

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR: 9410110051      DOC. DATE: 94/10/03      NOTARIZED: NO      DOCKET #  
FACIL: 50-316 Donald C. Cook Nuclear Power Plant, Unit 2, Indiana M 05000316  
AUTH. NAME      AUTHOR AFFILIATION  
WEBER, G.A.      Indiana Michigan Power Co. (formerly Indiana & Michigan Ele  
BLIND, A.A.      Indiana Michigan Power Co. (formerly Indiana & Michigan Ele  
RECIP. NAME      RECIPIENT AFFILIATION

SUBJECT: Interim LER 94-007-00: on 940909, containment Type B & C leakage exceeded LCO value. Final rept will be submitted by 941130, following completion of B & C leak rate tests. W/ 941003 ltr.

DISTRIBUTION CODE: IE22T      COPIES RECEIVED: LTR 1 ENCL 1 SIZE: 3  
TITLE: 50.73/50.9 Licensee Event Report (LER), Incident Rpt, etc.

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Indiana Michigan  
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616 465 5901



October 3, 1994

United States Nuclear Regulatory Commission  
Document Control Desk  
Rockville, Maryland 20852

Operating Licenses DPR-74  
Docket No. 50-316

Document Control Manager:

In accordance with the criteria established by  
10 CFR 50.73 entitled Licensee Event Report System,  
the following report is being submitted:

94-007-00

Sincerely,

A handwritten signature in cursive script that reads 'A. A. Blind'.

A. A. Blind  
Plant Manager

/sb  
Attachment

c: J. B. Martin, Region III  
E. E. Fitzpatrick  
P. A. Barrett  
R. F. Kroeger  
M. A. Bailey - Ft. Wayne  
NRC Resident Inspector  
J. B. Hickman - NRC  
J. R. Padgett  
G. Charnoff, Esq.  
D. Hahn  
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PDR ADDCK 05000316  
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## LICENSEE EVENT REPORT (LER)

(See reverse for required number of digits/characters for each block)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNBB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)

D. C. COOK NUCLEAR PLANT - UNIT 2

DOCKET NUMBER (2)

05000 316

PAGE (3)

1 OF 1

TITLE (4)

CONTAINMENT TYPE B AND C LEAKAGE EXCEEDS LCO VALUE

EVENT DATE (5)			LER NUMBER (6)			REPORT NUMBER (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
09	09	94	94	007	00	10	03	94	FACILITY NAME	DOCKET NUMBER
OPERATING MODE (9)			THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more) (11)							
6			20.402(b)			20.405(c)			50.73(a)(2)(iv)	
POWER LEVEL (10)			20.405(a)(1)(i)			50.36(c)(1)			50.73(a)(2)(v)	
00			20.405(a)(1)(ii)			50.36(c)(2)			50.73(a)(2)(vii)	
			20.405(a)(1)(iii)			X 50.73(a)(2)(i)			50.73(a)(2)(viii)(A)	
			20.405(a)(1)(iv)			50.73(a)(2)(ii)			50.73(a)(2)(viii)(B)	
			20.405(a)(1)(v)			50.73(a)(2)(iii)			50.73(a)(2)(x)	

(Specify in Abstract below and in Text, NRC Form 366A)

## LICENSEE CONTACT FOR THIS LER (12)

NAME: G. A. WEBER - PLANT ENGINEERING SUPERINTENDENT  
TELEPHONE NUMBER (Include Area Code): 616-465-5901

## COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS

## SUPPLEMENTAL REPORT EXPECTED (14)

<input checked="" type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)	<input type="checkbox"/> NO	EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
			11	30	94

## ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16)

On September 9, 1992, with the Reactor Coolant System in Mode 6 (Refueling), the accumulated leakage found while performing the Type B and C Leak Rate Tests on Containment penetrations exceeded the L.C.O. value (0.60 La) of Technical Specification 3.6.1.2.b.

Containment Isolation Valve 2-NS-357 (Post Accident Sampling Station Sample Waste return to Unit 2 Containment Check Valve) would not seat and could not be pressurized to the desired test pressure of 12 psig. An estimated leak rate of 60.3 scfm (15.49 La) was calculated for 2-NS-357.

Those Containment Isolation Valves that exhibit leak rates in excess of the acceptance criteria are being repaired and retested to ensure the leak rates are within allowable limits. The B and C Leak Rate Testing is still in progress and will not be completed until the end of the Refueling Outage. The Type B and C Leak Rate Testing is 78 percent completed.

This is an interim report. The final report will be submitted by November 30, 1994, following completion of the B and C Leak Rate Tests.