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SUBJECT: Forwards rev to current ISI program,incorporating changes for listed valves.Editoral changes,including note referring to TS amends,incorporated into revised pages also.

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March 30, 1995

AEP:NRC:0969AC

Docket Nos.: 50-315
50-316

U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, D. C. 20555

Gentlemen:

Donald C. Cook Nuclear Plant Units 1 and 2
REVISION TO CURRENT INSERVICE INSPECTION PROGRAM

The Cook Nuclear Plant valve inservice test program has been revised and the attached pages are provided for updating the NRC's copy of the program. The revisions to the program incorporate changes for the following valves:

Valves 1/2-FRV-247, -257, -258: A recent configuration change and a system engineer review of these valves has resulted in the removal of their previously assigned closed safety function. Since these valves are now passive Category B valves (normally open with an open safety function), there are no inservice test requirements per IWV-3700, Table IWV-3700-1. The valves, therefore, have been removed from the program.

Valves 1/2-RH-133, -134: Seat leakage acceptance criterion was revised from a 5 gpm limit to a 4 gpm limit as a result of technical specification amendment Nos. 188 (Unit 1) and 174 (Unit 2).

Also, editorial changes, including a note referring to the technical specification amendments, have been incorporated into the revised pages.

A044

The changes to the program are being provided for your information and do not require any additional relief from the requirements of ASME Section XI.

Sincerely,

for W. G. Smith

E. E. Fitzpatrick
Vice President

plt

Attachment

cc: A. A. Blind
G. Charnoff
J. B. Martin
NFEM Section Chief
NRC Resident Inspector - Bridgman
J. R. Padgett

ATTACHMENT TO AEP:NRC:0969AC

REVISED INSERVICE INSPECTION SERVICE

PROGRAM PAGES

DONALD C. COOK NUCLEAR PLANT

ASME SECTION XI VALVE TEST PROGRAM FOR UNIT #1

ATTACHMENT "A"

Revision No: 4A

Date: 3-3-95

3. PRESSURE ISOLATION VALVES (Category A or AC)

Testing Method: (SLT-1) Seat leakage test the valve per ASME Code Section XI (refer to Figure 3, Item #3).

Valve No.	Flow Diagram	Size	Type	Permissible Leakage Values (GPM)
CS-299E	5129	4	CK	2.0
CS-299W	5129	4	CK	2.0
SI-152-N	5143	4	CK	5.0
SI-152-S	5143	4	CK	5.0
ICM-129	5143	14	GA(DD)*	10.0
SI-161-L1	5143	6	CK	5.0
SI-161-L4	5143	6	CK	5.0
SI-161-L2,-L3	5143	6	CK	10.0
SI-170-L1	5143	10	CK	5.0
SI-170-L2	5143	10	CK	5.0 **

* Double Discs

** Per Technical Specification Amendment No. 188 to DPR-58.

DONALD C. COOK NUCLEAR PLANT

ASME SECTION XI VALVE TEST PROGRAM FOR UNIT #1

ATTACHMENT "A"

Revision No: 4A

Date: 3-3-95

Valve No.	Flow Diagram	Size	Type	Permissible Leakage Values (GPM)
SI-170-L3	5143	10	CK	5.0 **
SI-170-L4	5143	10	CK	5.0
SI-158-L1,-L4	5143	6	CK	10.0
SI-158-L2,-L3	5143	6	CK	10.0
SI-151-E	5143	8	CK	5.0
SI-151-W	5143	8	CK	5.0
SI-166-L1	5143	10	CK	5.0
SI-166-L2	5143	10	CK	5.0
SI-166-L3	5143	10	CK	5.0
SI-166-L4	5143	10	CK	5.0
RH-133,-134	5143	8	CK	4.0 **

** Per Technical Specification Amendment No. 188 to DPR-58.

DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 1
FLOW DIAGRAM: 1-5106A-38

