

PRIORITY 1

(ACCELERATED RIDS PROCESSING)

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR:9510130390 DOC.DATE: 95/10/10 NOTARIZED: NO DOCKET #
FACIL:50-315 Donald C. Cook Nuclear Power Plant, Unit 1, Indiana M 05000315
AUTH.NAME AUTHOR AFFILIATION
NICHOLS,W. Indiana Michigan Power Co. (formerly Indiana & Michigan Ele
BLIND,A.A. Indiana Michigan Power Co. (formerly Indiana & Michigan Ele
RECIP.NAME RECIPIENT AFFILIATION

SUBJECT: LER 95-004-00:on 950704,TS 3.0.3 was entered on loss of
injection capability due to personnel error during
performance of surveillance.Restored RHR valves.W/951010
ltr.

DISTRIBUTION CODE: IE22T COPIES RECEIVED:LTR 1 ENCL 1 SIZE: 4
TITLE: 50.73/50.9 Licensee Event Report (LER), Incident Rpt, etc.

NOTES:

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EXTERNAL: L ST LOBBY WARD	1 1	LITCO BRYCE,J H	2 2
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NOTE TO ALL "RIDS" RECIPIENTS:

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Indiana Michigan
Power Company
Cook Nuclear Plant
One Cook Place
Bridgman, MI 49106
616 465 5901



October 10, 1995

United States Nuclear Regulatory Commission
Document Control Desk
Rockville, Maryland 20852

Operating Licenses DPR-58
Docket No. 50-315

Document Control Manager:

In accordance with the criteria established by
10 CFR 50.73 entitled Licensee Event Report System, the
following report is being submitted:

95-004-00

Sincerely,

A handwritten signature in cursive script that reads 'A. A. Blind'.

A. A. Blind
Plant Manager

/clc

Attachment

c: H. J. Miller, Region III
E. E. Fitzpatrick
P. A. Barrett
R. F. Kroeger
M. A. Bailey - Ft. Wayne
S. J. Brewer
J. R. Padgett
G. Charnoff, Esq.
D. Hahn
Records Center, INPO
NRC Resident Inspector

100034

9510130390 951010
PDR ADDCK 05000315
S PDR

A handwritten signature in cursive script, possibly reading 'J. H. H.', followed by a vertical line.

LICENSEE EVENT REPORT (LER)

(See reverse for required number of digits/characters for each block)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNBB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)

Donald C. Cook Nuclear Plant - Unit 1

DOCKET NUMBER (2)

05000 315

PAGE (3)

1 OF 3

TITLE (4) T/S 3.0.3 Entered On Loss Of Four Loop Injection Capability Due To Personnel Error During Performance Of Surveillance.

EVENT DATE (5)			LER NUMBER (6)			REPORT NUMBER (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
07	04	95	95	-- 004 --	00	10	10	95	FACILITY NAME	DOCKET NUMBER
										05000
										05000

OPERATING MODE (9)	1	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more) (11)						
POWER LEVEL (10)	57	20.402(b)		20.405(c)		50.73(a)(2)(iv)		73.71(b)
		20.405(a)(1)(i)		50.36(c)(1)		50.73(a)(2)(v)		73.71(c)
		20.405(a)(1)(ii)		50.36(c)(2)		50.73(a)(2)(vii)		OTHER
		20.405(a)(1)(iii)	X	50.73(a)(2)(i)		50.73(a)(2)(viii)(A)		(Specify in Abstract below and in Text, NRC Form 366A)
		20.405(a)(1)(iv)		50.73(a)(2)(ii)		50.73(a)(2)(viii)(B)		
		20.405(a)(1)(v)		50.73(a)(2)(iii)		50.73(a)(2)(x)		

LICENSEE CONTACT FOR THIS LER (12)

NAME

W. Nichols, Operations Superintendent

TELEPHONE NUMBER (Include Area Code)

616-465-5901 x2536

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS

SUPPLEMENTAL REPORT EXPECTED (14)

YES

(If yes, complete EXPECTED SUBMISSION DATE)

X

NO

EXPECTED
SUBMISSION
DATE (15)

MONTH DAY YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16)

On July 4, 1995, with Unit 1 in Mode 1 at 57 percent power and ECCS valve operability testing in progress, the Safety Injection pump discharge cross-tie train "B" shutoff valve was cycled for ISI timing. It was subsequently determined that while the SI valve was in the closed position, the East and West RHR pump discharge cross-tie shutoff valves were also closed as a result of a personnel error. With the unit in this configuration, all four loop injection was lost and Tech Spec 3.0.3 was entered. Upon discovery, the RHR cross-tie valves were immediately reopened, restoring four loop injection.

