear engineering. Contractor welders are qualified by the contractor using procedures approved by the cognizant engineer welding in accordance with I&M procedures. Contractor qualification records are maintained by the contractor.

1.7.9.2.4

QC/NDE technicians assigned to performance assurance perform nondestructive testing for work performed by Cook Nuclear Plant and contractor personnel. These individuals are qualified to either SNT-TC-1A, or ANSI N45.2.6, and records of the qualifications/certifications are maintained by I&M.

1.7.9.2.5

For special processes that require qualified equipment, such equipment is qualified in accordance with applicable codes, standards and specifications.

1.7.9.2.6

Special process qualifications are reviewed during regularly scheduled QA audits. Qualification records are maintained in accordance with 1.7.17 herein.

1.7.9.2.7

The documentation resulting from welding and nondestructive testing is reviewed by appropriate personnel.

1.7.10 INSPECTION

1.7.10.1 SCOPE

Activities affecting the quality of safety-related structures, systems and components are inspected to verify their conformance with requirements. These inspections are performed by personnel other than those who perform the activity. Inspections are performed by qualified personnel utilizing written procedures which establish prerequisites and provide documentation for evaluating test and inspection results. Direct inspection, process monitoring, or both, are used as necessary. When applicable, hold points are used to ensure that inspections are accomplished at the correct points in the sequence of activities.

1.7.10.2 IMPLEMENTATION

1.7.10.2.1

Inspections are applied to appropriate activities to assure conformance to specified requirements.

Hold points are provided in the sequence of procedures to allow for the inspection, witnessing, examination, measurement, or review necessary to assure that the critical, or irreversible, elements of an activity are being performed as required. Note that hold points may not apply to all procedures but each procedure which includes inspections must be reviewed for this attribute.

Hold points specify exactly what is to be done (e.g., type of inspection or examination, etc.), acceptance criteria, or reference to another procedure, etc., for the satisfactory completion of the hold point. When hold points are included in the sequence of a procedure, the activities required by hold points are completed prior to continuing work beyond that point.

Process monitoring is used in whole, or in part, where direct inspection alone is impractical or inadequate.

1.7.10.2.2

Training, qualification and certification programs for personnel who perform inspections are established, implemented and documented in accordance with 1.7.2 herein and as described in Appendix B hereto, item 9b, with exceptions as noted therein.

1.7.10.2.3

Inspection requirements are specified in procedures, instructions, drawings or checklists as applicable. They provide for the following, as appropriate:

- a) Identification of applicable revisions of required instructions, drawings and specifications.
- b) Identification of characteristics and activities to be inspected.
- c) Inspection methods.
- d) Specification of measuring and test equipment having the necessary accuracy.
- e) Identification of personnel responsible for performing the inspection.
- f) Acceptance and rejection criteria.
- g) Recording of the inspection results and the identification of the inspector.

1.7.10.2.4

Inspections are conducted using the following programs:

- a. Peer Inspection Program. The Peer Inspection Program is based on the premise that I&M personnel are qualified to ANSI N18.1 (1971), Selection and Training of Nuclear Power Plant Personnel, and are periodically trained in their skill area using INPO accredited training. As a result of their experience, qualifications, and training, I&M personnel may perform inspections of work functions associated with normal operation of the Plant, routine maintenance, and certain routine technical

activities which are routinely performed by I&M personnel (peers). Peer inspection personnel are independent in that they do not perform or directly supervise the work being inspected, but they may be from the same work group.

- b. ANSI N45.2.6 Inspection Program. Major modification and non-routine maintenance work on safety-related equipment is inspected per ANSI N45.2.6, .
Qualifications of Inspection, Examination, and Testing Personnel for Nuclear Power Plants, whether it is performed by I&M or contractor personnel. All safety-related work performed by contract personnel is inspected per ANSI N45.2.6. Inspections of these work activities are performed by inspectors qualified and certified in accordance with Regulatory Guide 1.58 and ANSI N45.2.6. Contractors performing work on safety-related equipment are required to comply with the applicable requirements of Regulatory Guide 1.33 and ANSI N45.2.

1.7.10.2.5

Inspections associated with the packaging and shipment of radioactive waste and materials are conducted using the following program:

- a) NRC Licensed Packagings - Inspections of NRC licensed radioactive material packagings shall be performed by individuals independent from the work being performed. The independent inspectors shall be I&M personnel, qualified in accordance with Regulatory Guide 1.8 and ANSI N18.1, as a minimum.

Additionally, the inspector shall be familiar with the activities being performed.

- b) Non-NRC Licensed Packagings and Containers -
Inspections of non-NRC licensed radioactive material packagings and containers (shipping and/or burial) shall be performed by ~~I&M~~ personnel, qualified in accordance with Regulatory Guide 1.8 and ANSI N18.1, as a minimum.
- c) Transportation Vehicles - Inspection of transportation vehicles being shipped as "exclusive use", shall be performed by ~~I&M~~ personnel, qualified in accordance with Regulatory Guide 1.8 and ANSI N18.1, as a minimum.
- d) Other inspections and Verification - Inspections and verifications of other activities associated with the packaging and shipment of radioactive materials and waste shall be performed by ~~I&M~~ personnel, qualified in accordance with Regulatory Guide 1.8 and ANSI N18.1, as a minimum.

1.7.10.2.6

Inspections are performed, documented, and the results evaluated by designated personnel in order to ensure that the results substantiate the acceptability of the item or work. Evaluation and review results are documented.

1.7.11 TEST CONTROL

1.7.11.1 SCOPE

Testing is performed in accordance with established programs to demonstrate that structures, systems and components will perform satisfactorily in service. The testing is performed by qualified personnel in accordance with written procedures that incorporate specified requirements and acceptance criteria. Types of tests are:

Scheduled

Surveillance, preventive maintenance, post-design, qualification.

Unscheduled

Pre-maintenance and post-maintenance.

Test parameters (including any prerequisites), instrumentation requirements, and environmental conditions are specified in test procedures. Test results are documented and evaluated.

1.7.11.2 IMPLEMENTATION

1.7.11.2.1

Tests are performed in accordance with programs, procedures and criteria that designate when tests are required and how they are to be performed. Such testing includes the following:

- a) Qualification tests, as applicable, to verify design adequacy.
- b) Acceptance tests of equipment and components to assure their operation prior to delivery or installation.
- c) Post-design tests to assure proper and safe operation of systems and equipment prior to unrestricted operation.
- d) Surveillance tests to assure continuing proper and safe operation of systems and equipment. The PMI on surveillance testing controls the periodic testing of equipment and systems to fulfill the surveillance requirements established by the Technical Specifications. Controls have been established to identify uncompleted surveillance testing to assure it is rescheduled for completion to meet Technical Specification frequency requirements. Data taken during surveillance testing is reviewed by appropriate management personnel to assure that acceptance criteria is fulfilled, or corrective action is taken to correct deficiencies.
- e) Maintenance tests after preventive or corrective maintenance.

1.7.11.2.2

Test procedures, as required, provide mandatory hold points for witness or review.

1.7.11.2.3

Testing is accomplished after installation, maintenance, or repair, by surveillance test procedures, or performance tests, which must be satisfactorily completed prior to determining the equipment is in an operable status, except for Technical Specification equipment addressed in section 1.7.11.2.4 below. All data resulting from these tests is retained at the Cook Nuclear Plant after review by appropriate management personnel.

1.7.11.2.4

Technical Specification equipment can be considered operable prior to the satisfactory completion of tests when Technical Specification surveillance requirements provide an allowance of time to analyze test results.

1.7.12 CONTROL OF MEASURING AND TEST EQUIPMENT

1.7.12.1 SCOPE

Measuring and testing equipment used in activities affecting the quality of safety-related structures, systems and components are properly identified, controlled, calibrated and adjusted at specified intervals to maintain accuracy within necessary limits.

1.7.12.2 IMPLEMENTATION

1.7.12.2.1

Established procedures and instructions are used for calibration and control of measuring and test equipment utilized in the measurement, inspection and monitoring of

structures, systems and components. These procedures and instructions describe calibration techniques and frequencies, and maintenance and control of the equipment.

~~Performance assurance~~ periodically assesses the effectiveness of the calibration program via the ~~audit~~ audit program.

1.7.12.2.2

Measuring and test equipment is uniquely identified and is traceable to its calibration source.

1.7.12.2.3

A system has been established for attaching, or affixing labels, to measuring and test equipment to display the date calibrated and the next calibration due date, or a control system is used that identifies to potential users any equipment beyond the calibration due date.

1.7.12.2.4

Measuring and test equipment is calibrated at specified intervals. These intervals are based on the frequency of use, stability characteristics and other conditions that could adversely affect the required measurement accuracy. Calibration standards are traceable to nationally recognized standards; or where such standards do not exist, provisions are established to document the basis for calibration.

The primary standards used to calibrate secondary standards have, except in certain instances, an accuracy of at least four (4) times the required accuracy of the secondary standard. In those cases where the four (4) times accuracy cannot be achieved, the basis for acceptance is documented and is authorized by the responsible manager. The secondary standards have an accuracy that assures equipment being calibrated will be within required tolerances. The basis for acceptance is documented and authorized by the responsible manager.

1.7.12.2.5

Cook Nuclear Plant procedures define the requirements for the control of standards, test equipment and process equipment.

1.7.12.2.6

When measuring and testing equipment used for inspection and testing is found to be outside of required accuracy limits at the time of calibration, evaluations are conducted to determine the validity of the results obtained since the most recent calibration. Retests, or reinspections, are performed on suspect items. The results of evaluations are documented.

1.7.13 HANDLING, STORAGE, AND SHIPPING

1.7.13.1 SCOPE

Activities with the potential for causing contamination or deterioration, by environmental conditions such as temperature or humidity that could adversely affect the

ability of an item to perform its safety-related functions and activities necessary to prevent damage or loss, are identified and controlled. These activities are cleaning, packaging, preserving, handling, shipping and storing. Controls are effected through the use of appropriate procedures and instructions.

1.7.13.2 IMPLEMENTATION

1.7.13.2.1

Procedures are used to control the cleaning, handling, storing, packaging, preserving and shipping of materials, components and systems in accordance with designated procurement requirements. These procedures include, but are not limited to, the following functions:

- a) Cleaning - to assure that required cleanness levels are achieved and maintained.
- b) Packaging and preservation - to provide adequate protection against damage or deterioration. When necessary, these procedures provide for special environments, such as inert gas atmosphere, specific moisture content levels and temperature levels.
- c) Handling - to preclude damage or safety hazards.
- d) Storing - to minimize the possibility of loss, damage or deterioration of items in storage, including consumables such as chemicals, reagents and lubricants.