SYSTEM: AUXILIARY FEEDWATER

VALVE						VALVE POSITION		ASME SECTION XI						
Valve Tag	Rev	Type	Size	Actuator	Coords	Power Operation	Safety Function	Class	Active/ Passive	Category	Test Required	Test Performed	Test Frequency	Relief Request
1-FHO-211	4	GL	4.00	MO	J/4	O	O	3	A	B	EF-1	EF-1	P	NO
											EF-5	EF-5	-	NO
											ET-XXX	ET-XXX	P	NO
1-FHO-212	4	GL	4.00	MO	J/5	O	O	3	A	B	EF-1	EF-1	P	NO
											EF-5	EF-5	-	NO
											ET-XXX	ET-XXX	P	NO
1-FHO-221	4	GL	4.00	MO	F/5	O	O	3	A	B	EF-1	EF-1	P	NO
											EF-5	EF-5	-	NO
											ET-XXX	ET-XXX	P	NO
1-FHO-222	4	GL	4.00	MO	F/6	O	O	3	A	B	EF-1	EF-1	P	NO
											EF-5	EF-5	-	NO
											ET-XXX	ET-XXX	P	NO
1-FHO-231	4	GL	4.00	MO	F/5	O	O	3	A	B	EF-1	EF-1	P	NO
											EF-5	EF-5	-	NO
											ET-XXX	ET-XXX	P	NO
1-FHO-232	4	GL	4.00	MO	F/6	O	O	3	A	B	EF-1	EF-1	P	NO
											EF-5	EF-5	-	NO
											ET-XXX	ET-XXX	P	NO
1-FHO-241	4	GL	4.00	MO	J/5	O	O	3	A	B	EF-1	EF-1	P	NO
											EF-5	EF-5	-	NO
											ET-XXX	ET-XXX	P	NO
1-FHO-242	4	GL	4.00	MO	J/5	O	O	3	A	B	EF-1	EF-1	P	NO
											EF-5	EF-5	-	NO
											ET-XXX	ET-XXX	P	NO
1-FW-124	4	CK	8.00	SA	H/7	C	O	3	A	C	CF-1	CF-1	P	NO
1-FW-128	4	CK	6.00	SA	F/7	C	O	3	A	C	CF-1	CF-1	P	NO
1-FW-132-1	4	CK	4.00	SA	H/5	C	O/C	2	A	C	CF-1	CF-2	-	NO, CSJ 1
1-FW-132-2	4	CK	4.00	SA	F/6	C	O/C	2	A	C	CF-1	CF-2	-	NO, CSJ 1
1-FW-132-3	4	CK	4.00	SA	F/6	C	O/C	2	A	C	CF-1	CF-2	-	NO, CSJ 1
1-FW-132-4	4	CK	4.00	SA	H/5	C	O/C	2	A	C	CF-1	CF-2	-	NO, CSJ 1

DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 1
FLOW DIAGRAM: 1-5106A-38

SYSTEM: AUXILIARY FEEDWATER

VALVE						VALVE POSITION		ASME SECTION XI						
Valve Tag	Rev	Type	Size	Actuator	Coords	Power Operation	Safety Function	Class	Active/ Passive	Category	Test Required	Test Performed	Test Frequency	Relief Request
1-FW-134	4	CK	10.00	SA	L/9	C	O	3	A	C	CF-1	CF-3	C	NO, CSJ 2
1-FW-135	4	CK	8.00	SA	J/8	C	O	3	A	C	CF-1	CF-3	C	NO, CSJ 2
1-FW-138-1	4	CK	4.00	SA	H/4	C	O/C	2	A	C	CF-1	CF-2	-	NO, CSJ 3
1-FW-138-2	4	CK	4.00	SA	F/5	C	O/C	2	A	C	CF-1	CF-2	-	NO, CSJ 3
1-FW-138-3	4	CK	4.00	SA	F/5	C	O/C	2	A	C	CF-1	CF-2	-	NO, CSJ 3
1-FW-138-4	4	CK	4.00	SA	H/4	C	O/C	2	A	C	CF-1	CF-2	-	NO, CSJ 3
1-FW-149	4	CK	0.75	SA	L/3	C	O	3	A	C	CF-1	CF-1	P	NO, NOTE 4
1-FW-150	4	CK	0.75	SA	L/4	C	O	3	A	C	CF-1	CF-1	P	NO, NOTE 4
1-FW-153	4	CK	1.00	SA	F/8	C	O/C	3	A	C	CF-1	CF-1	P	NO, NOTE 5
1-FW-159	4	CK	6.00	SA	C/7	C	O	3	A	C	CF-1	CF-1	P	NO
1-FW-160	4	CK	1.00	SA	C/8	C	O/C	3	A	C	CF-1	CF-1	P	NO, NOTE 5
1-FW-161	4	CK	8.00	SA	E/7	C	O	3	A	C	CF-1	CF-1	P	NO
1-SV-140-A	4	REL	0.75	SA	M/2	C	O	3	A	C	TF-1	TF-1	R	NO
1-SV-140-B	4	REL	0.75	SA	L/2	C	O	3	A	C	TF-1	TF-1	R	NO
1-SV-169-A	4	REL	0.75	SA	D/8	C	O	3	A	C	TF-1	TF-1	R	NO
1-SV-169-B	4	REL	0.75	SA	G/8	C	O	3	A	C	TF-1	TF-1	R	NO
12-CRV-51	4	GL	8.00	A	M/8	O/C	C	3	A	B	EF-1 EF-5 EF-7 ET-XXX	EF-1 EF-5 EF-7 ET-XXX	- - - -	NO NO NO NO

