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ACCESSION NBR: 9311180156 DOC. DATE: 93/11/13 NOTARIZED: NO DOCKET #  
 FACIL: 50-316 Donald C. Cook Nuclear Power Plant, Unit 2, Indiana M 05000316  
 AUTH. NAME AUTHOR AFFILIATION  
 LOOPE, D.C. Indiana Michigan Power Co. (formerly Indiana & Michigan Ele  
 BLIND, A.A. Indiana Michigan Power Co. (formerly Indiana & Michigan Ele  
 RECIP. NAME RECIPIENT AFFILIATION

SUBJECT: LER 93-010-00: on 930920, Unit 2 RCS total gaseous activity sample, required by plant Tech Specs missed. Caused by personnel error. Appropriate administrative action taken w/ involved technician. W/931113 ltr.

DISTRIBUTION CODE: IE22T COPIES RECEIVED: LTR 1 ENCL 1 SIZE: 4  
 TITLE: 50.73/50.9 Licensee Event Report (LER), Incident Rpt, etc.

## NOTES:

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NRR/DSSA/SRXB	1 1	REG FILE 02	1 1
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Indiana Michigan  
Power Company  
Cook Nuclear Plant  
One Cook Place  
Bridgman, MI 49106  
616 465 5901



INDIANA  
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November 13, 1993

United States Nuclear Regulatory Commission  
Document Control Desk  
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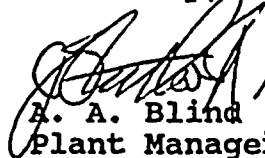
Operating Licenses DPR-74  
Docket No. 50-316

Document Control Manager:

In accordance with the criteria established by  
10 CFR 50.73 entitled Licensee Event Report System,  
the following report is being submitted:

93-010-00

Sincerely,

 11/13/93  
A. A. Blind  
Plant Manager

/sb

Attachment

c: J. B. Martin, Region III  
E. E. Fitzpatrick  
P. A. Barrett  
R. F. Kroeger  
M. A. Bailey - Ft. Wayne  
NRC Resident Inspector  
B. A. Wetzel - NRC  
M. R. Padgett  
G. Charnoff, Esq.  
D. Hahn  
INPO  
S. J. Brewer  
B. A. Svensson

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PDR ADOCK 05000316  
S PDR



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## LICENSEE EVENT REPORT (LER)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 60.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1) Donald C. Cook Nuclear Plant - Unit 2										DOCKET NUMBER (2) 0   5   0   0   0   3   1   6										PAGE (3) 1 OF 0   3		
TITLE (4) Missed Reactor Coolant System Total Gaseous Activity Sample Technical Specification Surveillance Requirement Due to Personnel Error																						
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)												
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES						DOCKET NUMBER(S)							
0   9	2   0	9   3	9   3	0   1   0	0   0	1   1	1   3	9   3							0   5   0   0   0							
OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)																				
1		20.402(b)				20.406(e)				60.73(a)(2)(iv)				73.71(b)								
POWER LEVEL (10)		20.406(a)(1)(i)				60.36(a)(1)				60.73(a)(2)(v)				73.71(c)								
0   7   5		20.406(a)(1)(ii)				60.36(a)(2)				60.73(a)(2)(vi)				OTHER (Specify in Abstract below and in Text, NRC Form 366A)								
		20.406(a)(1)(iii)				X 60.73(a)(2)(i)				60.73(a)(2)(viii)(A)												
		20.406(a)(1)(iv)				60.73(a)(2)(ii)				60.73(a)(2)(viii)(B)												
		20.406(a)(1)(v)				60.73(a)(2)(iii)				60.73(a)(2)(ix)												
LICENSEE CONTACT FOR THIS LER (12)																						
NAME D. C. Loope - Chemistry Superintendent										TELEPHONE NUMBER												
										AREA CODE		6   1   6   4   6   5   -   5   9   0   1										
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																						
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDs		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDs												
SUPPLEMENTAL REPORT EXPECTED (14)										EXPECTED SUBMISSION DATE (15)				MONTH	DAY	YEAR						
YES (If yes, complete EXPECTED SUBMISSION DATE)										X NO												

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On September 20, 1993 a Unit 2 Reactor Coolant System (RCS) total gaseous activity sample, required by the plant's technical specifications, was missed due to personnel error. From a review of logged data of September 20, 1993 it has been concluded that the results recorded for the Unit 2's RCS total gaseous activity were actually Unit 1 results and that the Unit 2 RCS total activity sample surveillance requirement was not performed. An investigation has determined that the Unit 2 RCS sample was collected and prepared; however, the Unit 1 RCS sample was mistakenly counted for the Unit 2 sample. The event was reported per the plant's corrective action program on October 14, 1993.

An evaluation of the Unit 2 total gaseous activity samples taken on September 17, 1993 and September 24, 1993 showed that no adverse trend existed. An evaluation of the Unit 2 liquid reactor coolant sample results taken before, during and after the event corroborate the total gaseous activity evaluation that no adverse trend existed.

Appropriate administrative action has been taken with the involved technician.

LICENSEE EVENT REPORT (LER)  
TEXT CONTINUATION

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 60.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)

DOCKET NUMBER (2)

LER NUMBER (6)

PAGE (3)

YEAR	SEQUENTIAL NUMBER	REVISION NUMBER
93	01	0

Donald C. Cook Nuclear Plant - Unit 2 0500031693-010-002 OF 03

TEXT (If more space is required, use additional NRC Form 368A's) (17)

Conditions Prior to Occurrence

Unit 2 in Mode 1 at approximately 75% power.

Description of Event

On September 20, 1993 a Reactor Coolant System (RCS) total gaseous activity sample, required by the plant's technical specifications, was missed due to personnel error. From a review of the logged data of September 20, 1993 it has been concluded that the results recorded for the Unit 2's RCS total gaseous activity were actually Unit 1 results and that the Unit 2 RCS total activity sample surveillance requirement was not performed. An investigation has determined that the Unit 2 RCS sample was collected and prepared; however, the Unit 1 RCS sample was mistakenly counted for the Unit 2 sample. The event was reported per the plant's corrective action program on October 14, 1993.

Cause of Event

The cause of the event was personnel error. The technician involved did not self check to ensure the correct sample bottle was going to be counted and did not note the abrupt change from the previous Unit 2 samples once the sample had been counted. Had the change been noted, the Unit 2 Reactor Coolant System total gaseous activity sample could have been retaken.

Analysis of Event

This event is being reported in accordance with 10CFR50.73(2)(i)(B) as operation prohibited by the plant's Technical Specification. Technical Specification 3/4.11.2.6 requires that the Reactor Coolant System total gaseous activity be determined once every four days. The total gaseous activity was determined on September 17, 1993 and next on September 24, 1993. This is in violation of the required technical specification frequency requirement.

An evaluation of the total gaseous activity samples taken on September 17, 1993 and September 24, 1993 showed that no adverse trend existed. An evaluation of the liquid reactor coolant sample results taken before, during and after the event corroborate the total gaseous activity evaluation that no adverse trend existed.

Corrective Action

Appropriate administrative action has been taken with the involved technician. In addition, a letter has been written to all section personnel restating the importance of being aware of the results being recorded/reported.

LICENSEE EVENT REPORT (LER)  
TEXT CONTINUATION

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FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
Donald C. Cook Nuclear Plant - Unit 2	05000316	93	010	00	03	OF	03

TEXT (If more space is required, use additional NRC Form 368A's) (17)

Previous Similar Events

315/92-004  
314/92-013  
316/91-003  
325/90-011

Failed Component Identification

None