1.7.13.2.2

Controls have been established for limited shelf life items such as "O" rings, epoxy, lubricants, solvents and chemicals to assure they are correctly identified, stored and controlled to prevent shelf life expired materials from being used in the Cook Nuclear Plant. Controls are established in plant procedures.

1.7.13.2.3

Packaging and shipping requirements are provided to vendors in engineering specifications (DCCs) which are a part of the procurement document, or are otherwise specified in the procurement document. Controls for receipt inspection, damaged items and special handling requirements at the Cook Nuclear Plant are established by plant procedures. Special controls are provided to assure that stainless steel components and materials are handled with approved lifting slings.

1.7.13.2.4

Storage and surveillance requirements have been established to assure segregation of storage. Special controls have been implemented for critical, high value, or perishable items. Routine surveillance is conducted on stored material to provide inspection for damage, rotation of stored pumps and motors, inspection for protection of exposed surfaces and cleanness of the storage area.

1.7.13.2.5

Special handling procedures have been implemented for the processing of nuclear fuel during refueling outages. These procedures minimize the risk of damage to the new and spent fuel and the possible release of radioactive material when placing the spent fuel into the spent fuel pool.

1.7.14 INSPECTION, TEST, AND OPERATING STATUS

1.7.14.1 SCOPE

Operating status of structures, systems and components is indicated by tagging of valves and switches, or by other specified means, in such a manner as to prevent inadvertent operation. The status of inspections and tests performed on individual items is clearly indicated by markings and/or logging under strict procedural controls to prevent inadvertent bypassing of such inspections and tests.

1.7.14.2 IMPLEMENTATION

1.7.14.2.1

For design change activities, including item fabrication, installation and test, a program exists which specifies the degree of control required for the identification of inspection and test status of structures, systems and components.

Physical identification is used to the extent practical to indicate the status of items requiring inspections, tests, or examinations. Procedures exist which provide for the use of calibration and rejection stickers, tags, stamps and other forms of identification to indicate test and

inspection status. The Clearance Permit System uses various tags to identify equipment and system operability status. Another program establishes a tagging system for lifted leads, etc. For those items requiring calibration, the program provides for physical indication of calibration status by calibration stickers, or a control system is used.

1.7.14.2.2

Application and removal of inspection and welding stamps, and of such status indicators as tags, markings, labels, etc., is controlled by plant procedures.

The inspection status of materials received at the Cook Nuclear Plant is identified in accordance with established instructions. The status is identified as Hold, Hold for Quality Control Clearance, Reject, or Accept.

The inspection status of work in progress is controlled by the use of hold points in procedures. Performance Assurance, or departmental ANSI N18.1 qualified personnel (reference 1.7.10.2.4 herein), inspect an activity at various stages and sign off the procedural inspection steps.

The status of welding is controlled through the use of a weld data block which identifies the inspection and NDE status of each weld.

1.7.14.2.3

Required surveillance test procedures are defined in PMIs. These instructions provide for documenting bypassed tests and rescheduling of the test.

The status of testing after minor maintenance is recorded as part of the Job Order and/or procedure. The status of testing after major maintenance is included as part of the procedure, and includes the performance of functional testing and approval of data by supervisory personnel.

Testing, inspection and other operations important to safety are conducted in accordance with properly reviewed and approved procedures. The PMI for plant procedures requires that procedures be followed as written.

Alteration to the sequence of a procedure can only be accomplished by a procedure change which is subject to the same controls as the original review and approval. When an immediate procedure change is required to continue in-process work or testing and the required complete review and approval process cannot be accomplished, an "On The Spot" change is processed in accordance with the PMI on plant procedures.

1.7.14.2.4

Nonconforming, inoperable, or malfunctioning structures, systems and components are clearly identified by tags, stickers, stamps, etc., and documented to prevent inadvertent use.

1.7.15 NONCONFORMING ITEMS

1.7.15.1 SCOPE

Materials, parts, or components that do not conform to requirements are controlled in order to prevent their inadvertent use. Nonconforming items are identified, documented, segregated when practical and dispositioned. Affected organizations are notified of nonconformances.

1.7.15.2 IMPLEMENTATION

1.7.15.2.1

Items, services, or activities that are deficient in characteristic, documentation, or procedure, which render the quality unacceptable or indeterminate, are identified as nonconforming and any further use is controlled. Nonconformances are documented and dispositioned, and notification is made to affected organizations. Personnel authorized to disposition, conditionally release and close out nonconformances are designated.

The Job Order System and/or the Condition Reports (refer to 1.7.16 herein) are used at Cook Nuclear Plant to identify nonconforming items and initiate corrective action for items which are installed or have been released to the Cook Nuclear Plant. Systems, components, or materials which require repair or inspection are controlled under the Job Order System. In addition, the various procedures identified in 1.7.14 herein provide for identification, segregation and documentation of nonconforming items.

1.7.15.2.2

Nonconforming items are identified by marking, tagging, segregating, or by documented administrative controls. Documentation describes the nonconformance, the disposition of the nonconformance and the inspection requirements. It also includes signature approval of the disposition.

Completed Job Orders are reviewed by the supervisor responsible for accomplishing the work, and the supervisor of the department/section that originated the Job Order. Performance Assurance periodically audits the Job Order System, and on a sample basis, Job Orders.

1.7.15.2.3

Items that have been repaired or reworked are inspected and tested in accordance with the original inspection and test requirements, or alternatives, that have been documented.

Items that have the disposition of "repair" or "use-as-is" require documentation justifying acceptability. The changes are recorded to denote the as-built condition.

When required by established procedures, surveillance or operability tests are conducted on an item after rework, repair or replacement.

1.7.15.2.4

Disposition of conditionally released items are closed out before the items are relied upon to perform safety-related functions.

1.7.16 CORRECTIVE ACTION

1.7.16.1 SCOPE

Conditions adverse to quality, such as failures, malfunctions, deficiencies, deviations, defective material and equipment, and nonconformances are identified promptly and corrected as soon as practical.

For significant conditions adverse to quality, the cause of the condition is determined, corrective action is taken to correct the immediate condition, and preventive action is implemented to prevent recurrence. In these cases, the condition, cause and corrective action taken is documented and reported to appropriate levels of management.

1.7.16.2 IMPLEMENTATION

1.7.16.2.1

Procedures are established that describe ~~ICM's~~ corrective action program. These procedures are reviewed and concurred with by ~~performance assurance~~.

1.7.16.2.2

Condition Reports and audit/surveillance reports provide the mechanism for ~~ICM~~ personnel to notify management of conditions adverse to quality. Condition Reports are also used to report violations to codes, regulations and the Technical Specifications. Investigations of reported conditions adverse to quality are assigned by management. The Condition Report is used to document the investigation of an adverse condition; and to identify the need for a design change to correct system or equipment deficiencies,

or to identify the need for the initiation of Job Orders to correct minor deficiencies. In the case of significant conditions adverse to quality, Condition Reports are used to identify those actions necessary to prevent recurrence of the reported condition.

The PNSRC evaluates actions taken or being taken to correct and prevent recurrence of the deficiency for condition reports involving:

- a. a violation of Technical Specification
- b. a reportable event
- c. any accidental, unplanned, or uncontrolled radioactive release
- d. a safety-related adverse trend
- e. a potential nuclear safety hazard
- f. an entry into Technical Specification LCO 3.0.3 (failure to comply with LCO's not containing specific shutdown schedule)
- g. a Technical Specification LCO entry that was the result of personnel error.

The NSDRC is responsible for assuring that independent reviews of violations (as specified in the Technical Specifications) are performed. These violations are considered significant conditions which are documented on Condition Reports. The reviews will provide an independent evaluation of the reported conditions and corrective actions.

Performance assurance periodically audits the corrective action system for compliance and effectiveness.

1.7.17 QUALITY ASSURANCE RECORDS

1.7.17.1 SCOPE

Records that furnish evidence of activities affecting the quality of safety-related structures, systems and components are maintained. They are accurate, complete, legible and are protected against damage, deterioration, or loss. They are identifiable and retrievable.

1.7.17.2 IMPLEMENTATION

1.7.17.2.1

Documents that furnish evidence of activities affecting the quality of safety-related items are generated and controlled in accordance with the procedure that governs those activities. Upon completion, these documents are considered records. These records include:

- a) Results of reviews, inspections, surveillances, tests, audits and material analyses.
- b) Qualification of personnel, procedures and equipment.
- c) Operation logs.
- d) Maintenance and modification procedures and related inspection results.
- e) Reportable occurrences.
- f) Records required by the plant Technical Specifications.
- g) Condition Reports.
- h) Other documentation such as drawings, specifications, dedication plans, procurement documents, calibration procedures and reports.
- i) Radiographs.

1.7.17.2.2

Instructions and procedures establish the requirements for the identification and preparation of records for systems and equipment under the QA Program, and provide the controls for retention of these records.

Criteria for the storage location of quality related records, and a retention schedule for these records, has been established.

File Indexes have been established to provide direction for filing, and to provide for the retrievability of the records.

Controls have been established for limiting access to the Plant Master File to prevent unauthorized entry, unauthorized removal, and for use of the records under emergency conditions. The Nuclear Records Management Supervisor is responsible for the control and operation of the Plant Master File Room.

1.7.17.2.3

Within ICM, each [REDACTED] manager is responsible for the identification, collection, maintenance and storage of records generated by their [REDACTED] organization. Procedures ensure the maintenance of records sufficient to furnish objective evidence that activities affecting quality are in compliance with the established QA Program.

1.7.17.2.4

When a document becomes a record, it is designated as permanent, or nonpermanent, and then transmitted to file. Nonpermanent records have specified retention times. Permanent records are maintained for the life of the plant or equipment, as applicable.

1.7.17.2.5

Only authorized personnel may issue corrections or supplements to records.

1.7.17.2.6

Traceability between the record and the item or activity to which it applies is provided.

1.7.17.2.7

Except for records that can only be stored as originals, such as radiographs and some strip charts, or micrographs thereof, records are stored in remote, dual facilities to prevent damage, deterioration, or loss due to natural or unnatural causes. When only the single original can be retained, special fire-rated facilities are used.

When optical disk technology is used for records storage, the following quality controls are used:

- The optical disk technology does not allow deletion or modification of record images.
- The image of each record is written onto two optical disks.

- The legibility of each record image is verified to ensure that the image is legible on both disks. If the image is illegible, the hard copy record is maintained as the record copy.
- One optical disk is stored in the document imaging system for on-line retrieval.
- The second (backup) optical disk is stored in a special fire-rated facility or in a separate remote location.
- To ensure permanent retention of records, the records stored on an optical disk are acceptably copied onto a new optical disk before the manufacturer's certified useful life of the original disk is exceeded. This includes verification of the records so copied.
- Periodic random inspections of images stored on optical disks are performed to verify that there has been no degradation of image quality.
- If the optical disk document imaging system in use is to be replaced by an incompatible new system, the records stored on the old system's disks are acceptably converted into the new system before the old system is taken out of service. This includes verification of the records so copied.

