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MURLEY,T.E. Document Control Branch (Document Control Desk)

SUBJECT: Suppls 880901 telcon re integrated startup testing program including,testing re vibration measurements of RCS,natural circulation of RCS & steam generator level control instrumentation.

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AMERICAN ELECTRIC POWER

DATE: August 13, 1991 TIME SENT: _____

TO: John Stang NUMBER OF PAGES INCLUDING COVER PAGE: 5

FROM: Laurie Hollen

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W. Long 8/14/91

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INDIANA MICHIGAN POWER

AEP:NRC:09800

Donald C. Cook Nuclear Plant Unit 2
Docket No. 50-316
License No. DPR-74
INTEGRATED STARTUP TESTING PROGRAM

U.S. Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, D. C. 20555

Attn: T. E. Murley

September 7, 1988

Dear Dr. Murley:

This letter is being sent as a follow-up to a September 1, 1988 telephone conversation between American Electric Power Service Corporation and Nuclear Regulatory Commission personnel.

The phone call was made to respond to three questions regarding the Unit 2 Integrated Startup Testing Program. The three areas questioned concerned testing relative to vibration measurements of the reactor coolant system, natural circulation of the reactor coolant system, and steam generator level control instrumentation. The attachment to this letter provides a discussion of each of the items listed above.

This document has been prepared following Corporate procedures which incorporate a reasonable set of controls to ensure its accuracy and completeness prior to signature by the undersigned.

Sincerely,

M. P. Alexich
Vice President

MPA/eh

Attachment

cc: D. H. Williams, Jr.
W. G. Smith, Jr. - Bridgman
R. C. Callen
G. Charnoff
G. Bruchmann
NRC Resident Inspector - Bridgman
A. B. Davis - Region III

QUESTION 1

What procedure(s) will be used to conduct vibration measurements of the Unit 2 reactor coolant system components?

RESPONSE

Reactor internals vibration measurements are deemed unnecessary since no changes were made to the internals package. Changes in replacement steam generators have slightly reduced the reactor coolant system resistance, with only a negligible impact on system flow velocities.

Reactor coolant system vibration measurements are also deemed unnecessary. The replacement steam generators offer only a slight increase in total system mass and are expected to have negligible impact on system vibration response.

During start-up and operation the reactor coolant system is routinely monitored by operator observations and monitoring instrumentation. These same activities are deemed sufficient to notice any unusual system vibration which might occur as a result of Steam Generator Repair Project activities.

QUESTION 2

What procedure(s) will be used to conduct natural circulation tests of the reactor coolant system to confirm that the design heat removal capability exists or to verify that flow (without pumps) or temperature data are comparable to prototype designs for which equivalent tests have been successfully completed?

RESPONSE

Ramp down testing was conducted during initial start-up, and the overall reactor coolant system flow resistance was compared to that of our protype plant, Trojan Nuclear Plant. System resistance was lower than Trojan's and therefore a natural circulation test was not performed.

The replacement steam generators have a lower pressure drop than the original steam generators. Specifically the tube ends are flush with the tube sheet surface, rounded and smooth which result in lower inlet pressure losses. In addition the replacement steam generators have an increased number of tubes. This will reduce the system resistance to a value less than during initial start-up. Therefore, no natural circulation test is required.

Discussions have been held with Westinghouse regarding the licensee's response to Questions 1 and 2 and Westinghouse concurs with both responses.

QUESTION 3

What procedure(s) will be used to calibrate and verify the performance of the steam generator level control system?

RESPONSE

Steam generator level control will be calibrated, using Plant Procedures THP 6030 IMP.204, .205, .206, and .207. The individual inputs to the steam generator level control/feedflow (steam flow, steam pressure, steam generator level) will be calibrated using appropriate plant procedures. No additional testing or calibration is planned as a result of the Steam Generator Repair Project.