DONALD C. COOK NUCLEAR PLANT

ASME SECTION XI VALVE TEST PROGRAM FOR UNIT #2

ATTACHMENT "A"

Revision No: 4A

Date: 3-3-95

1. CONTAINMENT ISOLATION VALVES (Category A or AC)

Testing Method: (SLT-2) Seat leakage test the valve in accordance with 10CFR50, Appendix J, in lieu of ASME Code Section XI except for paragraphs IWV-3426 and IWV-3427, which are applicable (refer to Figure 3, Item #3).

Valve No	Flow Diagram	Size	Type	Permissible Leakage Values (SCCM)
WCR-920,-922	5114A	3	DA	900
WCR-921,-923	5114A	3	DA	900
WCR-932,-934	5114A	3	DA	900
WCR-933,-935	5114A	3	DA	900
WCR-941,-945	5114A	3	DA	900
WCR-944,-948	5114A	3	DA	900
WCR-951,-955	5114A	3	DA	900
WCR-954,-958	5114A	3	DA	900
WCR-924,-926	5114A	3	DA	900
WCR-925,-927	5114A	3	DA	900
WCR-928,-930	5114A	3	DA	900

DONALD C. COOK NUCLEAR PLANT

ASME SECTION XI VALVE TEST PROGRAM FOR UNIT #2

ATTACHMENT "A"

Revision No: 4A

Date: 3-3-95

Valve No.	Flow Diagram	Size	Type	Permissible Leakage Values (SCCM)
WCR-929,-931	5114A	3	DA	900
WCR-942,-946	5114A	3	DA	900
WCR-952,-956	5114A	3	DA	900
WCR-943,-947	5114A	3	DA	900
WCR-953,-957	5114A	3	DA	900
WCR-960,-962	5114A	2	DA	750
WCR-961,-963	5114A	2	DA	750
WCR-964,-966	5114A	2	DA	750
WCR-965,-967	5114A	2	DA	750
ECR-10,-20	5141B	0.50	GL	750
ECR-11,-21	5141B	0.50	GL	750
ECR-12,-22	5141B	0.50	GL	750
ECR-13,-23	5141B	0.50	GL	750
ECR-14,-24	5141B	0.50	GL	750

DONALD C. COOK NUCLEAR PLANT

ASME SECTION XI VALVE TEST PROGRAM FOR UNIT #2

ATTACHMENT "A"

Revision No: 4A

Date: 3-3-95

Valve No.	Flow Diagram	Size	Type	Permissible Leakage Values (SCCM)
ECR-15,-25	5141B	0.50	GL	750
ECR-16,-26	5141B	0.50	GL	750
ECR-17,-27	5141B	0.50	GL	750
ECR-18,-28	5141B	0.50	GL	750
ECR-19,-29	5141B	0.50	GL	750
CS-442-1	5128A	2	CK	750
CS-442-2	5128A	2	CK	750
CS-442-3	5128A	2	CK	750
CS-442-4	5128A	2	CK	750
SI-189	5128A	4	CK	1200
SM-1	5141F	1	CK	750
N-102	5143	1	CK	750
N-159	5128A	0.75	CK	750
PW-275	5128A	3	CK	900