1.7.18 AUDITS

1.7.18.1 SCOPE

A comprehensive system of audits is carried out to provide independent evaluation of compliance with, and the effectiveness of, the QA Program including those elements of the program implemented by suppliers and contractors.

The system of audits includes limited scope surveillances, which provide flexibility for more timely coverage of certain activities. Audits are performed in accordance with written procedures or checklists by qualified personnel not having direct responsibility in the areas audited. Audit results are documented and reviewed by management. Follow-up action is taken where indicated.

1.7.18.2 IMPLEMENTATION

1.7.18.2.1 Performance Assurance Responsibilities

The basic responsibility for the assessment of the QA Program is vested in performance assurance. Performance Assurance is primarily responsible for ensuring that proper QA programs are established and for verification of their implementation. These responsibilities are discharged in cooperation with T&M management and their staffs.

1.7.18.2.2

Internal audits are performed in accordance with established schedules that reflect the status and importance of safety to the activities being performed. All areas where the requirements of 10CFR50, Appendix B apply are audited within a period of one to two years.

1.7.18.2.3

Performance Assurance conducts audits to verify the adequacy and implementation of the QA Program at T&M and within the AEP System. QA audit reports are distributed to the appropriate management and the NSDRC (all audits).

1.7.18.2.4

The independent off-site review and audit organization is the [REDACTED] NSDRRC. This committee is composed of [REDACTED] management members. An NSDRRC Manual has been developed for this committee which contains the NSDRRC Charter and procedures. The NSDRRC conducts periodic audits of Cook Nuclear Plant operations pursuant to established criteria (Technical Specifications, etc.).

NSDRRC audit reports are submitted for review to the NSDRRC membership and the Chairman of the NSDRRC. Condition Reports and/or audit reports provide for the recording of actions taken to correct deficiencies found during these audits.

1.7.18.2.5

The [REDACTED] on-site review group is the PNSRC. This committee reviews plant operations as a routine evaluation and serves to advise the site vice president [REDACTED] on matters related to nuclear safety. The composition of the committee is defined in the Technical Specifications.

The PNSRC also reviews instructions, procedures, and design changes for safety-related systems prior to approval by the site vice president [REDACTED]. In addition, this committee serves to conduct investigations of violations to Technical Specifications, and reviews significant Condition Reports to determine if appropriate action has been taken.

1.7.18.2.6

Audits of suppliers and contractors are scheduled based on the status of safety importance of the activities being performed, and are initiated early enough to assure effective quality assurance during design, procurement, manufacturing, construction, installation, inspection and testing.

Principal contractors are required to audit their suppliers systematically in accordance with the criteria established within their quality assurance programs.

1.7.18.2.7

Regularly scheduled audits are supplemented by "special audits" when significant changes are made in the QA program, when it is suspected that quality is in jeopardy, or when an independent assessment of program effectiveness is considered necessary.

1.7.18.2.8

Audits include an objective evaluation of practices, procedures, instructions, activities and items related to quality; and a review of documents and records to confirm that the QA Program is effective and properly implemented.

1.7.18.2.9

Audit procedures and the scope, plans, checklists and results of individual audits are documented.

1.7.18.2.10

Personnel selected for auditing assignments have experience, or are given training commensurate with the needs of the audit, and have no direct responsibilities in the areas audited.

1.7.18.2.11

Management of the audited organization identifies and takes appropriate action to correct observed deficiencies. In the case of significant conditions adverse to quality, appropriate action is taken to prevent recurrence. Follow-up is performed by the auditing organization on selected adverse conditions, to ensure that the appropriate actions were taken. Such follow-up actions may include, but are not limited to, re-audits, subsequent audits, surveillances, or other appropriate means.

1.7.18.2.12

The adequacy of the QA Program is regularly assessed by management. The following activities constitute formal elements of that assessment:

- a) Audit reports, including follow-up on corrective action accomplishment and effectiveness, are distributed to appropriate levels of management.

- b) Individuals independent from the QA organization, but knowledgeable in auditing, and quality assurance, periodically review the effectiveness of the QA programs. Conclusions and recommendations are reported to the ~~ISM~~ vice president ~~ISM~~.

1.7.19 FIRE PROTECTION QA PROGRAM

1.7.19.1 Introduction

The Cook Nuclear Plant Fire Protection QA Program has been developed using the guidance of NRC Branch Technical Position (APCSB) 9.5-1, Appendix A, Section C, "Quality Assurance Program," and NRC clarification "Nuclear Plant Fire Protection Functional Responsibilities, Administrative Controls, and Quality Assurance," dated June 14, 1977. As such, the Fire Protection QA Program is part of the overall QA Program for the plant. The Fire Protection QA Program encompasses design, procurement, fabrication, construction, surveillance, inspection, operation, maintenance, modification, and audits.

Implementation and assessment of the Fire Protection QA Program is the responsibility of ~~ISM~~

1.7.19.2 Organization

The Fire Protection QA Program is under the management and control of I&M. This control consists of:

- 1) Verifying the effectiveness of the Fire Protection QA Program through review, surveillance, and audits.
- 2) Directing formulation, implementation, and assessment of the Fire Protection QA Program by procedural controls.
- 3) Assuring the QA program is acceptable to the management responsible for fire protection.

The site vice president has delegated responsibility to various Cook Nuclear Plant departments for the following fire protection activities:

- a) Maintenance of fire protection systems.
- b) Testing of fire protection equipment.
- c) Fire safety inspections.
- d) Fire pre-plans.
- e) Fire drills.
- f) Emergency remote shutdown procedures.
- g) Emergency repair procedures (10CFR50, Appendix R).

The Fire Protection QA Program at the Cook Nuclear Plant also provides for inspection of fire hazards, explosion hazards, and training of fire brigade and responding fire departments.

The plant protection department's fire protection shift supervisor on duty, or designee, is designated as the fire brigade leader and coordinates the fire fighting efforts of the fire brigade. The operations department provides an individual with plant systems knowledge to serve as an advisor to the fire brigade leader.

1.7.19.3 Design Control and Procurement Document Control

Quality standards are specified in the design documents such as appropriate fire protection codes and standards, and, as necessary, deviations and changes from these quality standards are controlled.

The Cook Nuclear Plant design was reviewed by qualified personnel to ensure inclusion of appropriate fire protection requirements. These reviews include items such as:

- 1) Verification as to the adequacy of electrical isolation and cable separation criteria.
- 2) Verification of appropriate requirements for room isolation (sealing penetrations, floors and other fire barriers).
- 3) Determination for increase in fire loadings.
- 4) Determination for the need of additional fire detection and suppression equipment.

Procurement of fire protection equipment and related items are subject to the requirements of the fire protection

procurement documents. A review of these documents is performed to assure fire protection requirements and quality requirements are correctly stated, verifiable, and controllable, and that there is adequate acceptance and rejection criteria. Procurement documents must be prepared, reviewed, and approved according to QA Program requirements.

Design and procurement document changes, including field changes and design deviations, are controlled by procedure.

1.7.19.4 Instructions, Procedures and Drawings

Inspections, tests, administrative controls, fire drills and training that assist in implementing the fire protection program are prescribed by approved instructions or procedures.

Indoctrination and training programs for fire prevention and fire fighting are implemented in accordance with approved procedures. Activities associated with the fire protection systems and fire protection related systems are prescribed and accomplished in accordance with documented instructions, procedures, and drawings. Instructions and procedures for design, installation, inspection, tests, maintenance, modification and administrative controls are reviewed through audits to assure that the fire protection program is maintained.

Operation and maintenance information has been provided to the plant in the form of System Descriptions and equipment supplier information.

1.7.19.5 Control of Purchased Items and Services

Measures are established to assure that purchased items and services conform to procurement documents. These measures include provisions, as appropriate, for source evaluation and selection, objective evidence of quality furnished by the contractor, inspections at suppliers, or receipt inspection.

Source or receipt inspection is provided, as a minimum, for those items where quality cannot be verified after installation.

1.7.19.6 Inspection

A program for independent inspection of the fire protection activities has been established and implemented.

These inspections are performed by personnel other than those responsible for implementation of the activity. The inspections include:

- a) Inspection of installation, maintenance and modification of fire protection systems and equipment.
- b) Inspections of penetration seals and fire retardant coating installations to verify the activity is satisfactorily completed in accordance with installation specifications.

- c) Inspections of cable routing to verify conformance with design requirements as specified in engineering specifications and/or plant procedures.
- d) Inspections to verify that appropriate requirements for fire barriers are satisfied following installation, modification, repair or replacement activities.
- e) Measures to assure that inspection personnel are independent from the individuals performing the activity being inspected and are knowledgeable in the design and installation requirements for fire protection.
- f) Inspection procedures, instructions or checklists for required inspections.
- g) Periodic inspections of fire protection systems, emergency breathing and auxiliary equipment.
- h) Periodic inspections of materials subject to degradation, such as fire stops, seals and fire retardant coating as required by Technical Specifications or manufacturer's recommendations.

1.7.19.7 Test and Test Control

- a) Installation testing - Following installation, modification, repair, or replacement, sufficient testing is performed to demonstrate that the fire protection systems and equipment will perform

satisfactorily. Written test procedures for installation tests incorporate the requirements and acceptance limits contained in applicable design documents.

- b) Periodic testing - Periodic testing occurs to document that fire protection equipment functions in accordance with its design.
- c) Programs have been established to verify the testing of fire protection systems, and to verify that test personnel are effectively trained.
- d) Test results are documented, evaluated, and their acceptability determined by a qualified responsible individual or group.

1.7.19.8 Inspection, Test and Operating Status

The inspection, test and operating status for plant Technical Specification fire protection systems are performed as described in 1.7.14 herein.

1.7.19.9 Nonconforming Items

Technical Specification fire protection equipment nonconformances are identified and dispositioned as described in 1.7.15 herein.

1.7.19.10 Corrective Action

The corrective action mechanism described in 1.7.16 herein applies to the Technical Specification fire protection equipment.

1.7.19.11 Records

Records that furnish evidence of the quality of activities and of systems, structures and components associated with the fire protection program are maintained. The maintenance of the records includes assuring that records are accurate, complete, legible, and protected against damage, deterioration, or loss. The records are identifiable and retrievable. The records include results of reviews, inspections, tests, audits, monitoring of work performance, and qualifications of personnel and equipment. Inspection and test records identify the inspector or data recorder, the type of observations, results, acceptability, and actions taken in connection with any deficiencies noted. Records provide for traceability of activities that occur at the plant that affect the quality of fire protection systems, structures, and components.

