

## REACTIVITY CONTROL SYSTEMS

### SURVEILLANCE REQUIREMENTS

4.1.1.4 The MTC shall be determined to be within its limits during each fuel cycle as follows:

- a) The MTC shall be measured and compared to the BOL limit specified in the COLR prior to initial operation above 5% of RATED THERMAL POWER, after each fuel loading.
- b) The MTC shall be measured at any THERMAL POWER within 7 EFPD after reaching an equilibrium boron concentration of 300 ppm\*. The measured value shall be compared to the 300 ppm surveillance limit specified in the COLR. In the event this comparison indicates that the MTC will be more negative than the EOL limit, the MTC shall be remeasured at least once per 14 EFPD during the remainder of the fuel cycle and the MTC value compared to the EOL limit.

ADD

\* Measurement of the MTC in accordance to 4.1.1.4.b. may be suspended provided the benchmark criteria and the revised prediction as documented in the COLR are satisfied. Data required for the calculation of the revised prediction is provided in the "Most Negative Moderator Temperature Coefficient Limit Report," per Specification 6.9.1.12.



ADMINISTRATIVE CONTROLS

CORE OPERATING LIMITS REPORT (Continued)

- b. WCAP-8385, "Power Distribution Control and Load Following Procedures - Topical Report," September 1974 (Westinghouse Proprietary),
- c. WCAP-10216-P-A, Part B, "Relaxation of Constant Axial Offset Control/F<sub>Q</sub> Surveillance Technical Specification," June 1983 (Westinghouse Proprietary),
- d. WCAP-10266-P-A Rev. 2, "The 1981 Version of Westinghouse Evaluation Mode Using BASH Code," March 1987 (Westinghouse Proprietary).

INSERT  
A  
HERE

6.9.1.11.3 The core operating limits shall be determined so that all applicable limits (e.g., fuel thermal-mechanical limits, core thermal-hydraulic limits, ECCS limits, nuclear limits such as shutdown margin, and transient and accident analysis limits) of the safety analysis are met.

6.9.1.11.4 The CORE OPERATING LIMITS REPORT, including any mid-cycle revisions or supplements thereto, shall be provided upon issuance, for each reload cycle, to the NRC document control desk with copies to the Regional Administrator and Resident Inspector.

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SPECIAL REPORTS

6.9.2 Special reports shall be submitted to the attention of the document control desk - U.S. Nuclear Regulatory Commission (Washington, D.C. 20555), with copies to the Region III Administrator and the Resident Inspector at the Cook Nuclear Plant within the time period specified for each report. These reports shall be submitted covering the activities identified below pursuant to the requirements of the applicable reference specification:

- a. Inoperable Seismic Monitoring Instrumentation, Specification 3.3.3.3.
- b. Seismic Monitoring Instrumentation Actuated, Specification 4.3.3.3.2.
- c. Inoperable Meteorological Monitoring Instrumentation, Specification 3.3.3.4.
- d. High Specific Activity in RCS Coolant, Specification 3.4.8.
- e. RCS Pressure Transient Mitigated By RHR Safety Valve or RCS Vent(s), Specification 3.4.9.3.
- f. Moderator Temperature Coefficient, Specification 3.1.1.4.

Insert A

- e. WCAP-13749-P, "Safety Evaluation Supporting The Conditional Exemption of the Most Negative EOL Moderator Temperature Coefficient Measurement," May, 1993 (Westinghouse Proprietary).

Insert B

6.9.1.12 The "Most Negative Moderator Temperature Coefficient Limit Report" shall be prepared prior to each cycle startup and maintained on file. This report will have data to be used for the determination of the revised prediction of the 300 ppm/ARO/RTP MTC, per WCAP-13749-P.

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ATTACHMENT 3 TO AEP:NRC:1028A

PROPOSED  
TECHNICAL SPECIFICATION PAGES



10/10/10

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