

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR:9509220041 DOC.DATE: 95/09/18 NOTARIZED: NO DOCKET #
FACIL:50-316 Donald C. Cook Nuclear Power Plant, Unit 2, Indiana M 05000316
AUTH.NAME AUTHOR AFFILIATION
NICHOLS,W.A. Indiana Michigan Power Co. (formerly Indiana & Michigan Ele
BLIND,A.A. Indiana Michigan Power Co. (formerly Indiana & Michigan Ele
RECIP.NAME RECIPIENT AFFILIATION

SUBJECT: LER 94-010-00:on 941012,unexpected isolation of containment
purge valves occurred during surveillance testing due to
personnel error.Removed containment purge exhaust fans from
svc.W/950918 ltr.

DISTRIBUTION CODE: IE22T COPIES RECEIVED:LTR 1 ENCL 1 SIZE: 4
TITLE: 50.73/50.9 Licensee Event Report (LER), Incident Rpt, etc.

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| | NOAC MURPHY,G.A | 1 1 | NOAC POORE,W. | 1 1 |
| | NRC PDR | 1 1 | NUDOCS FULL TXT | 1 1 |

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Indiana Michigan
Power Company
Cook Nuclear Plant
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616 465 5901



September 18, 1995

United States Nuclear Regulatory Commission
Document Control Desk
Rockville, Maryland 20852

Operating Licenses DPR-74
Docket No..50-316

Document Control Manager:

In accordance with the criteria established by
10 CFR 50.73 entitled Licensee Event Report System, the
following report is being submitted:

94-010-00

Sincerely,

A. A. Blind
Plant Manager

/clc

Attachment

c: H. J. Miller, Region III
E. E. Fitzpatrick
P. A. Barrett
R. F. Kroeger
M. A. Bailey - Ft. Wayne
S. J. Brewer
M. R. Padgett
G. Charnoff, Esq.
D. Hahn
Records Center, INPO
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250013

9509220041 950918
PDR ADUCK 05000316
S PDR

LICENSEE EVENT REPORT (LER)

(See reverse for required number of digits/characters for each block)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS
INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD
COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION
AND RECORDS MANAGEMENT BRANCH (MNBB 7714), U.S. NUCLEAR
REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO
THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF
MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)

Donald C. Cook Nuclear Plant

DOCKET NUMBER (2)

05000 316

PAGE (3)

1 OF 3

TITLE (4)

Unexpected isolation of containment purge valves during surveillance testing due to personnel error.

| EVENT DATE (5) | | | LER NUMBER (6) | | | REPORT NUMBER (7) | | | OTHER FACILITIES INVOLVED (8) | | |
|--------------------|-----|------|---------------------------------------------------------------------------------------------|-------------------|-----------------|-------------------|-----|------|-------------------------------|----------------------|--------------------------------------------------------|
| MONTH | DAY | YEAR | YEAR | SEQUENTIAL NUMBER | REVISION NUMBER | MONTH | DAY | YEAR | FACILITY NAME | DOCKET NUMBER | |
| 10 | 12 | 94 | 94 | 010 | 00 | 09 | 18 | 95 | | 05000 | |
| OPERATING MODE (9) | | | THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more) (11) | | | | | | | | |
| POWER LEVEL (10) | | | 20.402(b) | | | 20.405(c) | | | X | 50.73(a)(2)(iv) | 73.71(b) |
| | | | 20.405(a)(1)(i) | | | 50.36(c)(1) | | | | 50.73(a)(2)(v) | 73.71(c) |
| | | | 20.405(a)(1)(ii) | | | 50.36(c)(2) | | | | 50.73(a)(2)(vii) | OTHER |
| | | | 20.405(a)(1)(iii) | | | 50.73(a)(2)(i) | | | | 50.73(a)(2)(viii)(A) | (Specify in Abstract below and in Text, NRC Form 368A) |
| | | | 20.405(a)(1)(iv) | | | 50.73(a)(2)(ii) | | | | 50.73(a)(2)(viii)(B) | |
| | | | 20.405(a)(1)(v) | | | 50.73(a)(2)(iii) | | | | 50.73(a)(2)(x) | |

LICENSEE CONTACT FOR THIS LER (12)

NAME

W.A. Nichols - Operations Superintendent

TELEPHONE NUMBER (include Area Code)

616/465-5901 ext. 2536

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

| CAUSE | SYSTEM | COMPONENT | MANUFACTURER | REPORTABLE TO NPRDS | | CAUSE | SYSTEM | COMPONENT | MANUFACTURER | REPORTABLE TO NPRDS |
|-------|--------|-----------|--------------|---------------------|--|-------|--------|-----------|--------------|---------------------|
| | | | | | | | | | | |
| | | | | | | | | | | |

