

PRIORITY 10

(ACCELERATED RIDS PROCESSING)

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR: 9511270104 DOC. DATE: 95/11/20 NOTARIZED: NO DOCKET #
 FACIL: 50-315 Donald C. Cook Nuclear Power Plant, Unit 1, Indiana M 05000315
 AUTH. NAME AUTHOR AFFILIATION
 ALLARD, J. Indiana Michigan Power Co. (formerly Indiana & Michigan Ele
 BLIND, A.A. Indiana Michigan Power Co. (formerly Indiana & Michigan Ele
 RECIP. NAME RECIPIENT AFFILIATION

SUBJECT: LER 95-010-00: on 951020, inadequate communication resulted in unexpected ESF actuation. Completed repairs to 1NRI-36 & declared detector operable. W/951120 ltr.

DISTRIBUTION CODE: IE22T COPIES RECEIVED: LTR 1 ENCL 1 SIZE: 4
 TITLE: 50.73/50.9 Licensee Event Report (LER), Incident Rpt, etc.

NOTES:

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Indiana Michigan
Power Company
Cook Nuclear Plant
One Cook Place
Bridgman, MI 49106
616 465 5901



November 20, 1995

United States Nuclear Regulatory Commission
Document Control Desk
Rockville, Maryland 20852

Operating Licenses DPR-58
Docket No. 50-315

Document Control Manager:

In accordance with the criteria established by
10 CFR 50.73 entitled Licensee Event Report System, the following report is being
submitted:

95-010-00

Sincerely,

A handwritten signature in cursive script, reading "A. A. Blind", is positioned above the printed name.

A. A. Blind
Plant Manager

/s/c

Attachment

c: H. J. Miller, Region III
E. E. Fitzpatrick
P. A. Barrett
R. F. Kroeger
M. A. Bailey - Ft. Wayne
S. J. Brewer
J. R. Padgett
G. Charnoff, Esq.
D. Hahn
Records Center, INPO
NRC Resident Inspector

9511270104 951120
PDR ADCK 05000315
S PDR

LICENSEE EVENT REPORT (LER)

(See reverse for required number of digits/characters for each block)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MN8B 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)

Donald C. Cook Nuclear Plant - Unit 1

DOCKET NUMBER (2)

05000 315

PAGE (3)

1 OF 3

TITLE (4)

Inadequate Communication Results in Unexpected ESF Actuation

EVENT DATE (5)

LER NUMBER (6)

REPORT NUMBER (7)

OTHER FACILITIES INVOLVED (8)

MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
10	20	95	95	010	00	11	20	95	FACILITY NAME	DOCKET NUMBER
										05000
										05000

OPERATING MODE (9)	POWER LEVEL (10)	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5: (Check one or more) (11)			
3	0	20.402(b)	20.405(a)(1)(i)	20.405(a)(1)(ii)	20.405(a)(1)(iii)
		20.405(a)(1)(iv)	20.405(a)(1)(v)	20.405(a)(2)(i)	20.405(a)(2)(ii)
		20.405(a)(2)(iii)	20.405(a)(2)(iv)	20.405(a)(2)(v)	20.405(a)(2)(vi)
		20.405(a)(2)(vii)	20.405(a)(2)(viii)	20.405(a)(2)(ix)	20.405(a)(2)(x)

LICENSEE CONTACT FOR THIS LER (12)

NAME

John Allard, Maintenance Supervisor

TELEPHONE NUMBER (Include Area Code)

616/465-5901, x2522

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS

SUPPLEMENTAL REPORT EXPECTED (14)

YES

(If yes, complete EXPECTED SUBMISSION DATE)

X

NO

EXPECTED
SUBMISSION
DATE (15)

MONTH

DAY

YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16)

On October 20, 1995, with Unit 1 in Mode 3, repair activities were underway on Intermediate Range Neutron Flux Detector 1-NRI-36. When Maintenance Instrumentation and Control (I&C) personnel pulled the control power fuses for this detector, an ESF signal was generated. This signal, which resulted in a Reactor Protection System trip, was not expected by Control Room personnel. The reactor trip breakers opened but no rod movement occurred as the rods were already fully inserted. No other equipment changed status.

