

PRIORITY 1

(ACCELERATED RIDS PROCESSING)

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR: 9511030200 DOC. DATE: 95/10/31 NOTARIZED: NO DOCKET #
FACIL: 50-315 Donald C. Cook Nuclear Power Plant, Unit 1, Indiana M 05000315 P
AUTH. NAME AUTHOR AFFILIATION
WEBER, G.A. Indiana Michigan Power Co. (formerly Indiana & Michigan Ele R
BLIND, A.A. Indiana Michigan Power Co. (formerly Indiana & Michigan Ele
RECIP. NAME RECIPIENT AFFILIATION I

SUBJECT: LER 95-006-01: on 950825, containment Type B & C exceeded LCO
value due to leakage of post accident sample line check O
valve & ice condenser Glycol Header isolation valve pressure R
relief check valve. Evaluated event. W/951031 ltr.

DISTRIBUTION CODE: IE22T COPIES RECEIVED: LTR 1 ENCL 1 SIZE: 5
TITLE: 50.73/50.9 Licensee Event Report (LER), Incident Rpt, etc. I

NOTES: T

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October 31, 1995

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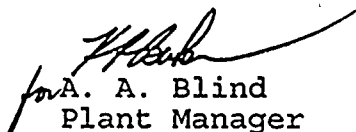
Operating Licenses DPR-58
Docket No. 50-315

Document Control Manager:

In accordance with the criteria established by
10 CFR 50.73 entitled Licensee Event Report System, the
following report is being submitted:

95-006-01

Sincerely,


for A. A. Blind
Plant Manager

/clc

Attachment

c: H. J. Miller, Region III
E. E. Fitzpatrick
P. A. Barrett
R. F. Kroeger
M. A. Bailey - Ft. Wayne
S. J. Brewer
J. R. Padgett
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Records Center, INPO
NRC Resident Inspector

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LICENSEE EVENT REPORT (LER)

(See reverse for required number of digits/characters for each block)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNBB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)

Donald C. Cook Nuclear Plant - Unit 1

DOCKET NUMBER (2)

05000 315

PAGE (3)

1 OF 4

TITLE (4) Containment Type B & C Leakrate Exceeds LCO Value Due to Leakage of Post Accident Sample Line Check Valve & Ice Condenser Glycol Header Isolation Valve Pressure Relief Check Valve.

EVENT DATE (5)			LER NUMBER (6)			REPORT NUMBER (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
08	25	95	95	-- 006 --	01	10	31	95	FACILITY NAME	DOCKET NUMBER
OPERATING MODE (9) 6			THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5: (Check one or more) (11)							
POWER LEVEL (10) 00			20.402(b)		20.405(c)		50.73(a)(2)(iv)		73.71(b)	
			20.405(a)(1)(i)		50.36(c)(1)		50.73(a)(2)(v)		73.71(c)	
			20.405(a)(1)(ii)		50.36(c)(2)		50.73(a)(2)(vii)		OTHER	
			20.405(a)(1)(iii)		X 50.73(a)(2)(i)		50.73(a)(2)(viii)(A)		(Specify in Abstract below and in Text, NRC Form 366A)	
			20.405(a)(1)(iv)		50.73(a)(2)(ii)		50.73(a)(2)(viii)(B)			
			20.405(a)(1)(v)		50.73(a)(2)(iii)		50.73(a)(2)(x)			

LICENSEE CONTACT FOR THIS LER (12)

NAME

G.A. Weber -- Plant Engineering Superintendent

TELEPHONE NUMBER (include Area Code)

616-465-5901 x2511

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRCDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRCDS
B	BD	ISV	C 631	Y					
B	BD	ISV	K 085	Y					

SUPPLEMENTAL REPORT EXPECTED (14)

YES

(if yes, complete EXPECTED SUBMISSION DATE)

X

NO

EXPECTED SUBMISSION DATE (15)

MONTH DAY YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16)

On August 25, 1995 with Unit 1 in Mode 6 and defueled, the accumulated leakage of the Type B and C leak rate testing was determined to be 0.89 L_s. This exceeded the Technical Specification LCO value of 0.60 L_s. On September 14, 1995 an interim LER, 315/95-006-00, was submitted. This supplemental report, LER 315/95-006-01, is being submitted to provide the required additional information regarding this event.

Valves 1-R-157, the Ice Condenser Refrigeration Glycol Return Header Containment Isolation Valves Pressure Relief Header Check Valve, and 1-NS-357, the Post Accident Sampling Station Sample Waste Return To Unit 1 Containment Check Valve, were found to exceed their leakage limits. The cause of the excessive leakage for 1-R-157 could not be determined. The valve had failed two previous tests, and the decision was made to replace the valve after this failure. The cause of the excessive leakage for 1-NS-357 was determined to have been caused by debris on the valve seating surface. The valve was cleaned and left with an acceptable leakage rate.

This event was evaluated and determined to have minimal safety significance. The health and safety of the public was not affected.

LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1) Donald C. Cook Nuclear Plant - Unit 1	DOCKET NUMBER (2) 0 5 0 0 0 3 1 5	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		9 5	0 0 6	0 1	0 2	OF	0 4

TEXT (If more space is required, use additional NRC Form 366A's) (17)

Conditions Prior to Occurrence

Unit One - Mode 6, Refueling

Description of Event

On August 25, 1995, the accumulated Leakage of the Type B and C leak rate testing of the containment penetrations and isolation valves exceeded the Technical Specification LCO value of 0.6 L_a, which is equivalent to 66,000 SCCM. The total as-found leak rate was found to be 0.89 L_a.