DONALD C. COOK NUCLEAR PLANT

ASME SECTION XI VALVE TEST PROGRAM FOR UNIT #2

ATTACHMENT "A"

Revision No: 4A

Date: 3-3-95

Valve No.	Flow Diagram	Size	Type	Permissible Leakage Values (SCCM)
CS-321	5129	3	CK	1800
VCR-10,-11	5146B	4	DA	1200
VCR-20,-21	5146B	4	DA	1200
DCR-203,-207	5137A	1	DA,GL	750
N-160, DCR-201	5137A	1	CK,DA	1125
DCR-610,-611	5137A	2.50	DA	750
DCR-620,-621	5137A	1	DA	750
DCR-205,-206	5137A	4	DA	1200
DCR-600,-601	5124	3	DA	900
QCR-300	5129	2	GL	375
QCR-301	5129	2	GL	375
QCM-250,-350	5129A	4	GA	1200
QCR-919,-920	5115A	2	DA	750
SF-152,-154	5136	2.50	DA,GL	750
SF-159,-160	5137A	3	DA	900

DONALD C. COOK NUCLEAR PLANT

ASME SECTION XI VALVE TEST PROGRAM FOR UNIT #2

ATTACHMENT "A"

Revision No: 4A

Date: 3-3-95

Valve No.	Flow Diagram	Size	Type	Permissible Leakage Values (SCCM)
NCR-105,-106	5141	0.50	GL	750
NCR-107,-108	5141	0.50	GL	750
NCR-109,-110	5141	0.50	GL	750
RCR-100,-101	5128A	0.375	GL	750
DCR-202,-204	5137A	0.75	DA	750
ICR-5,-6	5141	0.50	GL	750
ECR-33,-35	5141F	0.75,2	GL,DA	750
ICM-260	5142	4	GA(DD)*	600
ICM-265	5142	4	GA(DD)*	600
ECR-31,-32	5141F	1	GL	750
XCR-100	5120B	1	GL	375
XCR-101	5120B	1	GL	375
XCR-102	5120B	1	GL	375
XCR-103	5120B	1	GL	375

* Double Discs

DONALD C. COOK NUCLEAR PLANT

ASME SECTION XI VALVE TEST PROGRAM FOR UNIT #2

ATTACHMENT "A"

Revision No: 4A

Date: 3-3-95

Valve No.	Flow Diagram	Size	Type	Permissible Leakage Values (SCCM)
GCR-301	5128A	0.75	DA	375
GCR-314	5143	1	GL	375
SI-171,-172,-194	5143	0.75	GL	1125
NCR-252	5128A	3	GL	450
CCR-460,-462	5135	3	GL	900
CCR-457,CCW-135	5135	2,2.50	GL,CK	1125
CCR-455,-456	5135	2	GL	750
SM-4,-6	5147A	0.50	GL	750
ICM-251	5142	4	GA(DD)*	600
ICM-250	5142	4	GA(DD)*	600
SM-8,-10	5147A	0.50	ND	750
CCW-243-25	5135B	1	CK	750
CCW-244-25	5135B	1	CK	750

* Double Discs

DONALD C. COOK NUCLEAR PLANT

ASME SECTION XI VALVE TEST PROGRAM FOR UNIT #2

ATTACHMENT "A"

Revision No: 4A

Date: 3-3-95

Valve No.	Flow Diagram	Size	Type	Permissible Leakage Values (SCCM)
CCW-243-72	5135B	1	CK	750
CCW-244-72	5135B	1	CK	750
CCM-430	5135B	1.50	GL	375
CCM-431	5135B	1.50	GL	375
CCR-440	513B	1.50	GL	375
CCR-441	5135B	1.50	GL	375
CCM-432	5135B	1.50	GL	375
CCM-433	5135B	1.50	GL	375
R-156	5146B	0.375	CK	750
R-157	5146B	0.375	CK	750
NS-357	5124	0.50	CK	750
ECR-496,-497	5141C	0.50	GL	750
ECR-416	5141C	0.50	GL	375
ECR-417	5141C	0.50	GL	375
ECR-535	5141C	0.50	GL	375

