

PRIORITY 1

(ACCELERATED RIDS PROCESSING)

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ACCESSION NBR:9509290024 DOC.DATE: 95/09/25 NOTARIZED: NO DOCKET #
FACIL:50-315 Donald C. Cook Nuclear Power Plant, Unit 1, Indiana M 05000315
AUTH.NAME AUTHOR AFFILIATION
WEBER,G.A. Indiana Michigan Power Co. (formerly Indiana & Michigan Ele
BLIND,A.A. Indiana Michigan Power Co. (formerly Indiana & Michigan Ele
RECIP.NAME RECIPIENT AFFILIATION

SUBJECT: LER 95-007-00:on 950825,Conax seal assemblies on RV post-
accident vent SVs 1-NSO-21 & 1-NSO-23 found to be loose.
Determined that assemblies missing midlock ferrule.Further
investigation currently in progress.W/950925 ltr.

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TITLE: 50.73/50.9 Licensee Event Report (LER), Incident Rpt, etc.

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Indiana Michigan
Power Company
Cook Nuclear Plant
One Cook Place
Bridgman, MI 49106
616 465 5901



September 25, 1995

United States Nuclear Regulatory Commission
Document Control Desk
Rockville, Maryland 20852

Operating Licenses DPR-58
Docket No. 50-315

Document Control Manager:

In accordance with the criteria established by
10 CFR 50.73 entitled Licensee Event Report System, the
following report is being submitted:

95-007-00

Sincerely,

A. A. Blind
Plant Manager

/clc

Attachment

c: H. J. Miller, Region III
E. E. Fitzpatrick
P. A. Barrett
R. F. Kroeger
M. A. Bailey - Ft. Wayne
S. J. Brewer
J. R. Padgett
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LICENSEE EVENT REPORT (LER)

(See reverse for required number of digits/characters for each block)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNBB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)

D.C. Cook Nuclear Plant - Unit 1

DOCKET NUMBER (2)

05000 315

PAGE (3)

1 OF 1

TITLE (4)

Conax Seal Assemblies on Environmentally Qualified Equipment Found Installed Incorrectly

EVENT DATE (5)			LER NUMBER (6)			REPORT NUMBER (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
8	25	95	95	007	00	9	25	95	Cook -.Unit 2	05000 316
									FACILITY NAME	DOCKET NUMBER
										05000
OPERATING MODE (9)		6	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more) (11)							
POWER LEVEL (10)		0	20.402(b)		20.405(c)		50.73(a)(2)(iv)		73.71(b)	
			20.405(a)(1)(i)		50.36(c)(1)		X 50.73(a)(2)(v)		73.71(c)	
			20.405(a)(1)(ii)		50.36(c)(2)		50.73(a)(2)(vii)		OTHER	
			20.405(a)(1)(iii)		50.73(a)(2)(i)		50.73(a)(2)(viii)(A)		(Specify in Abstract below and in Text, NRC Form 366A)	
			20.405(a)(1)(iv)		50.73(a)(2)(ii)		50.73(a)(2)(viii)(B)			
			20.405(a)(1)(v)		50.73(a)(2)(iii)		50.73(a)(2)(x)			

LICENSEE CONTACT FOR THIS LER (12)

NAME

G.A. Weber - Plant Engineering Superintendent

TELEPHONE NUMBER (Include Area Code)

616-465-5901, x2511

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS

SUPPLEMENTAL REPORT EXPECTED (14)

X	YES (If yes, complete EXPECTED SUBMISSION DATE)	NO	EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
				10	31	95

ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16)

On August 25, 1995, at 2000 hours with Unit 1 in Mode 6 (refueling with fuel removed), Conax seal assemblies on Reactor Vessel Post-Accident Vent Solenoid Valves 1-NSO-21 and 1-NSO-23 were found to be loose. Upon further inspection of this condition, it was determined that both of these assemblies were missing a midlock ferrule. This ferrule "swages" on the feedthrough barrel to provide the environmental qualification (EQ) seal when appropriately torqued. A review of the design change package that originally installed these Conax seal assemblies to the head vents revealed that the Unit 1 and Unit 2 Reactor Vessel Post-Accident Vent Solenoid Valves and Pressurizer Post-Accident Vent Solenoid Valves were inadequately torqued. Two other EQ installation anomalies were also discovered during this evolution: 1) missing ferrules on the Acoustic Valve Monitoring System (AVMS) Conax assemblies and, 2) an inadequate torque requirement implementation on the Pressurizer Pressure Relief Valves' Namco limit switch Conax Assemblies.

Corrective actions have been completed for the anomalies identified in Unit 2. Thus far, the investigation has determined no actual impact on the health and safety of the public. Further investigation is currently in progress to resolve this issue, including any generic issue concerning the installation of EQ equipment. This is an interim report. The final report is expected to be submitted by October 31, 1995.