SUPPLEMENTAL REPORT EXPECTED (14)

| | | | | | | |
|----------------------------------------------------|---|----|-------------------------------|-------|-----|------|
| YES (If yes, complete EXPECTED SUBMISSION DATE) | X | NO | EXPECTED SUBMISSION DATE (15) | MONTH | DAY | YEAR |
|----------------------------------------------------|---|----|-------------------------------|-------|-----|------|

ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16)

On October 12, 1994, in preparation for performance of Engineered Safety Features (ESF) Time Response Testing, containment purge isolation valves were closed. When delays occurred in restoring plant conditions from a previous surveillance test the same purge isolation valves were opened and the containment purge exhaust fans started to provide containment cooling. When conditions again existed to continue with the ESF time response testing the test coordinator incorrectly assumed that the purge valves had remained closed. During actual ESF time response testing the opened containment purge isolation valves automatically closed per design. This was unexpected by the test coordinator as he had assumed the valves were closed.

Based on the available information at the time of the event, this event was originally determined to not be reportable under 10CFR50.72 and 10CFR50.73. However, the event was later reevaluated and determined to be reportable under 10CFR50.72(b)(2)(ii), as any event or condition that results in manual or automatic actuation of any Engineered Safety Feature, including the Reactor Protection System and under 10CFR50.73(a)(2)(iv) as any event or condition that resulted in a manual or automatic actuation of any Engineered Safety Feature, including the Reactor Protection System as the closure of the valves was an unexpected response to an ESF signal.

This event has minimal safety significance as the system performed as designed and the reactor core had been unloaded into the spent fuel pool at the time of testing.

LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

| | | | | | | | |
|-------------------------------------------------------|------------------------------------------------------------|----------------|-------------------|-----------------|----------|--|--|
| FACILITY NAME (1) Donald C. Cook Nuclear Plant | DOCKET NUMBER (2) 0 5 0 0 0 3 1 6 9 4 — 0 1 0 — 0 0 | LER NUMBER (6) | | | PAGE (3) | | |
| | | YEAR | SEQUENTIAL NUMBER | REVISION NUMBER | | | |
| | | | | | | | |

TEXT (If more space is required, use additional NRC Form 366A's) (17)

CONDITIONS PRIOR TO OCCURRENCE

Mode 6, Refueling, reactor core had been unloaded into spent fuel pool.

DESCRIPTION OF EVENT

On October 12, 1994, in preparation for performance of Engineered Safety Features (ESF) Time Response Testing, containment purge isolation valves (EIS/JM-ISV) were closed. When delays occurred in restoring plant conditions from a previous surveillance test the same purge isolation valves were opened and the containment purge exhaust fans started to provide containment cooling. When conditions again existed to continue with the ESF time response testing the test coordinator incorrectly assumed that the purge valves had remained closed. During actual ESF time response testing the opened containment purge isolation valves automatically closed per design. This was unexpected by the test coordinator as he had assumed the valves were closed.

CAUSE OF EVENT

The cause for this event is personnel error. The Test Coordinator incorrectly assumed that equipment line-ups had remained unchanged during delays without establishing component control or reconfirming proper alignment.

ANALYSIS

Based on the available information at the time of the event, this event was originally determined to not be reportable under 10CFR50.72 and 10CFR50.73. The original reportability review had concluded that this event was not reportable based on the preplanning of the engineered safety feature actuation signal without consideration of reporting the unexpected consequences. However, the event was later reevaluated and determined to be reportable under 10CFR50.72(b)(2)(ii), as any event or condition that results in manual or automatic actuation of any Engineered Safety Feature, including the Reactor Protection System and under 10CFR50.73(a)(2)(iv) as any event or condition that resulted in a manual or automatic actuation of any Engineered Safety Feature, including the Reactor Protection System as the closure of the valves was an unexpected response to an ESF signal.

On August 17, 1995, after concluding that this event was reportable, an NRC ENS notification was made in accordance with 10CFR50.72. This Licensee Event Report is being made in accordance with 10CFR50.73(a)(2)(iv) 30 days from determination that the event is reportable.

This event has minimal safety significance as the containment purge system performed as designed and the reactor core had been unloaded into the spent fuel pool at the time of testing.

LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)

DOCKET NUMBER (2)

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Donald C. Cook Nuclear Plant

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TEXT (If more space is required, use additional NRC Form 366A's) (17)

CORRECTIVE ACTION

The containment purge exhaust fans were immediately removed from service upon automatic closure of their associated isolation valves.

The individual involved was counselled on the importance of assuring proper test conditions and not assuming plant conditions following delays or interruptions when component control has not been established.

This event was reviewed with subsequent test coordinators prior to performance of similar testing on Unit 1 during the Unit 1 1995 refueling outage to sensitize them to this issue.

FAILED COMPONENT IDENTIFICATION

None

PREVIOUS SIMILAR EVENTS

None