

PRIORITY 1

(ACCELERATED RIDS PROCESSING)

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR:9509210056 DOC.DATE: 95/09/14 NOTARIZED: NO DOCKET #
FACIL:50-315 Donald C. Cook Nuclear Power Plant, Unit 1, Indiana M 05000315 P
AUTH.NAME AUTHOR AFFILIATION
WEBER,G.A. Indiana Michigan Power Co. (formerly Indiana & Michigan Ele R
BLIND,A.A. Indiana Michigan Power Co. (formerly Indiana & Michigan Ele
RECIP.NAME RECIPIENT AFFILIATION

SUBJECT: LER 95-006-00:on 950825,containment Type B&C leak rate
exceeded LCO value due to leakrate from two valves.
Evaluation of valves failure will be included in suppl rept.
Final rept expected to be submitted by 951031.W/950914 ltr.

DISTRIBUTION CODE: IE22T COPIES RECEIVED:LTR 1 ENCL 1 SIZE: 2
TITLE: 50.73/50.9 Licensee Event Report (LER), Incident Rpt, etc.

NOTES:

RECIPIENT ID CODE/NAME	COPIES LTTR ENCL	RECIPIENT ID CODE/NAME	COPIES LTTR ENCL
PD3-1 PD	1 1	HICKMAN,J	1 1
INTERNAL: AEOD/SPD/RAB	2 2	AEOD/SPD/RRAB	1 1
FILE CENTER	1 1	NRR/DE/ECGB	1 1
NRR/DE/EELB	1 1	NRR/DE/EMEB	1 1
NRR/DISP/PIPB	1 1	NRR/DRCH/HHFB	1 1
NRR/DRCH/HICB	1 1	NRR/DRCH/HOLB	1 1
NRR/DRPM/PECB	1 1	NRR/DSSA/SPLB	1 1
NRR/DSSA/SPSB/B	1 1	NRR/DSSA/SRXB	1 1
RES/DSIR/EIB	1 1	RGN3 FILE 01	1 1
EXTERNAL: L ST LOBBY WARD	1 1	LITCO BRYCE,J H	2 2
NOAC MURPHY,G.A	1 1	NOAC POORE,W.	1 1
NRC PDR	1 1	NUDOCS FULL TXT	1 1

NOTE TO ALL "RIDS" RECIPIENTS:

PLEASE HELP US TO REDUCE WASTE! CONTACT THE DOCUMENT CONTROL
DESK, ROOM OWFN 5D8 (415-2083) TO ELIMINATE YOUR NAME FROM
DISTRIBUTION LISTS FOR DOCUMENTS YOU DON'T NEED!

FULL TEXT CONVERSION REQUIRED
TOTAL NUMBER OF COPIES REQUIRED: LTTR 26 ENCL 26



September 14, 1995

United States Nuclear Regulatory Commission
Document Control Desk
Rockville, Maryland 20852

Operating Licenses DPR-58
Docket No. 50-315

Document Control Manager:

In accordance with the criteria established by
10 CFR 50.73 entitled Licensee Event Report System, the
following report is being submitted:

95-006-00

Sincerely,

A handwritten signature in cursive script that reads 'A. A. Blind'.

A. A. Blind
Plant Manager

/clc

Attachment

c: H. J. Miller, Region III
E. E. Fitzpatrick
P. A. Barrett
R. F. Kroeger
M. A. Bailey - Ft. Wayne
S. J. Brewer
J. R. Padgett
G. Charnoff, Esq.
D. Hahn
Records Center, INPO
NRC Resident Inspector

000000

9509210056 950914
PDR ADDCK 05000315
S PDR

Handwritten initials 'JE22' in a stylized cursive font.

LICENSEE EVENT REPORT (LER)

(See reverse for required number of digits/characters for each block)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNBB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)

D.C. Cook Nuclear Plant - Unit 1

DOCKET NUMBER (2)

05000315

PAGE (3)

1 OF 1

TITLE (4)

Containment Type B&C Leakrate Exceeds LCO Value

EVENT DATE (5)

LER NUMBER (6)

REPORT NUMBER (7)

OTHER FACILITIES INVOLVED (8)

MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
08	25	95	95	006	00	09	14	95		05000
										05000

OPERATING MODE (9)

6

POWER LEVEL (10)

000

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more) (11)

20.402(b)	20.405(c)	50.73(a)(2)(iv)	73.71(b)
20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)
20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	OTHER
20.405(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	(Specify in Abstract below and in Text, NRC Form 366A)
20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME

G.A. Weber - Plant Engineering Superintendent

TELEPHONE NUMBER (Include Area Code)

616-465-5901

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRPDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRPDS

SUPPLEMENTAL REPORT EXPECTED (14)

X	YES (If yes, complete EXPECTED SUBMISSION DATE)	NO	EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
				10	31	95

ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16)

On August 25, 1995, with the Reactor Coolant System in Mode 6 (refueling with fuel removed), the accumulated leakage of the type B and C Leak Rate Testing was 0.88 La. This exceeds the Technical Specification LCO value of 0.60 La. The measured leakrate from two valves is the primary reason for exceeding the 0.60 La limit. Valve 1-R-157 (Ice Condenser Refrigeration Glycol Return Header Containment Isolation Valves Pressure Relief Header Check Valve) had a measured leakrate of 45,600 SCCM and valve 1-NS-357 (Post Accident Sampling Station Sample Waste Return to Unit-1 Containment Check Valve) had a measured leakrate of 45,500 SCCM. The leakage from these two valves has accounted for 93 percent of the measured leakage. An evaluation of the valve failures will be included in the supplemental report to this LER.

The Type B and C Leakrate Testing is 97 percent complete and will continue throughout the Refueling Outage. Those Containment Isolation Valves that exhibit unacceptable leakrates are repaired and retested to ensure that the as-left leakrates are within allowable limits.

This is an interim report. The final report is expected to be submitted by October 31, 1995.