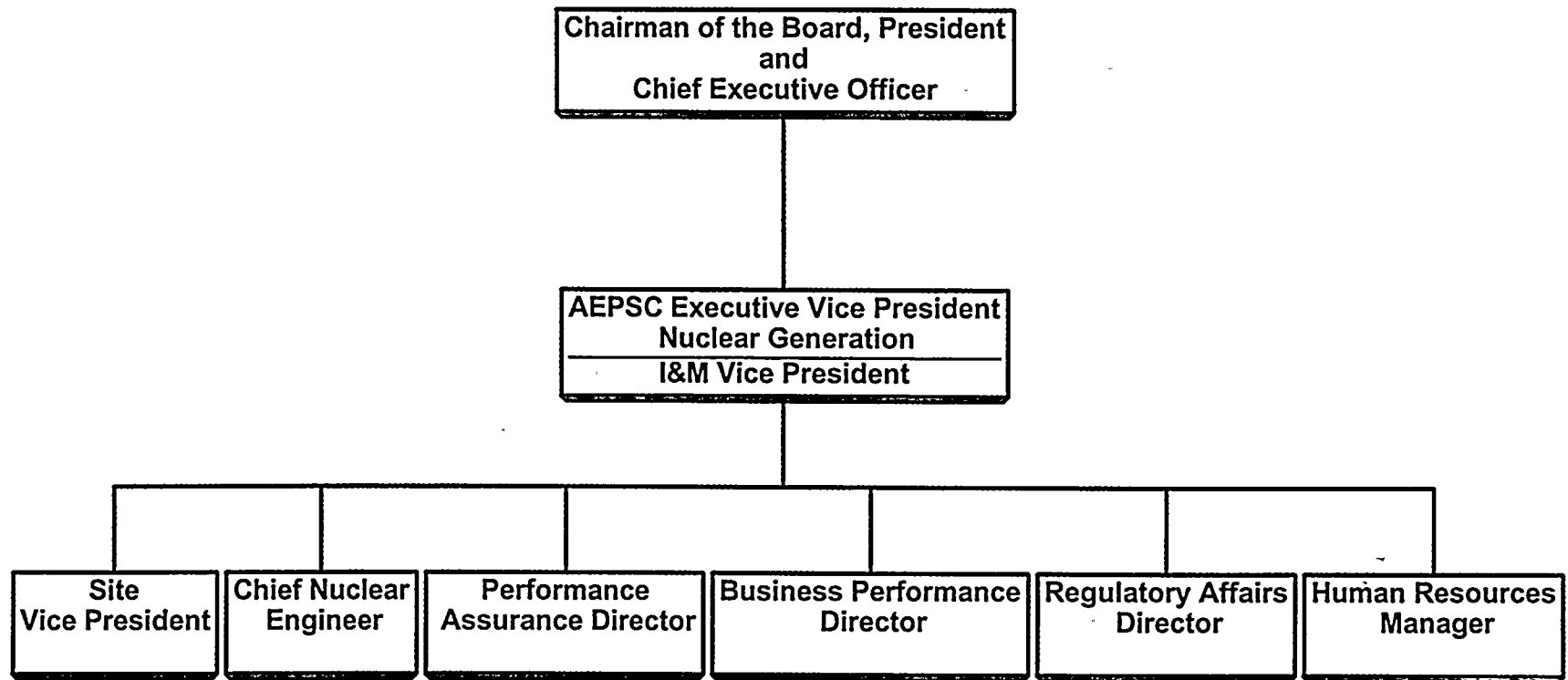
1.7.19.12 Audits

Audits are conducted and documented to verify compliance with the Fire Protection QA Program as described in 1.7.18.1 herein.

Audits are periodically performed to verify compliance with the administrative controls and implementation of fire

protection quality assurance criteria. The audits are performed in accordance with pre-established written procedures or checklists. Audit results are documented and reviewed by management having responsibility in the area audited. Follow-up action is taken by responsible management to correct the deficiencies revealed by the audit.

AMERICAN ELECTRIC POWER

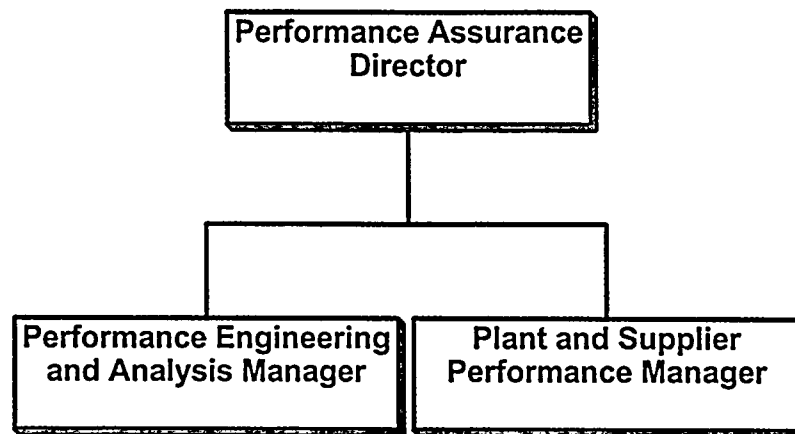


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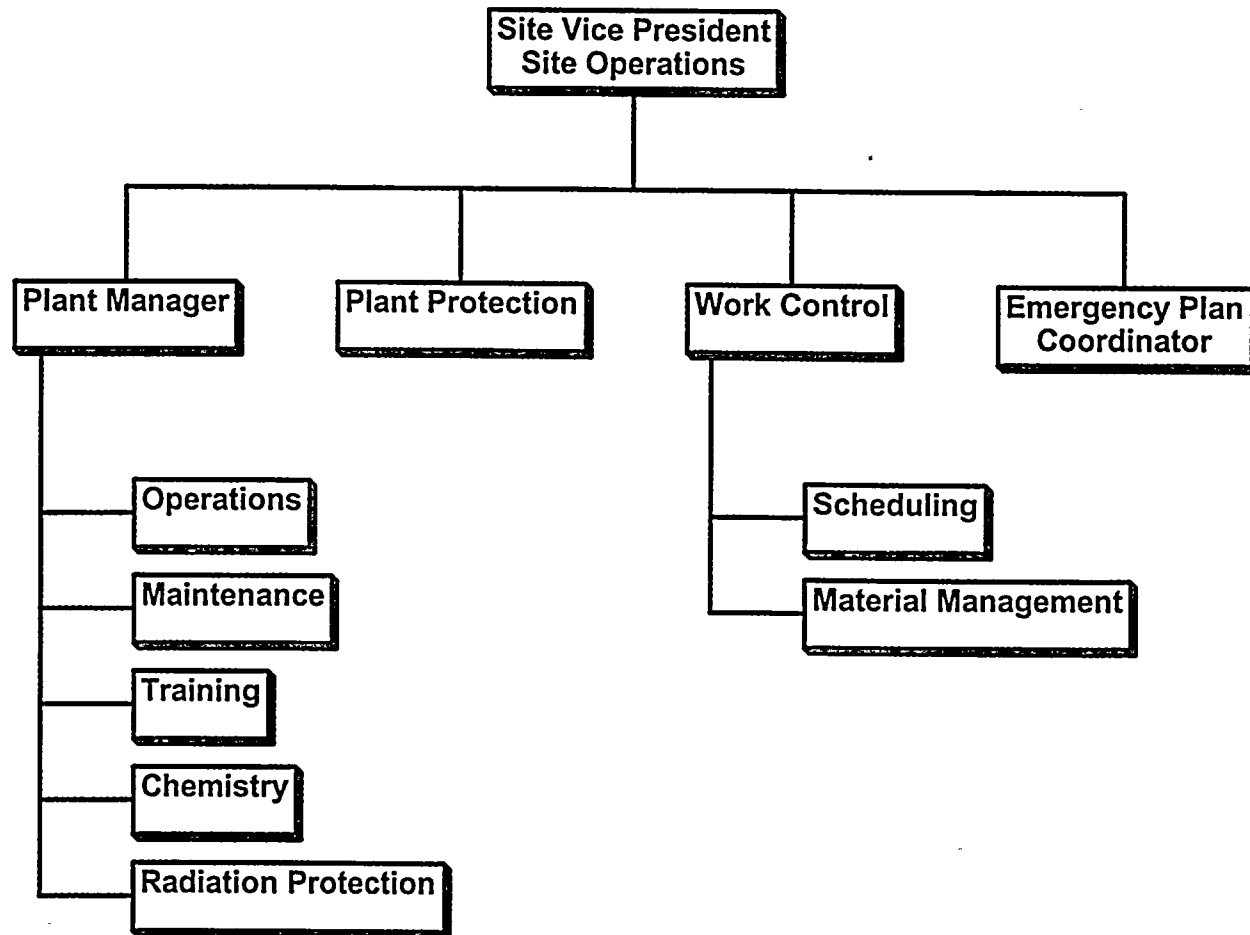
July 1997

Figure 1.7-1

PERFORMANCE ASSURANCE



SITE OPERATIONS



REGULATORY AND SAFETY GUIDES/ANSI STANDARDS

- 1.7-111**

July 1997

(ANS 3.2 1976)

Assurance for the Operational
Phase of Nuclear Power Plants

ANSI N45.2 (1977)

-

Quality Assurance Program
Requirements for Nuclear
Facilities

6. Reg. Guide 1.37 (3/73) -

Quality Assurance
Requirements for Cleaning of
Fluid Systems and Associated
Components of Water-Cooled
Nuclear Power Plants

ANSI N45.2.1 (1973)

-

Cleaning of Fluid Systems and
Associated Components During
Construction Phase of Nuclear
Power Plants

7. Reg. Guide 1.38 (10/76) -

Quality Assurance
Requirements for Packaging,
Shipping, Receiving, Storage
and Handling of Items for
Water-Cooled Nuclear Power
Plants

ANSI N45.2.2 (1972)

-

Packaging, Shipping,
Receiving, Storage and
Handling of Items for Nuclear
Power Plants (During the
Construction Phase)



8. Reg. Guide 1.39 (10/76) - Housekeeping Requirements for Water-Cooled Nuclear Power Plants
- ANSI N45.2.3 (1973) - Housekeeping During the Construction Phase of Nuclear Power Plants
9. Reg. Guide 1.54 (6/73) - Quality Assurance Requirements for Protective Coatings Applied to Water-Cooled Nuclear Power Plants
- ANSI N101.4 (1972) - Quality Assurance for Protective Coatings Applied to Nuclear Facilities
10. Reg. Guide 1.58 (9/80) - Qualification of Nuclear Power Plant Inspection, Examination and Testing Personnel
- ANSI N45.2.6 (1978) - Qualifications of Inspection, Examination, and Testing Personnel for Nuclear Power Plants
11. Reg. Guide 1.63 (7/78) - Electric Penetration Assemblies in Containment Structures for Light-Water-Cooled Nuclear Power Plants