DONALD C. COOK NUCLEAR PLANT

ASME SECTION XI VALVE TEST PROGRAM FOR UNIT #2

ATTACHMENT "A"

Revision No: 4A

Date: 3-3-95

Valve No.	Flow Diagram	Size	Type	Permissible Leakage Values (SCCM)
ECR-536	5141C	0.50	GL	375
ECR-36	5141F	2	DA	375
PCR-40	5120B	2	GA	375
PA-342	5120B	2	CK	750
NS-283	5141D	0.50	CK	750
NPX-151	5128A	0.50	GL	375
WCR-900,-902	5114A	6	DA	1800
WCR-901,-903	5114A	6	DA	1800
WCR-912,-914	5114A	6	DA	1800
WCR-913,-915	5114A	6	DA	1800
WCR-904,-906	5114A	6	DA	1800
WCR-905,-907	5114A	6	DA	1800
WCR-908,-910	5114A	6	DA	1800
WCR-909,-911	5114A	6	DA	1800
VCR-101,-201	5147A	14	BF	4200

DONALD C. COOK NUCLEAR PLANT

ASME SECTION XI VALVE TEST PROGRAM FOR UNIT #2

ATTACHMENT "A"

Revision No: 4A

Date: 3-3-95

Valve No.	Flow Diagram	Size	Type	Permissible Leakage Values (SCCM)
VCR-102,-202	5147A	14	BF	4200
VCR-103,-203	5147A	24	BF	7200
VCR-104,-204	5147A	30	BF	9000
VCR-105,-205	5147A	30	BF	9000
VCR-106,-206	5147A	24	BF	7200
VCR-107,-207	5147A	14	BF	4200
ICM-305	5143	18	GA(DD)*	2700
ICM-306	5143	18	GA(DD)*	2700
CCM-452,-454,-458	5135	8,4,8	BF,GL,BF	3000
CCM-451,-453,-459	5135	8,4,8	BF,GL,BF	3000

* Double Discs

DONALD C. COOK NUCLEAR PLANT

ASME SECTION XI VALVE TEST PROGRAM FOR UNIT #2

ATTACHMENT "A"

Revision No: 4A

Date: 3-3-95

2. CONTAINMENT SPRAY VALVES (Category A or AC)

Testing Method: As described in "SLT-2A," Figure 3, Item #3.

Valve No.	Flow Diagram	Size	Type	Permissible Leakage Values (CCM)
CTS-131W	5144	8	CK	35.00
CTS-131E	5144	8	CK	35.00
CTS-127W	5144	6	CK	22.55
CTS-127E	5144	6	CK	21.21
RH-141	5144	8	CK	20.70
RH-142	5144	8	CK	23.00

DONALD C. COOK NUCLEAR PLANT

ASME SECTION XI VALVE TEST PROGRAM FOR UNIT #2

ATTACHMENT "A"

Revision No: 4A

Date: 3-3-95

3. PRESSURE ISOLATION VALVES (Category A or AC)

Testing Method: (SLT-1) Seat leakage test the valve per ASME Code Section XI (refer to Figure 3, Item #3).

Valve No.	Flow Diagram	Size	Type	Permissible Leakage Values (GPM)
CS-299E	5129	4	CK	2.0
CS-299W	5129	4	CK	2.0
SI-152-N	5143	4	CK	5.0
SI-152-S	5143	4	CK	5.0
ICM-129	5143	14	GA(DD)*	10.0
SI-161-L1	5143	6	CK	5.0
SI-161-L4	5143	6	CK	5.0
SI-161-L2,-L3	5143	6	CK	10.0
SI-170-L1	5143	10	CK	5.0
SI-170-L2	5143	10	CK	5.0 **