Based on the available information, this event was originally determined to not be reportable under either 10CFR50.72 or 10CFR50.73. However, the event was later reevaluated and determined to be reportable under 10CFR50.73(a)(2)(i)(B), as an event where a Limiting Condition for Operation was not met.

This event has minimal safety significance as the duration of the event was momentary. An analysis was subsequently performed that supported the conclusion that operation of the unit at reduced power with both the RHR and SI cross-ties closed does not represent a significant safety concern. At no time was the health or safety of the public jeopardized.

LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
Donald C. Cook Nuclear Plant - Unit 1	0 5 0 0 0 3 1 5	9 5	0 0 4	0 0	0 2	OF	0 3

TEXT (If more space is required, use additional NRC Form 365A's) (17)

Conditions Prior to Occurrence

Unit 1 in Mode 1 at 57 percent Rated Thermal Power.

Description of Event

On July 4, 1995, during performance of **01-OHP 4030.STP.053B, ECCS Valve Operability Test, Step 1.8.1.e was not performed correctly. This step contains the requirement to verify 1-IMO-314, the East Residual Heat Removal pump discharge cross-tie shutoff valve, and 1-IMO-324, the West Residual Heat Removal pump discharge cross-tie shutoff valve, to be OPERABLE and in the "open" position. This was not performed in accordance with procedure prior to proceeding to the next step which cycles the 1-IMO-275, Safety Injection pump discharge cross-tie Train "B" shutoff valve. The 1-IMO-275 valve was cycled for ISI timing and returned to its required open position.

Later in the same procedure it was discovered that the 1-IMO-314 and 1-IMO-324 valves were still closed. With the RHR pump discharge cross-tie shutoff valves in the "closed" position, the closure of the SI pump discharge cross-tie shutoff valve earlier in the procedure had rendered four loop injection inoperable, and caused an entry in Technical Specification 3.0.3.

Cause of Event

This event is attributable to a lack of attention to detail by the Reactor Operator performing the procedure.

Analysis of Event

This event is being reported under 10CFR50.73(a)(2)(i)(B), as an event where a Limiting Condition for Operation was not met. In accordance with Technical Specifications 3.5.2 and 3.0.3, requirements for operable ECCS subsystems were not met.

This event has minimal safety significance as the duration of the event was very short, approximately 49 seconds. The analysis for 57 percent power which was subsequently performed supported the conclusion that operation of the unit at reduced power with both the RHR and SI cross-ties valves in the "closed" position does not represent a significant safety concern. At no time was the health or safety of the public jeopardized.

LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-630), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)

DOCKET NUMBER (2)

LER NUMBER (6)

PAGE (3)

Donald C. Cook Nuclear Plant - Unit 1

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YEAR	SEQUENTIAL NUMBER	REVISION NUMBER
9 5	— 0 0 4	— 0 0

0 | 3 | OF | 0 | 3

TEXT (If more space is required, use additional NRC Form 366A's) (17)

The event was originally evaluated and determined to not be reportable under either 10CFR50.72 or 10CFR50.73. This conclusion was based on the fact that there is no specific requirement in the Unit 1 Technical Specifications regarding the cross-ties. A subsequent analysis for the reduced power level also supported the classification as not reportable. However, the event was reevaluated in August of 1995 and it was determined that the event was reportable under 10CFR50.73(a)(2)(i)(B), as Technical Specification 3.0.3 was entered.

Corrective Action

The RHR valves were immediately restored to the open position.

The Reactor Operator was counseled by the Shift Supervisor and the Shift Lead Trainer on the importance of maintaining attention to detail when performing procedural evolutions.

The Operations Department management has developed an action plan to minimize events caused by personnel errors. This plan includes long term provisions to improve the quality of operating procedures and promote a continuously self-checking culture among the Operations personnel. All Operations shift personnel were briefed on the action plan.

Failed Component Identification

Not Applicable

Previous Similar Events

316/95-001-00

315/92-014-00

316/91-008-00