12. Reg. Guide 1.64 (10/73) - Quality Assurance
Requirements for the Design
of Nuclear Power Plants
- ANSI N45.2.11 (1974) - Quality Assurance
Requirements for the Design
of Nuclear Power Plants
13. Reg. Guide 1.74 (2/74) - Quality Assurance Terms and
Definitions
- ANSI N45.2.10 (1973) - Quality Assurance Terms and
Definitions
14. Reg. Guide 1.88 (10/76) - Collection, Storage, and
Maintenance of Nuclear Power
Plant Quality Assurance
Records
- ANSI N45.2.9 (1974) - Requirements for Collection,
Storage, and Maintenance of
Quality Assurance Records for
Nuclear Power Plants
15. Reg. Guide 1.94 (4/76) - Quality Assurance
Requirements for
Installation, Inspection, and
Testing of Structural
Concrete and Structural Steel
During the Construction Phase
of Nuclear Power Plants



- ANSI N45.2.5 (1974) - Supplementary Quality Assurance Requirements for Installation, Inspection, and Testing of Structural Concrete and Structural Steel During the Construction Phase of Nuclear Power Plants
16. Reg. Guide 1.123 (7/77) - Quality Assurance Requirements for Control of Procurement of Items and Services for Nuclear Power Plants
- ANSI N45.2.13 (1976) - Quality Assurance Requirements for Control of Procurement of Items and Services for Nuclear Power Plants
17. Reg. Guide 1.144 (1/79) - Auditing of Quality Assurance Programs for Nuclear Power Plants
- ANSI N45.2.12 (1977) - Requirements for Auditing of Quality Assurance Programs for Nuclear Power Plants
18. Reg. Guide 1.146 (8/80) - Qualification of Quality Assurance Program Audit Personnel for Nuclear Power Plants

- ANSI N45.2.23 (1978) - Qualification of Quality Assurance Program Audit Personnel for Nuclear Power Plants
19. ANSI N45.2.8 (1975) - Supplementary Quality Assurance Requirements for Installation, Inspection and Testing of Mechanical Equipment and Systems for the Construction Phase of Nuclear Power Plants
20. ANSI N45.4 (1972) - Leakage-Rate Testing of Containment Structures for Nuclear Reactors

APPENDIX B

I&M EXCEPTIONS TO OPERATING PHASE STANDARDS AND REGULATORY GUIDES

1. GENERAL

Requirement

Certain Regulatory Guides invoke, or imply, Regulatory Guides and standards in addition to the standard each primarily endorses.

Certain ANSI Standards invoke, or imply, additional standards.

Exception/Interpretation

The I&M commitment refers to the Regulatory Guides and ANSI Standards specifically identified in Appendix A. Additional Regulatory Guides, ANSI Standards and similar documents implied, or referenced, in those specifically identified are not part of this commitment.

2. N18.7, General

Exception/Interpretation

I&M has established both an on-site and off-site standing committee for independent review activities; together they form the independent review body.

The standard numeric and qualification requirement may not be met by each group individually. Procedures will be established to specify how each group will be involved in review activities. This exception/interpretation is consistent with the plant's Technical Specifications.

2a. Sec. 4.3.1

Requirement

"Personnel assigned responsibility for independent reviews shall be specified in both number and technical disciplines, and shall collectively have the experience and competence required to review problems in the following areas:"

Exception/Interpretation

~~The~~ Nuclear Safety and Design Review Committee (NSDRC) and Plant Nuclear Safety Review Committee (PNSRC) will not have members specified by number, nor by technical disciplines, and its members may not have the experience and competence required to review problems in all areas listed in this section. This exception/interpretation is consistent with the plant's Technical Specifications.

The NSDRC and PNSRC will not specifically include a member qualified in nondestructive testing, but will use qualified technical consultants to perform this and other functions as determined necessary by the respective committee chairman.

2b. Sec. 4.3.2.1

Requirement

"When a standing committee is responsible for the independent review program, it shall be composed of no less than five persons of whom no more than a minority are members of ~~site operations~~. Competent alternates are permitted if designated in advance. The use of alternates shall be restricted to legitimate absences of principals."

Exception/Interpretation

See Item 2a.

2c. Sec. 4.3.3.1

Requirement

"... recommendations ... shall be disseminated promptly to appropriate members of management having responsibility in the area reviewed."

Exception/Interpretation

Recommendations made as a result of review will generally be conveyed to the on-site, or off-site, standing committee. Procedures will be maintained specifying how recommendations are to be considered.

2d. Sec. 4.3.4

Requirement

"The following subjects shall be reviewed by the independent review body:"

Exception/Interpretation

Subjects requiring review will be as specified in the plant Technical Specifications.

2e. Sec. 4.3.4(3)

Requirement

"Changes in the Technical Specifications or License Amendments relating to nuclear safety are to be reviewed by the independent review body prior to implementation, except in those cases where the change is identical to a previously reviewed proposed change."

Exception/Interpretation

Although the usual practice is to meet this requirement, exceptions are made to NSDRC review and approval prior to implementation in rare cases with the permission of the NSDRC Chairman and Secretary. PNSRC review and approval is always done prior to implementation of Technical Specification changes.

2f. Sec. 4.4

Requirement

"The on-site operating organization shall provide, as part of the normal duties of plant supervisory personnel"

Exception/Interpretation

Some of the responsibilities of the on-site operating organization described in Section 4.4 may be carried out by the PNSRC and/or NSDRC as described in plant Technical Specifications.

2g. Sec. 5.2.2

Requirement

"Temporary changes, which clearly do not change the intent of the approved procedure, shall as a minimum be approved by two members of the plant staff knowledgeable in the areas affected by the procedures. At least one of these individuals shall be the supervisor in charge of the shift and hold a senior operator's license on the unit affected."

Exception/Interpretation

I&M considers that this requirement applies only to procedures identified in plant Technical Specifications. Temporary changes to these procedures shall be approved as described in plant Technical Specifications.

2h. Sec. 5.2.6

Requirement

"In cases where required documentary evidence is not available, the associated equipment or materials must be considered nonconforming in accordance with Section 5.2.14. Until suitable documentary evidence is available to show the equipment or material is in conformance, affected systems shall be considered to be inoperable and reliance shall not be placed on such systems to fulfill their intended safety functions."

Exception/Interpretation

I&M initiates appropriate corrective action when it is discovered that documentary evidence does not exist for a test or inspection which is a requirement to verify equipment acceptability. This action includes a technical evaluation of the equipment's operability status.

2i. Sec. 5.2.

Requirement

"A surveillance testing and inspection program ... shall include the establishment of a master surveillance schedule reflecting the status of all planned in-plant surveillance tests and inspections."

Exception/Interpretation

Separate master schedules may exist for different programs, such as ISI, pump and valve testing, and Technical Specification surveillance testing.

2j. Sec. 5.2.13.1

Requirement

"To the extent necessary, procurement documents shall require suppliers to provide a Quality Assurance Program consistent with the pertinent requirements of ANSI N45.2 - 1977."

Exception/Interpretation

To the extent necessary, procurement documents require that the supplier has a documented Quality Assurance Program consistent with the pertinent requirements of 10CFR50, Appendix B; ANSI N45.2; or other nationally recognized codes and standards.

2k. Sec. 5.2.13.2

Requirement

ANSI N18.7 and N45.2.13 specify that where required by code, regulation, or contract, documentary evidence that items conform to procurement requirements shall be available at the nuclear power plant site prior to installation or use of such items.

Exception/Interpretation

The required documentary evidence is available at the site prior to use, but not necessarily prior to installation. This allows installation to proceed while any missing documents are being obtained, but precludes dependence on the item for safety purposes.

2l. Sec. 5.2.15

Requirement

"Plant procedures shall be reviewed by an individual knowledgeable in the area affected by the procedure no less

frequently than every two years to determine if changes are necessary or desirable."

Exception/Interpretation

Biennial reviews are not performed in that I&M has programmatic control requirements in place that make the biennial review process redundant from a regulatory perspective. These programmatic controls were effected in an effort to ensure that plant instructions and procedures are reviewed for possible revision when pertinent source material is revised, therefore maintaining the procedures current. We believe that this approach, in addition to an annual random sampling of procedures, better addresses the intent of the biennial review process and is more acceptable from both a technical and practical perspective than a static two-year review process.

2m. Sec. 5.2.16

Requirement

Records shall be made, and equipment suitably marked, to indicate calibration status.

Exception/Interpretation

See Item 6b.

2n. Sec. 5.3.5(4)

Requirement

This section requires that where sections of documents such as vendor manuals, operating and maintenance instructions, or drawings are incorporated directly, or by reference into a maintenance procedure, they shall receive the same level of review and approval as operating procedures.

Exception/Interpretation

Such documents are reviewed by appropriately qualified personnel prior to use to ensure that, when used as instructions, they provide proper and adequate information to ensure the required quality of work. Maintenance procedures which reference these documents receive the same level of review and approval as operating procedures.

3. N45.2.1,

3a. Sec. 3

Requirement

N45.2.1 establishes criteria for classifying items into "cleanness levels," and requires that items be so classified.

Exception/Interpretation

Instead of using the cleanness level classification system of N45.2.1, the required cleanness for specific items and activities is addressed on a case-by-case basis.

Cleanness is maintained, consistent with the work being performed, so as to prevent the introduction of foreign material. As a minimum, cleanness inspections are performed prior to closure of "nuclear" systems and equipment. Such inspections are documented.

3b. Sec. 5

Requirement

"Fitting and tack-welded joints (which will not be immediately sealed by welding) shall be wrapped with polyethylene or other nonhalogenated plastic film until the welds can be completed."

Exception/Interpretation

I&M sometimes uses other nonhalogenated material, compatible with the parent material, since plastic film is subject to damage and does not always provide adequate protection.

4. N45.2.2, General

Requirement

N45.2.2 establishes requirements and criteria for classifying safety-related items into protection levels.

Exception/Interpretation

Instead of classifying safety-related items into protection levels, controls over the packaging, shipping, handling and storage of such items are established on a case-by-case basis with due regard for the item's complexity, use and sensitivity to damage. Prior to installation or use, the items are inspected and serviced, as necessary, to assure that no damage or deterioration exists which could affect their function.

4a. Sec. 3.9 and Appendix A3.9

Requirement

"The item and the outside of containers shall be marked."
(Further criteria for marking and tagging are given in the Appendix.)

Exception/Interpretation

These requirements were originally written for items packaged and shipped to construction projects. Full compliance is not always necessary in the case of items shipped to operating plants and may, in some cases, increase the probability of damage to the item. The

requirements are implemented to the extent necessary to assure traceability and integrity of the item.

4b. Sec. 5.2.2

Requirement

"Receiving inspections shall be performed in an area equivalent to the level of storage."

