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 FACIL:50-315 Donald C. Cook Nuclear Power Plant, Unit 1, Indiana M 05000315
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SUBJECT: Provides annual submittal of LOCA model changes,per
 10CFR50.46(a)(3)(ii).

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AEP:NRC:1118E
10 CFR 50.46(a)(3)(ii)

Donald C. Cook Nuclear Plant Units 1 and 2
Docket Nos. 50-315 and 50-316
License Nos. DPR-58 and DPR-74
REPORT OF LOCA EVALUATION MODEL CHANGES
PURSUANT TO 10 CFR 50.46(a)(3)(ii)

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D. C. 20555

Attn: W. T. Russell

March 25, 1994

Dear Mr. Russell:

Pursuant to the requirements of 10 CFR 50.46(a)(3)(ii), this letter provides our annual submittal of LOCA model changes.

Attachment 1, which was provided to us by Westinghouse Electric Corporation (Westinghouse), describes LOCA model changes which have been permanently implemented and provides a discussion in general terms of the impact of these changes on calculated peak clad temperatures.

Attachment 2 contains the peak clad temperatures calculated specifically for Donald C. Cook Nuclear Plant Units 1 and 2. These temperatures are only slightly different from those contained in our previous submittal with the exception of the SBLOCA analyses with high head safety injection crossties closed. The crossties closed SBLOCA analyses contain larger changes in PCT due to burst and blockage assessments. In all cases, the calculated peak clad temperatures remain within the 10 CFR 50.46 limit of 2200°F.

In addition to the changes in peak clad temperatures associated with the licensing basis LOCA analyses, Attachment 2 also contains changes to LOCA analyses that were submitted via our letter AEP:NRC:1169, dated November 11, 1992; in support of a proposed technical specification change to increase main steam safety valve (MSSV) setpoint tolerances. This submittal, as

ADD 1/0

Mr. W. T. Russell


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AEP:NRC:1118E

modified by our letter AEP:NRC:1169A, dated December 17, 1993, is currently under NRC review and, therefore, the referenced SBLOCA analyses, which are affected, are not yet part of our licensing basis. We have elected to prepare this annual 10 CFR 50.46 report using the analyses performed for the MSSV setpoint tolerance relaxation because these analyses include in their modeling non-discretionary changes to the SBLOCA model as defined in WCAP-13251. The MSSV SBLOCA analyses will bound currently approved operational limits, and are, therefore, conservative.

Regarding plans for future LOCA analyses, the MSSV analyses will provide a new analysis of record for SBLOCA for each unit. The plan for revising the Unit 1 cross-ties open SBLOCA run, discussed in our letter designated AEP:NRC:1118G, is being delayed several months to ensure that the calculation is made with planned code revisions. Current plans also include both LBLOCA and SBLOCA reanalyses in conjunction with evaluations and analyses to support an increase in allowable steam generator tube plugging (SGTP) for Unit 1. This work is tentatively planned to be complete and submitted to the staff prior to start of Unit 1 Cycle 16. Plans for a new LBLOCA analysis of record for Unit 2 are tentative at this time but will not be performed until after the completion of the Unit 1 SGTP analyses.

Sincerely,

for 
E. E. Fitzpatrick
Vice President

cad

Attachments

cc: A. A. Blind - Bridgman
G. Charnoff
J. B. Martin - Region III
NFEM Section Chief
NRC Resident Inspector - Bridgman
J. R. Padgett