* Double Discs

** Per Technical Specification Amendment No. 174 to DPR-74.

DONALD C. COOK NUCLEAR PLANT

ASME SECTION XI VALVE TEST PROGRAM FOR UNIT #2

ATTACHMENT "A"

Revision No: 4A

Date: 3-3-95

Valve No.	Flow Diagram	Size	Type	Permissible Leakage Values (GPM)
SI-170-L3	5143	10	CK	5.0 **
SI-170-L4	5143	10	CK	5.0
SI-158-L1,-L4	5143	6	CK	10.0
SI-158-L2,-L3	5143	6	CK	10.0
SI-151-E	5143	8	CK	5.0
SI-151-W	5143	8	CK	5.0
SI-166-L1	5143	10	CK	5.0
SI-166-L2	5143	10	CK	5.0
SI-166-L3	5143	10	CK	5.0
SI-166-L4	5143	10	CK	5.0
RH-133,-134	5143	8	CK	4.0 **

**: Per Technical Specification Amendment No. 174 to DPR-74.

DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLOW DIAGRAM: 2-5106A-41

SYSTEM: AUXILIARY FEEDWATER

VALVE						VALVE POSITION		ASME SECTION XI						
Valve Tag	Rev	Type	Size	Actuator	Coords	Power Operation	Safety Function	Class	Active/ Passive	Category	Test Required	Test Performed	Test Frequency	Relief Request
2-FW-134	4	CK	10.00	SA	B/9	C	O	3	A	C	CF-1	CF-3	C	NO, CSJ 2
2-FW-135	4	CK	8.00	SA	E/8	C	O	3	A	C	CF-1	CF-3	C	NO, CSJ 2
2-FW-138-1	4	CK	4.00	SA	C/4	C	O/C	2	A	C	CF-1	CF-2	-	NO, CSJ 3
2-FW-138-2	4	CK	4.00	SA	D/5	C	O/C	2	A	C	CF-1	CF-2	-	NO, CSJ 3
2-FW-138-3	4	CK	4.00	SA	D/5	C	O/C	2	A	C	CF-1	CF-2	-	NO, CSJ 3
2-FW-138-4	4	CK	4.00	SA	C/4	C	O/C	2	A	C	CF-1	CF-2	-	NO, CSJ 3
2-FW-149	4	CK	0.75	SA	D/2	C	O	3	A	C	CF-1	CF-1	P	NO, NOTE 4
2-FW-150	4	CK	0.75	SA	D/3	C	O	3	A	C	CF-1	CF-1	P	NO, NOTE 4
2-FW-153	4	CK	1.00	SA	H/9	C	O/C	3	A	C	CF-1	CF-1	P	NO, NOTE 5
2-FW-159	4	CK	6.00	SA	L/7	C	O	3	A	C	CF-1	CF-1	P	NO
2-FW-160	4	CK	1.00	SA	M/9	C	O/C	3	A	C	CF-1	CF-1	P	NO, NOTE 5
2-FW-161	4	CK	8.00	SA	J/7	C	O	3	A	C	CF-1	CF-1	P	NO
2-SV-140-1	4	REL	0.75	SA	E/1	C	O	3	A	C	TF-1	TF-1	R	NO
2-SV-140-2	4	REL	0.75	SA	D/1	C	O	3	A	C	TF-1	TF-1	R	NO
2-SV-169-A	4	REL	0.75	SA	K/9	C	O	3	A	C	TF-1	TF-1	R	NO
2-SV-169-B	4	REL	0.75	SA	G/9	C	O	3	A	C	TF-1	TF-1	R	NO