Exception/Interpretation

Receiving inspection area environmental controls may be less stringent than storage environmental requirements for an item. However, such inspections are performed in a manner and in an environment which do not endanger the required quality of the item.

4c. Sec. 6.2.4

Requirement

"The use or storage of food, drinks and salt tablet dispensers in any storage area shall not be permitted."

Exception/Interpretation

Packaged food for emergency or extended overtime use may be stored in material stock rooms. The packaging assures that materials are not contaminated. Food will not be "used" in storage areas.

4d. Sec. 6.3.4

Requirement

"All items and their containers shall be plainly marked so that they are easily identified without excessive handling or unnecessary opening of crates and boxes."

Exception/Interpretation

See N45.2.2, Section 3.9 (Exception 4a. above).

4e. Sec. 6.4.1

Requirement

"Inspections and examinations shall be performed and documented on a periodic basis to assure that the integrity of the item and its container ... is being maintained."

Exception/Interpretation

The requirement implies that all inspections and examinations of items in storage are to be performed on the same schedule. Instead, the inspections and examinations are performed in accordance with material storage procedures which identify the characteristics to be inspected and include the required frequencies. These procedures are based on technical considerations which recognize that inspections and frequencies needed vary from item to item.

5. N45.2.3,

5a. Sec. 2.1

Requirement

Cleanliness requirements for housekeeping activities shall be established on the basis of five zone designations.

Exception/Interpretation

Instead of the five-level zone designation system referenced in ANSI N45.2.3, I&M bases its controls over housekeeping activities on a consideration of what is necessary and appropriate for the activity involved. The controls are effected through procedures or instructions. Factors considered in developing the procedures and

instructions include cleanliness control, personnel safety, fire prevention and protection, radiation control and security. The procedures and instructions make use of standard janitorial and work practices to the extent possible. However, in preparing these procedures, consideration is also given to the recommendations of Section 2.1 of ANSI N45.2.3.

6. N45.2.4,

6a. Sec. 2.2

Requirement

Section 2.2 establishes prerequisites which must be met before the installation, inspections and testing of instrumentation and electrical equipment may proceed. These prerequisites include personnel qualification, control of design, conforming and protected materials and availability of specified documents.

Exception/Interpretation

During the operations phase, this requirement is considered to be applicable to modifications and initial start-up of electrical equipment. For routine or periodic inspection and testing, the prerequisite conditions will be achieved, as necessary.

6b. Sec. 6.2.1

Requirement

"Items requiring calibration shall be tagged or labeled on completion, indicating date of calibration and identity of person that performed calibration."

Exception/Interpretation

Frequently, physical size and/or location of installed plant instrumentation precludes attachment of calibration labels or tags. Instead, each instrument is uniquely identified and is traceable to its calibration record.

A scheduled calibration program assures that each instrument's calibration is current.

7. N45.2.5,

7a. Sec. 2.5.2

Requirement

"When discrepancies, malfunctions or inaccuracies in inspection and testing equipment are found during calibration, all items inspected with that equipment since the last previous calibration shall be considered unacceptable until an evaluation has been made by the responsible authority and appropriate action taken."

Exception/Interpretation

I&M uses the requirements of N18.7, Section 5.2.16, rather than N45.2.5, Section 2.5.2. The N18.7 requirements are more applicable to an operating plant.

7b. Sec. 5.4

Requirement

"Hand torque wrenches used for inspection shall be controlled and must be calibrated at least weekly and more often if deemed necessary. Impact torque wrenches used for inspection must be calibrated at least twice daily."

Exception/Interpretation

Torque wrenches are controlled as measuring and test equipment in accordance with ANSI N18.7, Section 5.2.16. Calibration intervals are based on use and calibration history rather than as per N45.2.5.

8. N45.2.6, Sec. 1.2

Requirement

"The requirements of this standard apply to personnel who perform inspections, examinations and tests during fabrication prior to or during receipt of items at the construction site, during construction, during preoperational and start-up testing and during operational phases of nuclear power plants."

Exception/Interpretation

Personnel participating in testing who take data or make observations, where special training is not required to perform this function, need not be qualified in accordance with ANSI N45.2.6, but need only be trained to the extent necessary to perform the assigned function.

9. Reg. Guide 1.58 - General

Requirement

Qualification of nuclear power plant inspection, examination and testing personnel.

9a. C.2.a(7)

Requirement

Regulatory Guide 1.58 endorses the guidelines of SNT-TC-1A as an acceptable method of training and certifying personnel conducting leak tests.

Exception/Interpretation

I&M takes the position that the "Level" designation guidelines as recommended in SNT-TC-1A, paragraph 4 do not necessarily assure adequate leak test capability. I&M maintains that departmental supervisors are best able to judge whether engineers and other personnel are qualified to direct and/or perform leak tests. Therefore, I&M does not implement the recommended "Level" designation guidelines.

It is I&M's opinion that the training guidelines of SNT-TC-1A, Table I-G, paragraph 5.2 specifically are oriented towards the basic physics involved in leak testing, and further, towards individuals who are not graduate engineers. I&M maintains that it meets the essence of these training guidelines. The preparation of leak test procedures and the conduct of leak tests at Cook Nuclear Plant is under the direct supervision of performance engineers who hold engineering degrees from accredited engineering schools. The basic physics of leak testing have been incorporated into the applicable test procedures. The review and approval of the data obtained from leak tests is performed by department supervisors who are also graduate engineers.

I&M does recognize the need to assure that individuals involved in leak tests are fully cognizant of leak test procedural requirements and thoroughly familiar with the test equipment involved. Plant performance engineers receive routine, informal orientation on testing programs to ensure that these individuals fully understand the requirements of performing a leak test.

9b. C5, C6, C7, C8, C10

Exception/Interpretation

I&M takes the position that the classification of inspection, examination and test personnel (inspection personnel) into "Levels" based on the requirements stated in Section 3.0 of ANSI N45.2.6 does not necessarily assure adequate inspection capability. I&M maintains that departmental and first line supervisors are best able to judge the inspection capability of the personnel under their supervision, and that "Level" classification would require an overly burdensome administrative work load, could inhibit inspection activities, and provides no assurance of inspection capabilities. Therefore, I&M does not implement the "Level" classification concept for inspection, examination and test personnel.

The methodology under which inspections, examinations and tests are conducted at the Cook Nuclear Plant requires the involvement of first line supervisors, engineering personnel, departmental supervisors and plant management. In essence, the last seven (7) project functions shown in Table 1 to ANSI N45.2.6 are assigned to supervisory and engineering personnel, and not to personnel of the inspector category. These management supervisory and engineering personnel, as a minimum, meet the educational and experience requirements of "Level II and Level III" personnel, as required, to meet the criteria of ANSI 18.1 which exceeds those of ANSI N45.2.6. In I&M's opinion, no useful purpose is served by classification of management, supervisory and engineering personnel into "Levels."

Therefore, I&M takes the following positions relative to regulatory positions C5, 6, 7, 8 and 10 of Regulatory Guide 1.58.

- C-5 Based on the discussion in 9b, this position is not applicable to the Cook Nuclear Plant.
- C-6 Replacement personnel for Cook Nuclear Plant management, supervisory and engineering positions subject to ANSI 18.1 will meet the educational and experience requirements of ANSI 18.1 and therefore, those of ANSI N45.2.6.

Replacement inspection personnel will, as a minimum, meet the educational and experience requirements of ANSI N45.2.6, Section 3.5.1 - "Level I."

- C-7 I&M, as a general practice, complies with the training recommendations as set forth in this regulatory position.
- C-8 All I&M inspection, examination and test personnel are instructed in the normal course of employee training in radiation protection and the means to minimize radiation dose exposure.
- C-10 I&M maintains documentation to show that inspection personnel meet the minimum requirements of "Level I," and that management, supervisory and engineering personnel meet the minimum requirements of ANSI 18.1.

10. N45.2.8,

10a. Sec. 2.9e

Requirement

Section 2.9e of N45.2.8. lists documents relating to the specific stage of installation activity which are to be available at the construction site.

Exception/Interpretation

All of the documents listed are not necessarily required at the construction site for installation and testing. I&M assures that they are available to the site, as necessary.

10b. Sec. 2.9e

Requirement

Evidence that engineering or design changes are documented and approved shall be available at the construction site prior to installation.

Exception/Interpretation

Equipment may be installed before final approval of engineering or design changes. However, the system is not placed into service until such changes are documented and approved.

10c. Sec. 4.5.1

Requirement

"Installed systems and components shall be cleaned, flushed and conditioned according to the requirements of ANSI N45.2.1. Special consideration shall be given to the following requirements:" (Requirements are given for chemical conditioning, flushing and process controls.)

Exception/Interpretation

Systems and components are cleaned, flushed and conditioned as determined on a case-by-case basis. Measures are taken to help preclude the need for cleaning, flushing and conditioning through good practices during maintenance or modification activities.

11. N45.2.9

11a. Sec. 5.4, Item 2

Requirement

Records shall not be stored loosely. "They shall be firmly attached in binders or placed in folders or envelopes for storage on shelving in containers." Steel file cabinets are preferred.

Exception/Interpretation

Records are suitably stored in steel file cabinets, or on shelving in containers. Methods other than binders, folders, or envelopes (for example, dividers) may be used to organize the records for storage.

11b. Sec. 6.2

Requirement

"A list shall be maintained designating those personnel who shall have access to the files".

Exception/Interpretation

Rules are established governing access to and control of files as provided for in ANSI N45.2.9, Section 5.3, Item 5. These rules do not always include a requirement for a list of personnel who are authorized access. It should be noted that duplicate files and/or microforms may exist for general use.

11c. Sec. 5.6

Requirement

When a single records storage facility is maintained, at least the following features should be considered in its construction: etc.

Exception/Interpretation

The Cook Nuclear Plant Master File Room and other off-site record storage facilities comply with the requirements of NUREG-0800 (7/81), Section 17.1.17.4.

12. Reg. Guide 1.144/ANSI N45.2.12

12a. Sec. C3a(2)

Requirement

Applicable elements of an organization's Quality Assurance program for "design and construction phase activities should be audited at least annually or at least once within the life of the activity, whichever is shorter."

Exception/Interpretation

Since most modifications are straight forward, they are not audited individually. Instead, selected controls over modifications are audited periodically.

12b. Sec. C3b(1)

Requirement

This section identifies procurement contracts which are exempted from being audited.

Exception/Interpretation

In addition to the exemptions of Reg. Guide 1.144, I&M considers that the National Institute of Standards and Technology, or other State and Federal Agencies which may provide services to I&M, are not required to be audited.

12c. Sec. 3.3

Requirement

An effective audit system shall be established and maintained and shall include the following essential elements.....