Valve 1-R-157, the Ice Condenser Refrigeration Glycol Return Header Containment Isolation Valves Pressure Relief Header Check Valve (EIS:ISV/BD), had a measured leak rate of 46,500 SCCM and valve 1-NS-357, the Post Accident Sampling Station Sample Waste Return To Unit 1 Containment Check Valve (EIS:ISV/BD), had a measured leakrate of 41,500 SCCM. The leakage from these two valves totalled 88,000 SCCM and accounted for 90 percent of the total measured Type B and C Leak Rate.

NOTE: The leakage values initially submitted in LER 315/95-006-00 were incorrect for both valves.

Cause of Event

The failure of 1-NS-357 is attributed to debris on the valve seat. The debris appeared to be the consistency of metal filings. The source of the metal filings was most likely from the removal and replacement of this valve on June 20, 1995. When this valve was replaced a cleanliness inspection of the line was performed. The cleanliness inspection was not effective in ensuring that the line was free of debris prior to installing the new valve. The debris most likely entered the line during the cutting and grinding activities during the valve replacement and settled in the 1/2 inch line beyond the portion of line that was observable during the cleanliness inspection.

The cause for the 1-R-157 failure could not be determined. The valve was removed and inspected for damage. No degraded parts or abnormal condition were found. A review of the previous test history revealed that 1-R-157 had failed its two previous tests. Based on the previous test history, the valve has been replaced.

LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 500 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1) Donald C. Cook Nuclear Plant - Unit 1	DOCKET NUMBER (2) 0 5 0 0 0 3 1 5	LER NUMBER (6)			PAGE (3)		
		YEAR 9 5	SEQUENTIAL NUMBER — 0 0 6	REVISION NUMBER — 0 1			

TEXT (If more space is required, use additional NRC Form 366A's) (17)

Analysis of Event

This event is reportable in accordance with 10CFR50.73(a)(2)(i)(B) as operation prohibited by Plant Technical Specification 3.6.1.2. Technical Specification 3.6.1.2 limits the Type B and C Leak Rate to 0.6 L_s. The Technical Specification Limit was exceeded on August 25, 1995. A four hour phone report was made to the NRC at 1642 hours on August 25, 1995, as required by 10CFR50.72(b)(2).

The as-found leakage of 1-NS-357 was 41,500 SCCM. This valve is a post accident sample return line which is classified as a Class D line. It is designed with double isolation valves. The Type C test results of the second connected series isolation valves was found to be 85 SCCM. As a result, the isolation of this line was available by the secondary isolation valves and the failure of 1-NS-357 alone would not have jeopardized containment integrity.

The as-found leakage of 1-R-157 was 46,500 SCCM. This valve provides a relief path back to containment for glycol that is trapped between the glycol header containment isolation valves. This line is classified as a Class A line with double isolation valves. The Type C test results of the second isolation connected series isolation valve was found to be 00 SCCM. As a result, the isolation of this line was available by the secondary isolation valve and the failure of 1-R-157 alone would not have jeopardized containment integrity.

In conclusion, the as-found condition of the Containment Isolation Valves did not put the plant in an unsafe condition. This event was determined to be of minimal safety significance as the health and safety of the public was not in jeopardy.

Corrective Action

Valve 1-NS-357 was disassembled and inspected. The valve was cleaned and the line was flushed to ensure all debris had been removed. After reassembly, a retest indicated an acceptable as-left leakrate of 48 SCCM. A review of this event and the controls for the cleanliness inspections of small bore pipe will be reviewed to determine if program enhancements are necessary.

Valve 1-R-157 was removed, inspected, and reassembled. A retest indicated an acceptable leakrate of 0 SCCM, however, as a preventive action, 1-R-157 was replaced with a poppet check valve. This is a slightly different design than the lift check valve previously installed. This action was based on the previous history of the valve and lack of qualified replacement parts.

The as-left Type B and C Leakrate was 0.087 L_s, well below the 0.6 L_s Technical Specification Limit.

LICENSEE EVENT REPORT (LER)
TEXT CONTINUATIONESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS
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FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
Donald C. Cook Nuclear Plant - Unit 1	0 5 0 0 0 3 1 5	9 5	— 0 0 6	— 0 1	0 4	OF	0 4

TEXT (If more space is required, use additional NRC Form 366A's) (17)

Failed Component Identification

Component Name: Post Accident Sampling Station Sample Waste Return Check Valve (1-NS-357)
Manufacturer: Conval
Model: 50-12C2-S163D
EIS Code: ISV/BD

Component Name: Ice Condenser Refrigeration Glycol Return Header Containment Isolation Valves
Pressure Relief Header Check Valve(1-R-157)
Manufacturer: Kerotest
Model: 9645
EIS Code: ISV/BD

Previous Similar Events

315/79-34	316/79-20
315/81-11	316/79-53
315/81-25	316/81-18
315/82-58	316/83-16
315/83-72	316/84-05
315/85-17	316/86-09
315/87-12	316/89-05
315/89-04	316/90-07
315/92-07	316/94-07