DONALD C. COOK NUCLEAR PLANT
SECOND TEN YEAR INTERVAL
VALVE SUMMARY SHEET - UNIT 2
FLOW DIAGRAM: 2-5106A-41

SYSTEM: AUXILIARY FEEDWATER

VALVE						VALVE POSITION		ASME SECTION XI						
Valve Tag	Rev	Type	Size	Actuator	Coords	Power Operation	Safety Function	Class	Active/ Passive	Category	Test Required	Test Performed	Test Frequency	Relief Request
2-FMO-211	4	GL	4.00	MO	B/4	O	O	3	A	B	EF-1	EF-1	P	NO
											EF-5	EF-5	-	NO
											ET-XXX	ET-XXX	P	NO
2-FMO-212	4	GL	4.00	MO	B/4	O	O	3	A	B	EF-1	EF-1	P	NO
											EF-5	EF-5	-	NO
											ET-XXX	ET-XXX	P	NO
2-FMO-221	4	GL	4.00	MO	C/5	O	O	3	A	B	EF-1	EF-1	P	NO
											EF-5	EF-5	-	NO
											ET-XXX	ET-XXX	P	NO
2-FMO-222	4	GL	4.00	MO	C/5	O	O	3	A	B	EF-1	EF-1	P	NO
											EF-5	EF-5	-	NO
											ET-XXX	ET-XXX	P	NO
2-FMO-231	4	GL	4.00	MO	C/5	O	O	3	A	B	EF-1	EF-1	P	NO
											EF-5	EF-5	-	NO
											ET-XXX	ET-XXX	P	NO
2-FMO-232	4	GL	4.00	MO	C/5	O	O	3	A	B	EF-1	EF-1	P	NO
											EF-5	EF-5	-	NO
											ET-XXX	ET-XXX	P	NO
2-FMO-241	4	GL	4.00	MO	B/4	O	O	3	A	B	EF-1	EF-1	P	NO
											EF-5	EF-5	-	NO
											ET-XXX	ET-XXX	P	NO
2-FMO-242	4	GL	4.00	MO	B/4	O	O	3	A	B	EF-1	EF-1	P	NO
											EF-5	EF-5	-	NO
											ET-XXX	ET-XXX	P	NO
2-FW-124	4	CK	8.00	SA	F/7	C	O	3	A	C	CF-1	CF-1	P	NO
2-FW-128	4	CK	6.00	SA	H/7	C	O	3	A	C	CF-1	CF-1	P	NO
2-FW-132-1	4	CK	4.00	SA	C/4	C	O/C	2	A	C	CF-1	CF-2	-	NO, CSJ 1
2-FW-132-2	4	CK	4.00	SA	D/5	C	O/C	2	A	C	CF-1	CF-2	-	NO, CSJ 1
2-FW-132-3	4	CK	4.00	SA	D/5	C	O/C	2	A	C	CF-1	CF-2	-	NO, CSJ 1
2-FW-132-4	4	CK	4.00	SA	C/4	C	O/C	2	A	C	CF-1	CF-2	-	NO, CSJ 1

Indiana Michigan
Power Company
P.O. Box 16631
Columbus, OH 43216

AEP
INDIANA
MICHIGAN
POWER

March 30, 1995

AEP:NRC:0969AC

Docket Nos.: 50-315
50-316

U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, D. C. 20555

Gentlemen:

Donald C. Cook Nuclear Plant Units 1 and 2
REVISION TO CURRENT INSERVICE INSPECTION PROGRAM

The Cook Nuclear Plant valve inservice test program has been revised and the attached pages are provided for updating the NRC's copy of the program. The revisions to the program incorporate changes for the following valves:

Valves 1/2-FRV-247, -257, -258: A recent configuration change and a system engineer review of these valves has resulted in the removal of their previously assigned closed safety function. Since these valves are now passive Category B valves (normally open with an open safety function), there are no inservice test requirements per IWV-3700, Table IWV-3700-1. The valves, therefore, have been removed from the program.

Valves 1/2-RH-133, -134: Seat leakage acceptance criterion was revised from a 5 gpm limit to a 4 gpm limit as a result of technical specification amendment Nos. 188 (Unit 1) and 174 (Unit 2).

Also, editorial changes, including a note referring to the technical specification amendments, have been incorporated into the revised pages.

The changes to the program are being provided for your information and do not require any additional relief from the requirements of ASME Section XI.

Sincerely,

for W. G. Smith, Jr.
E. E. Fitzpatrick
Vice President

plt

Attachment

cc: A. A. Blind
G. Charnoff
J. B. Martin
NFEM Section Chief
NRC Resident Inspector - Bridgman
J. R. Padgett