

CATEGORY 1

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR: 9609250109 DOC. DATE: 96/09/19 NOTARIZED: NO DOCKET #
 FACI: 50-315 Donald C. Cook Nuclear Power Plant, Unit 1, Indiana M 05000315
 AUTH. NAME AUTHOR AFFILIATION
 FITZPATRICK, E. Indiana Michigan Power Co. (formerly Indiana & Michigan Ele
 RECIP. NAME RECIPIENT AFFILIATION
 Document Control Branch (Document Control Desk)

SUBJECT: Provides clarification to several questions raised by staff
 re application for amend to TS for incorporating 2.0 volt SG
 tube support plate plugging criterion.

DISTRIBUTION CODE: A001D COPIES RECEIVED: LTR 1 ENCL 0 SIZE: 2
 TITLE: OR Submittal: General Distribution

NOTES:

	RECIPIENT ID CODE/NAME	COPIES LTTR ENCL	RECIPIENT ID CODE/NAME	COPIES LTTR ENCL
	PD3-1 LA	1 1	PD3-1 PD	1 1
	HICKMAN, J.	1 1		
INTERNAL:	<u>FILE CENTER</u> 01	1 1	NRR/DE/EMCB	1 1
	NRR/DRCH/HICB	1 1	NRR/DSSA/SPLB	1 1
	NRR/DSSA/SRXB	1 1	NUDOCS-ABSTRACT	1 1
	OGC/HDS2	1 0		
EXTERNAL:	NOAC	1 1	NRC PDR	1 1

NOTE TO ALL "RIDS" RECIPIENTS:
 PLEASE HELP US TO REDUCE WASTE! CONTACT THE DOCUMENT CONTROL DESK,
 ROOM OWFN 5D-5 (EXT. 415-2083) TO ELIMINATE YOUR NAME FROM
 DISTRIBUTION LISTS FOR DOCUMENTS YOU DON'T NEED!

TOTAL NUMBER OF COPIES REQUIRED: LTTR 12 ENCL 0

C
A
T
E
G
O
R
Y

1

D
O
C
U
M
E
N
T



INDIANA
MICHIGAN
POWER

September 19, 1996

AEP:NRC:1166AE

Docket No.: 50-315

U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, D. C. 20555

Gentlemen:

Donald C. Cook Nuclear Plant Unit 1
TECHNICAL SPECIFICATION CHANGES TO INCORPORATE
2.0 VOLT STEAM GENERATOR TUBE SUPPORT PLATE
PLUGGING CRITERION REQUEST FOR ADDITIONAL INFORMATION

This letter provides clarification to several questions raised by your staff regarding our application for amendment to the technical specifications (T/Ss) of Donald C. Cook Nuclear Plant Unit 1 for incorporating the 2.0 volt steam generator tube support plate plugging criterion.

As discussed in our telephone conference call of September 6, 1996, we concur that the structural voltage limit should be 8.6 volts in lieu of the 8.8 volts referenced in our submittal AEP:NRC:1166AA. When using the 8.6 volts as the structural voltage it now yields an end-of-cycle safety margin of 5.4 volts and a maximum beginning-of-cycle (BOC) voltage repair limit of 5.375 volts.

The method of dealing with probe wear was also discussed during the call. An alternative method to the probe wear guidance contained in generic letter 95-05 will be used. The alternative method will follow the NRC endorsed NEI plan as outlined in NEI letters dated January 23, 1996, and February 23, 1996.

On September 9, 1996, you requested our comments regarding three NRC perceptions. Our comments follow directly after each NRC stated perception:

You will not be applying a fraction of bobbin coil indications in the BOC voltage distribution resolved during the inspection process as NDD with RPC probe. Is that a correct statement? The statement is correct.

You will use the probabilistic analytical methods described in WCAP-14277, not any deterministic methods also described in the same report, for determining leak rate and tube burst probabilities. Correct? The statement is correct.

9609250109 960919
PDR ADDCK 05000315
PDR

250033

A001 1/2

The proposed TS 4.4.5.4.10.c is slightly different from GL 95-05 model, "acceptable alternative inspection techniques" that are mentioned. Is it correct that they will be qualified using similar criteria as used for qualifying RPC probes? The statement is correct.

Sincerely,



E. E. Fitzpatrick
Vice President

/jmb

cc: A. A. Blind
A. W. Beach
NFEM Section Chief
NRC Resident Inspector
J. R. Padgett