3.3.7 Provision for verification of effective corrective action on a timely basis.

Exception/Interpretation

Verification of the implementation of effective corrective action is performed as indicated in Section 1.7.18.2.11 of this QAPD. Only selected corrective/preventive actions, determined by the auditing organization, will be verified by the auditing organization.

12d. 4.5.1

Requirement

...In the event that corrective action cannot be completed within thirty days, the audited organization's response shall include a scheduled date for the corrective action. The audited organization shall provide a follow-up report stating the corrective action taken and the date corrective action was completed.

Exception/Interpretation

The auditing organization will determine when it is necessary for the audited organization to provide a response within thirty days. If the auditing organization does not designate that the response must be completed within the thirty day time frame and forwarded to the auditing organization, the corrective action document will

be processed in accordance with the corrective action program. The program determines the safety significance, extent of the investigation required, investigation due date, and required level of management review and approval. The audited organization will provide follow-up documentation to the appropriate level of management as to the status of the corrective/preventive action. Documentation of follow-up will be provided to the auditing organization when specified by the auditing organization.

13. N45.2.13,

13a. Sec. 3.2.2

Requirement

N45.2.13 requires that technical requirements be specified in procurement documents by reference to technical requirement documents. Technical requirement documents are to be prepared, reviewed and released under the requirements established by ANSI N45.2.11.

Exception/Interpretation

For replacement parts and materials, I&M follow ANSI N18.7, Section 5.2.13, Subitem 1, which states: "Where the original item or part is found to be commercially 'off the shelf' or without specifically identified QA requirements, spare and replacement parts may be similarly procured, but care shall be exercised to ensure at least equivalent performance."

13b. Sec. 3.2.3

Requirement

"Procurement documents shall require that the supplier have a documented quality assurance program that implements parts or all of ANSI N45.2 as well as applicable quality assurance program requirements of other nationally recognized codes and standards."

Exception/Interpretation

Refer to Item 2j

13c. Sec. 3.3(a)

Requirement

Reviews of procurement documents shall be performed prior to release for bid and contract award.

Exception/Interpretation

Documents may be released for bid or contract award before completing the necessary reviews. However, these reviews are completed before the item or service is put into service, or before work has progressed beyond the point where it would be impractical to reverse the action taken.

13d. Sec. 3.3(b)

Requirement

Review of changes to procurement documents shall be performed prior to release for bid and contract award.

Exception/Interpretation

This requirement applies only to quality related changes (i.e., changes to the procurement document provisions identified in ANSI N18.7, Section 5.2.13.1, Subitems 1

through 5). The timing of reviews will be the same as for review of the original procurement documents.

13e. Sec. 10.1

Requirement

"Where required by code, regulation, or contract requirement, documentary evidence that items conform to procurement documents shall be available at the nuclear power plant site prior to installation or use of such items, regardless of acceptance methods."

Exception/Interpretation

Refer to Item 2k.

Requirement

"Post-installation test requirements and acceptance documentation shall be mutually established by the purchaser and supplier."

Exception/Interpretation

In exercising its ultimate responsibility for its quality assurance program, I&M establishes post-installation test requirements giving due consideration to supplier recommendations.

14. Reg. Guide 1.146/ANSI N45.2.23 and ANSI N45.2.2.12

14a. ANSI N45.2.23, Sec. 1.1

Requirement

This standard provides requirements and guidance for the qualification of audit team leaders, henceforth identified as "lead auditors."

14b. ANSI N45.2.12, Sec. 4.2.2

Requirement

A lead auditor shall be appointed team leader.

Exception/Interpretation

The [] audit program is directed by the performance assurance director and is administered by designated performance assurance [] managers/supervisors who are certified lead auditors.

Audits are, in most cases, conducted by individual auditors, not by "audit teams." These auditors are certified in accordance with established procedures and are assigned by the responsible performance assurance manager/supervisor based on their demonstrated audit capability and general knowledge of the audit subject. In certain cases, this results in an individual other than a "lead auditor" conducting the actual audit function.

Established I&M audit procedures require that, in all cases, the audit functions of preparation/organization, reporting of audit findings and evaluation of corrective actions be reviewed by a performance assurance manager/supervisor, thereby meeting the requirements of ANSI N45.2.23 relative to "lead auditors", and "audit team leaders."

15. ANSI N18.1

Section 4.2.2

Requirement

At the time of initial core loading or appointment to the active position the operations manager shall hold a senior reactor operator's license.

Exception/Interpretation

The requirement implies that only personnel which currently hold a senior reactor operator's license can be appointed as operations manager. I&M takes the position that the operations superintendent must hold or have held a senior operator license at Cook Nuclear Plant or a similar reactor; and one mid-level operations production supervisor shall hold a current senior operator license. This exception/interpretation is consistent with Technical Specification 6.2.2.h, previously approved by Nuclear Regulatory Commission.

ATTACHMENT TO AEP:NRC:80027A

APPENDIX C TO QAPD IN ACCORDANCE
WITH ADMINISTRATIVE LETTER 95-06

Appendix C

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6.5 REVIEW AND AUDIT

6.5.1 Plant Nuclear Safety Review Committee (PNSRC)

FUNCTION

6.5.1.1 The PNSRC shall function to advise the Site Vice President [REDACTED], or designee, on all matters related to nuclear safety.

COMPOSITION

6.5.1.2 The PNSRC shall be composed of a Plant Manager or his designee, Area Manager(s), and Department Superintendent(s) or [REDACTED] personnel reporting directly to an Area Manager or a Department Superintendent. The membership shall represent the functional areas of the plant, including, but not limited to Operations, Engineering, Licensing, Maintenance, and Radiation Protection.

The PNSRC membership shall consist of at least one individual from each of the areas designated. All members, including the Chairman and his alternates, the members and their alternates, shall be designated by the Site Vice President [REDACTED].



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PNSRC members and alternates shall meet or exceed the minimum qualifications of ANSI N18.1-1971 Section 4.4 for comparable positions. The nuclear power plant operations individual shall meet the qualifications of section 4.2.2 of ANSI N18.1-1971 except for the requirement to hold a current Senior Operator License. The operations individual must hold or have held a Senior Operator License at Cook Nuclear Plant or a similar reactor. The maintenance individual shall meet the qualifications of section 4.2.3 of ANSI N18.1-1971.

ALTERNATES

- 6.5.1.3 No more than two alternates shall participate as voting members in PNSRC activities at any one time.

MEETING FREQUENCY

- 6.5.1.4 The PNSRC shall meet at least once per calendar month and as convened by the PNSRC Chairman or his designated alternates.

QUORUM

- 6.5.1.5 The quorum of the PNSRC shall consist of the Chairman or his designated alternate and at least three members including alternates.

RESPONSIBILITIES

- 6.5.1.6 The PNSRC shall be responsible for:
- a. Review of all Plant Manager Instructions (PMIs) and revisions thereto.
 - b. Review of safety evaluations for (1) plant site procedures and revisions thereto which affect the nuclear safety of the plant; (2) changes or modifications to nuclear safety-related structures, systems or components; and (3) tests or experiments which affect plant nuclear safety to verify that such actions did not constitute an unreviewed safety question as defined in 10 CFR 50.59.
 - c. Review of (1) proposed procedures and revisions to procedures, (2) changes to equipment, systems, or facilities, and (3) proposed tests or experiments which may involve an unreviewed safety question as defined in 10 CFR 50.59.
 - d. Review of proposed changes to Appendix "A" Technical Specifications or the Operating License and rendering determinations in writing with regard to whether or not the proposed change constitutes a Significant Hazards Consideration [S].
 - e. Investigation of all violations of the Technical Specifications including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence to the Chairman of the NSDRC.
 - f. Review of all REPORTABLE EVENTS.



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- g. Review of facility operations to detect potential nuclear safety hazards.
- h. Performance of special reviews, investigations of analyses and reports thereon as requested by the Chairman of the NSDRC.
- i. Deleted
- j. Deleted
- k. Review of any accidental, unplanned, or uncontrolled radioactive release including the preparation of reports covering evaluations, recommendations, and disposition of the corrective action to prevent recurrence and the forwarding of these reports to the Executive Vice President - Nuclear Generation and to the NSDRC.
- l. Review of changes to the PROCESS CONTROL PROGRAM, OFFSITE DOSE CALCULATION MANUAL, and radwaste treatment system.

AUTHORITY

6.5.1.7 The PNSRC shall:

- a. Recommend to the Site Vice President [REDACTED], or designee, written approval or disapproval of items considered under 6.5.1.6 (a) through (d) above.
- b. Render determinations in writing with regard to whether or not each item considered under 6.5.1.6 (a) through (c) and (e) above constitutes an unreviewed safety question.
- c. Provide written notification within 24 hours to the Executive Vice President - Nuclear Generation and the NSDRC of disagreement between the PNSRC and the Site Vice President [REDACTED]; however, the Site Vice President shall have responsibility for resolution of such disagreements pursuant to 6.1.1 above.

RECORDS

6.5.1.8 The PNSRC shall maintain written minutes of each meeting and copies shall be provided to the Chairman of the NSDRC.

6.5.2 NUCLEAR SAFETY AND DESIGN REVIEW COMMITTEE (NSDRC)

FUNCTION

- 6.5.2.1 The NSDRC shall function to provide independent review and audit of designated activities in the areas of:
- a. nuclear power plant operations
 - b. nuclear engineering
 - c. chemistry and radiochemistry
 - d. metallurgy
 - e. instrumentation and control

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- f. radiological safety
- g. mechanical and electrical engineering
- h. quality assurance practices

COMPOSITION

6.5.2.2 The NSDRC shall be composed of the following regular members:

- 1. Executive Vice President- Nuclear generation (NSDRC Chairman)
- 2. Director - Regulatory Affairs (NSDRC Secretary)
- 3. Site Vice President [REDACTED] - Donald C. Cook Nuclear Plant
- 4. Plant Manager - Donald C. Cook Nuclear Plant
- 5. Chief Nuclear Engineer
- 6. Director - Business Performance
- 7. Director - Performance Assurance
- 8. Director - Plant Engineering
- 9. Manager - Performance Engineering and Analysis
- 10. Special Assistant - Nuclear Engineering
- 11. [REDACTED]
- 12. [REDACTED]
- 13. [REDACTED]

Additional members and Vice Chairman may be appointed by the Executive Vice President.

ALTERNATE MEMBERS

6.5.2.3 Designated alternate members shall be appointed by the Executive Vice President or such other person as he shall designate. In addition, temporary alternate members may be appointed by the NSDRC Chairman to serve on an interim basis, as required. Temporary alternate members are empowered to act on the behalf of the regular or designated alternate members for whom they substitute.

