

# CATEGORY 1

REGULATOR INFORMATION DISTRIBUTION SYSTEM (RIDS)

X<sup>2</sup> ACCESSION NBR: 9703120048    DOC. DATE: 97/03/07    NOTARIZED: NO    DOCKET #  
 FACIL: 50-315 Donald C. Cook Nuclear Power Plant, Unit 1, Indiana M    05000315  
 AUTH. NAME    AUTHOR AFFILIATION  
 GILLESPIE, R.    Indiana Michigan Power Co.  
 BLIND, A.A.    Indiana Michigan Power Co.  
 RECIP. NAME    RECIPIENT AFFILIATION

SUBJECT: LER 97-003-00: on 970205, interim LER-condition outside  
 design basis occurred due to potential safety injection  
 pump runout. Update to interim LER will be submitted by  
 970331.W/970307 ltr.

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NOTES:

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	NRC PDR	1 1	NUDOCS FULL TXT	1 1

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March 7, 1997

United States Nuclear Regulatory Commission  
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Rockville, Maryland 20852

Operating Licenses DPR-58  
Docket No. 50-315

Document Control Manager:

In accordance with the criteria established by 10 CFR 50.73 entitled Licensee Event Report System, the following report is being submitted:

97-003-00

Sincerely,

A. A. Blind  
Site Vice President

/mbd

Attachment

c: A. B. Beach, Region III  
E. E. Fitzpatrick  
P. A. Barrett  
S. J. Brewer  
J. R. Padgett  
D. Hahn  
Records Center, INPO  
NRC Resident Inspector

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PDR ADOCK 05000315  
S PDR





## LICENSEE EVENT REPORT (LER)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)  
Donald C. Cook Nuclear Plant - Unit 1DOCKET NUMBER (2)  
50-315

Page 1 of 1

## TITLE (4)

Interim LER -Condition Outside the Design Basis Due to Potential Safety Injection Pump Runout

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
02	05	97	97	- 003 -	00	03	07	97	Cook Unit 2	50-316
									FACILITY NAME	DOCKET NUMBER
OPERATING MODE (9)			THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 50.73(a)(2)(iii) (Check one or more) (11)							
1			20.2201(b)			20.2203(a)(3)(i)			50.73(a)(2)(iii)	73.71(b)
POWER LEVEL (10)			20.2203(a)(1)			20.2203(a)(3)(ii)			50.73(a)(2)(iv)	73.71
96.7			20.2203(a)(2)(i)			20.2203(a)(4)			50.73(a)(2)(v)	OTHER
			20.2203(a)(2)(ii)			50.36(c)(1)			50.73(a)(2)(vii)	(Specify in Abstract below and in Text, NRC Form 366A)
			20.2203(a)(2)(iii)			50.36(c)(2)			50.73(a)(2)(viii)(A)	
			20.2203(a)(2)(iv)			50.73(a)(2)(I)			50.73(a)(2)(viii)(B)	
			20.2203(a)(2)(v)			X 50.73(a)(2)(ii)			50.73(a)(2)(x)	

## LICENSEE CONTACT FOR THIS LER (12)

NAME  
Robert Gillespie, Operations SuperintendentTELEPHONE NUMBER (Include Area Code)  
616/465-5901, x2325

## COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDs	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDs

## SUPPLEMENTAL REPORT EXPECTED (14)

X YES	NO	EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
			03	31	97

## ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16)

On February 5, 1997, with Unit 1 at 96.7 percent Rated Thermal Power and Unit 2 at 99.8 percent Rated Thermal Power, preliminary information was received from Westinghouse that indicated that the Safety Injection pumps could potentially experience runout conditions if the accumulator fill line were open during a large break LOCA demand. An ENS notification was made under 10CFR50.72(b)(1)(ii)(B), to report a condition outside the design basis at that time.

Cook Technical Specification 4.5.2.h states that total Safety Injection system flow shall not exceed 700 gpm with a single pump running and the minimum flow valve open. If only one safety Injection pump is providing flow to all four injection lines while the accumulator fill line is fully open and the minimum flow valve is open, which may occur during the injection phase of a large break LOCA towards the end of blowdown, the runout limit may be exceeded. By letter dated February 21, 1997, Westinghouse has confirmed, by model, that the pumps will run out to between 708 and 720 gpm, dependent on initial conditions.

The impact on the LOCA analysis is currently being analyzed by Westinghouse, and final results are expected by mid-March. Preliminary feedback indicates that the impact will be minimal and of little safety consequence.

An update to this interim LER will be submitted by March 31, 1997, which will incorporate the final results from Westinghouse.

