

# ACCELERATED DISTRIBUTION DEMONSTRATION SYSTEM

## REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR: 9209150307 DOC: DATE: 92/09/11 NOTARIZED: NO DOCKET #  
 FACIL: 50-315 Donald C. Cook Nuclear Power Plant, Unit 1, Indiana M 05000315  
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 BLIND, A. A. Indiana Michigan Power Co. (formerly Indiana & Michigan Ele  
 RECIP. NAME RECIPIENT AFFILIATION

SUBJECT: LER 92-009-00: on 920812, determined failure of two  
 pressurizer safety valves to meet TS required surveillance  
 test criteria. Updated rept will be submitted after  
 evaluation of repair activities. W/920911 ltr.

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 TITLE: 50.73/50.9 Licensee Event Report (LER), Incident Rpt, etc.

### NOTES:

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	NRR/DET/EMEB 7E	1 1	NRR/DLPQ/LHFB10	1 1
	NRR/DLPQ/LPEB10	1 1	NRR/DOEA/OEAB	1 1
	NRR/DREP/PRPB11	2 2	NRR/DST/SELB 8D	1 1
	NRR/DST/SICBBH3	1 1	NRR/DST/SPLB8D1	1 1
	NRR/DST/SRXB 8E	1 1	<u>REG FILE</u> 02	1 1
	RES/DSIR/EIB	1 1	RGN3 FILE 01	1 1
EXTERNAL:	EG&G BRYCE, J. H	2 2	L ST LOBBY WARD	1 1
	NRC PDR	1 1	NSIC MURPHY, G. A	1 1
	NSIC POORE, W.	1 1	NUDOCS FULL TXT	1 1

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September 11, 1992

United States Nuclear Regulatory Commission  
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Operating Licenses DPR-58  
Docket No. 50-315

Document Control Manager:

In accordance with the criteria established by  
10 CFR 50.73 entitled Licensee Event Report System, the  
following report is being submitted:

92-009-00

Sincerely,

*A. A. Blind*  
A. A. Blind  
Plant Manager

/sb

Attachment

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PDR ADDCK 05000315  
S PNR

*JE22*

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On August 12, 1992 with Unit 1 in Mode 6 (Refueling) it was determined that two of three pressurizer safety valves, which were sent to an off site test laboratory for testing, were found with lift settings outside of the Technical Specification acceptance criteria. Acceptable settings are between 2461 psig and 2509 psig. Valve 1-SV-45B was found to have a lift setpoint of 2514 psig and valve 1-SV-45C had a lift setpoint of 2582 psig. The remaining Pressurizer Safety Valve had an acceptable lift setting.

Calibrated safety valves were installed when these valves were removed and sent off site for testing.

The as-found lift setpoints of these Safety Valves would not have resulted in the Reactor Coolant System exceeding the Technical Specification safety limit of 2735 psig.

This is an interim report. An updated report will be submitted after an evaluation of the repair activities and root cause analysis.