CONSULTANTS

6.5.2.4 Consultants shall be utilized as determined by the NSDRC Chairman to provide expert advice to the NSDRC.

MEETING FREQUENCY

6.5.2.5 The NSDRC shall meet at least once per six months.



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QUORUM

- 6.5.2.6 A quorum, the minimum number of regular members and alternates required to hold a NSDRC meeting shall be eight members, of whom no more than two shall be designated or temporary alternates. The Chairman or acting Chairman shall be present for all NSDRC meetings. If the number of members present* is greater than a quorum, then the majority participating and voting at the meeting shall not have line responsibility for operations of the facility. For the purpose of a quorum, only the Plant Manager is considered to have line responsibility.

REVIEW

- 6.5.2.7 The NSDRC is responsible for assuring that independent** reviews of the following are performed:
- a. The safety evaluations for 1) changes to procedures, equipment or systems and 2) tests or experiments completed under the provision of 10 CFR 50.59 to verify that such actions did not constitute an unreviewed safety question.
 - b. Proposed changes to procedures, equipment or systems which involve an unreviewed safety question as defined in 10 CFR 50.59.
 - c. Proposed tests or experiments which involve an unreviewed safety question as defined in 10 CFR 50.59.
 - d. Proposed changes in Technical Specifications or this operating license.
 - e. Violations of codes, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance.
 - f. Significant operating abnormalities or deviations from normal and expected performance of plant equipment that affect nuclear safety.
 - g. All REPORTABLE EVENTS.
 - h. All recognized indications of an unanticipated deficiency in some aspect of design or operation of safety-related structures, systems, or components.
 - i. Reports and meeting minutes of the PNSRC.

*Regular NSDRC members are expected to attend the meeting whenever possible, and alternates may attend as voting members only on an irregular basis. If both a regular member and his alternate attend a meeting, only the regular member may participate as a voting member, and the alternate is considered a guest.

**Independent reviews may be performed by groups which report directly to the NSDRC and which must have NSDRC membership participation.



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AUDITS

- 6.5.2.8 Audits of facility activities shall be performed under the cognizance of the NSDRC. These audits shall encompass:
- a. The conformance of facility operation to provisions contained within the Technical Specifications and applicable license conditions at least once per 12 months.
 - b. The performance, training, and qualifications of the entire facility staff at least once per 12 months.
 - c. The results of actions taken to correct deficiencies occurring in facility equipment, structures, systems or method of operation that affect nuclear safety at least once per 6 months.
 - d. The performance of activities required by the Quality Assurance Program to meet the criteria of Appendix "B", 10 CFR 50, at least once per 24 months.
 - e. Deleted.
 - f. Deleted.
 - g. The fire protection programmatic controls including the implementing procedures at least once per 24 months by qualified licensee QA personnel.
 - h. The fire protection equipment and program implementation at least once per 12 months using either a qualified offsite licensee fire protection engineer or an outside independent fire protection consultant. An outside independent fire protection consultant shall be used at least every third year.
 - i. The Radiological Environmental Monitoring Program and the results thereof at least once per 12 months.
 - j. The OFFSITE DOSE CALCULATION MANUAL and implementing procedures at least once per 24 months.
 - k. The PROCESS CONTROL PROGRAM and implementing procedures for solidification of radioactive wastes at least once per 24 months.
 - l. The performance of activities required by the Quality Assurance Program to meet the criteria of Regulatory Guide 1.21, Rev. 1, June 1974 and Regulatory, Guide 4.1. Rev. 1, April 1975 at least once per 12 months.
 - m. Any other area of facility operation considered appropriate by the NSDRC.

AUTHORITY

- 6.5.2.9 The NSDRC shall report to and advise the ~~Executive~~ Vice President Nuclear Generation on those areas of responsibility specified in Sections 6.5.2.7 and 6.5.2.8.

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RECORDS

6.5.2.10 Records of NSDRC activities shall be prepared, approved and distributed as indicated below:

- a. Minutes of each NSDRC meeting shall be prepared, approved and issued within 14 days following each meeting.
- b. Reports of reviews encompassed by Section 6.5.2.7 above, shall be prepared, approved and issued within 14 days following completion of the review.
- c. Audit reports encompassed by Section 6.5.2.8 above, shall be forwarded to the Executive Vice President Nuclear Generation and to the management positions responsible for the areas audited within 30 days after completion of the audit.

6.5.3 TECHNICAL REVIEW AND CONTROL

6.5.3.1 Activities which affect nuclear safety shall be conducted as follows:

- a. Procedures required by Specification 6.8 and other procedures which affect plant nuclear safety, and changes thereto, shall be prepared, reviewed and approved. Each such procedure or procedure change shall be reviewed by a qualified individual/group other than the individual/group which prepared the procedure or procedure change, but who may be from the same organization as the individual/group which prepared the procedure or procedure change. Procedures other than Plant Manager Procedures shall be approved by the appropriate department head as previously designated in writing by the Site Vice President, or designee. The Site Vice President, or designee, shall approve Plant Manager Procedures. Temporary changes to procedures which do not change the intent of the approved procedures shall be approved for implementation by two members of the plant staff, at least one of whom holds a Senior Operator license, and documented. The temporary changes shall be approved by the original approval authority within 14 days of implementation. For changes to procedures which may involve a change in intent of the approved procedures, the person authorized above to approve the procedure shall approve the change prior to implementation.
- b. Proposed changes or modifications to plant nuclear safety-related structures, systems and components shall be reviewed as designated by the Site Vice President, or designee. Each such modification shall be reviewed (reference T/S 6.5.3.1.e) by a qualified (reference T/S 6.5.3.1.d) individual/group other than the individual/group which designed the modification, but who may be from the same organization as the individual/group which designed the modifications. Proposed modifications to plant nuclear safety-related structures, systems and components shall be approved prior to implementation by the Site Vice President, or designee.



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- c. Proposed tests and experiments which affect plant nuclear safety and are not addressed in the Final Safety Analysis Report or Technical Specifications shall be prepared, reviewed, and approved. Each such test or experiment shall be reviewed by qualified individuals/groups other than the individual/group which prepared the proposed test or experiment to assure cross disciplinary review as appropriate for the proposed test or experiment. Proposed tests and experiments shall be approved before implementation by the Site Vice President [REDACTED], or designee.
- d. Individuals who conducted the reviews performed in the accordance with Specification 6.5.3.1a, 6.5.3.1b, and 6.5.3.1c, shall be members of the plant management staff previously designated by the Site Vice President [REDACTED] and shall meet or exceed the minimum qualifications of ANSI N18.1-1971 Section 4.4 for comparable positions. Each such review shall include a determination of whether or not additional, cross-disciplinary review is necessary.

If deemed necessary, such review shall be performed by qualified personnel of the appropriate discipline.

- e. Each review shall include a determination of whether or not an unreviewed safety question is involved. Pursuant to 10 CFR 50.59, NRC approval of items involving unreviewed safety questions shall be obtained prior to the approval of the Site Vice President [REDACTED], or designee, for implementation.

f. Deleted.

g. Deleted.

- 6.5.3.2 Records of the above activities shall be provided to the Site Vice President [REDACTED] or designee, PNSRC and/or the NSDRC as necessary for required reviews.

6. 6 REPORTABLE EVENT ACTION

6.6.1 The following actions shall be taken for REPORTABLE EVENTS:

- a. (retained in technical specifications)
- b. Each REPORTABLE EVENT shall be reviewed by the PNSRC, and the results of this review shall be submitted to the NSDRC and the Executive Vice President - Nuclear Generation.

6.7 SAFETY LIMIT VIOLATION

6.7.1 The following actions shall be taken in the event a safety limit is violated:

- a. The NRC Operations Center shall be notified by telephone as soon as possible and in all cases within 1 hour. Within 24 hours notify the Executive Vice President - Nuclear Generation.

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- b. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by PNSRC. This report shall describe (1) applicable circumstances preceding the violation; (2) effects of the violation upon facility components, systems or structures; and (3) corrective action taken to prevent recurrence.
- c. Within 14 days of the violation, the Safety Limit Violation Report shall be submitted to the Commission, and to the Executive Vice President - Nuclear Generation.
- d. (retained in technical specifications)

6.8 PROCEDURES AND PROGRAMS

6.8.2 Each procedure and administrative policy of Technical Specification 6.8.1, and changes thereto, including temporary changes, shall be reviewed prior to implementation as set forth in Specification 6.5 above.

6.10 RECORD RETENTION

6.10.1 The following records shall be retained for at least five years:

- a. Records and logs of unit operation covering time interval at each power level.
- b. Records and logs of principal maintenance activities, inspections, repair and replacement of principal items of equipment related to nuclear safety.
- c. All REPORTABLE EVENTS submitted to the Commission.
- d. Records of surveillance activities, inspections and calibrations required by the Technical Specifications.
- e. Records of changes made to the procedures required by Technical Specification 6.8.1.
- f. Records of sealed source and fission detection leak tests and results.
- g. Records of annual physical inventory of all sealed source material on record.

6.10.2 The following records shall be retained for the duration of the Facility Operating License:

- a. Records and drawing changes reflecting unit design modifications made to systems and equipment described in the Final Safety Analysis Report.
- b. Records of new and irradiated fuel inventory, fuel transfers and assembly burnup histories.
- c. Records of radiation exposure for all individuals entering radiation control areas.
- d. Records of gaseous and liquid radioactive material released to the environment.

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- e. Records of transient or operational cycles for those facility components identified in the Updated Final Safety Analysis Report.
- f. Records of reactor tests and experiments.
- g. Records of training and qualification for current members of the Plant Staff.
- h. Records of in-service inspections performed pursuant to these Technical Specifications.
- i. Records of Quality Assurance activities required by the QA Manual.
- j. Records of reviews performed for changes made to procedures or equipment or review of tests and experiments pursuant to 10 CFR 50.59.
- k. Records of meetings of the PNSRC and the NSDRC.
- l. Records of radioactive shipments.
- m. Records of the service lives of hydraulic snubbers including the date at which service life commences and associated installation and maintenance records.
- n. Records of reviews performed for changes made to the OFFSITE DOSE CALCULATION MANUAL and the PROCESS CONTROL PROGRAM.

6.13 PROCESS CONTROL PROGRAM (PCP)

6.13.1 Changes to the PCP:

- a. Shall be documented and records of reviews performed shall be retained as required by Specification 6.10.2.n. This documentation shall contain:
 - 1. (retained in technical specifications)
 - 2. (retained in technical specifications)
- b. Shall become effective after review and acceptance by the PNSRC and approval of the Plant Manager.

6.14 OFFSITE DOSE CALCULATION MANUAL (ODCM)

6.14.1 Changes to the ODCM:

- a. Shall be documented and record of reviews performed shall be retained as required by Specification 6.10.2.n. This documentation shall contain:
 - 1. (retained in technical specifications)
 - 2. (retained in technical specifications)
- b. Shall become effective after review and acceptance by the PNSRC and the approval of the Plant Manager.
- c. (retained in technical specifications)