This event was reported via the ENS system at 1436 hours on October 20, 1995 in accordance with 10CFR50.72(b)(2)(ii).

The unexpected ESF actuation resulted from a lack of effective communication between I&C personnel and Operations Control Room staff. Corrective actions included a "stop work", a "Communicating with Operations Time Out" and detailed written guidance from supervision on proper methods of communicating with Operations personnel.

This event was determined to be of no safety significance. The event was not indicative of a degradation in the level of safety of the plant or a deterioration of plant conditions and occurred during a maintenance evolution. No control rod movement was seen as the control rods were all fully inserted. At no time was the health or safety of the public in jeopardy.

LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-630), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1) Donald C. Cook Nuclear Plant - Unit 1	DOCKET NUMBER (2) 0 5 0 0 0 3 1 5	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		9 5 —	0 1 0 —	0 0	0 2	OF	0 3

TEXT (If more space is required, use additional NRC Form 366A's) (17)

Conditions Prior to Event

Unit 1 was in Mode 3, Hot Standby

Description of Event

On the morning of October 19, 1995, Operations personnel observed Nuclear Instrumentation Channel II Intermediate Range Neutron Flux Detector 1-NRI-36 (EIS/DET), reading low. A corrective maintenance work request was initiated. The Log Current Amplifier of the detector was found out of calibration by Maintenance Instrumentation and Control (I&C) and was recalibrated and 1-NRI-36 was declared operable.

On the morning of October 20, 1995, the instrumentation was again found reading low. The work order of the previous day was reopened and a pre-job briefing was held. The work to be performed was discussed with Operations, however, the job was not immediately started. At the request of Operations the work was delayed until Rod Drop testing, which was in progress, could be completed.

Following completion of the Rod Drop testing the Reactor Trip breakers were reclosed with the rods fully inserted. I&C personnel returned to the control room and briefed Operations regarding the work to be done on 1-NRI-36. The Shift and Unit Supervisors then authorized the start of work.

I&C personnel began work on the detector drawer and removed the instrument power and control power fuses. When the control power was removed an ESF reactor trip signal was generated, opening the reactor trip breakers and causing a reactor trip annunciation. This ESF actuation was not expected by the control room personnel.

Although the reactor trip breakers opened, no control rod movement occurred as the rods were already inserted fully. No other equipment changed status. The reactor trip breakers were left open and discussion of the reactor trip took place between the I&C Maintenance Supervisor and control room personnel.

LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPLY MANAGEMENT BRANCH (PS-30), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
Donald C. Cook Nuclear Plant - Unit 1	0 5 0 0 0 3 1 5	9 5	— 0 1 0	— 0 0	0 3	OF	0 3

TEXT (If more space is required, use additional NRC Form 366A's) (17)

Cause of Event

This event resulted from a lack of effective communication between I&C Maintenance personnel and Operations control room staff. Assumptions were made by both organizations concerning the work to be done, but neither group verbally verified what specific tasks would take place.

Analysis of Event

This event was determined to be immediately reportable under 10CFR50.72(b)(2)(ii) as an event that resulted in the automatic actuation of any Engineered Safety Feature. A red phone call for this event was made at 1436 hours on October 20, 1995. This event is also reportable under 10CFR50.73(a)(2)(iv) as an ESF actuation which resulted in a Reactor Protection System trip.

This event was determined to be of no safety significance. The event was not indicative of a degradation in the level of safety of the plant or a deterioration of plant conditions and occurred during a maintenance evolution. No control rod movement was seen as the control rods were all fully inserted. At no time was the health or safety of the public in jeopardy.

Corrective Actions

Repairs to 1-NRI-36 were completed and the detector was declared operable.

Corrective actions included a "stop work" order and a "Communicating with Operations Time Out" where Maintenance personnel discussed the event and the generic aspects of communicating properly with Operations personnel. Additional written direction was provided by the Maintenance and Plant Modifications and Installation Services Superintendents to their personnel regarding communications with Operations.

In recognition of a negative trend in human performance, policy has been established regarding communications between work groups and Operations. To aid in improving communications throughout the plant, guidance documents for assuring good communications practices are also being developed.

Previous Similar Events

315/95-005-00
316/94-010-00
315/90-012-00