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RECIP.NAME RECIPIENT AFFILIATION

SUBJECT: Monthly operating rept for Jan 1991 for DC Cook Unit 1.W/ 910207 ltr.

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RIDS/ADDS

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Cook Nuclear Plant
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Bridgman, MI 49106
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AEP
INDIANA
MICHIGAN
POWER

February 7, 1991

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, D.C. 20555

Gentlemen:

Pursuant to the requirements of Donald C. Cook Nuclear Plant Unit 1 Technical Specification 6.9.1.10, the attached Monthly Operating Report for the month of January 1991 is submitted.

Respectfully,

A. Alan Blind
A.A. Blind
Plant Manager

Attachment

c: NRC Region III
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1/1

N.R.C. OPERATING DATA REPORT

DOCKET NO. 50-315
DATE 01-Feb-91
COMPLETED BY JD JANISSE
TELEPHONE 616-465-5901

OPERATING STATUS

1. Unit Name D. C. Cook Unit 1
2. Reporting Period JAN.91
3. Licensed Thermal Power (MWt) 3250
4. Name Plate Rating (Gross MWe) 1152
5. Design Electrical Rating (Net MWe) 1030
6. Maximum Dependable Capacity (GROSS MWe) 1056
7. Maximum Dependable Capacity (Net MWe) 1020

notes

8. If Changes Occur in Capacity Ratings (Items no. 3 through 7) Since Last Report Give Reasons _____

9. Power Level To Which Restricted. If Any (Net MWe) _____
10. Reasons For Restrictions. If Any: _____

	This Mo.	Yr. to Date	Cumm.
11. Hours in Reporting Period	744.0	744.0	141000.0
12. No. of Hrs. Reactor Was Critical	218.4	218.4	103605.4
13. Reactor Reserve Shutdown Hours	0.0	0.0	463.0
14. Hours Generator on Line	118.8	118.8	101747.6
15. Unit Reserve Shutdown Hours	0.0	0.0	321.0
16. Gross Therm. Energy Gen. (MWH)	168341	168341	295597989
17. Gross Elect. Energy Gen. (MWH)	43350	43350	96174580
18. Net Elect. Energy Gen. (MWH)	39334	39334	92472807
19. Unit Service Factor	16.0	16.0	73.2
20. Unit Availability Factor	16.0	16.0	73.2
21. Unit Capacity Factor (MDC Net)	5.2	5.2	65.2
22. Unit Capacity Factor (DER Net)	5.1	5.1	63.4
23. Unit Forced Outage Rate	0.0	0.0	7.1

24. Shutdowns Scheduled over Next Six Months (Type,Date,and Duration): _____

25. If Shut Down At End of Report Period, Estimated Date of Startup: _____

26. Units in Test Status (Prior to Commercial Operation):

	Forcast	Achieved
INITIAL CRITICALITY		
INITIAL ELECTRICITY		
COMMERCIAL OPERATION		

AVERAGE DAILY POWER LEVEL (MWe-Net)

DOCKET NO. 50-315
UNIT ONE
DATE 01-Feb-91
COMPLETED BY JD JANISSE
TELEPHONE 616-465-5901

MONTH JAN.91

DAY	AVERAGE DAILY POWER LEVEL	DAY	AVERAGE DAILY POWER LEVEL
1	0	17	0
2	0	18	0
3	0	19	0
4	0	20	0
5	0	21	0
6	0	22	0
7	0	23	0
8	0	24	0
9	0	25	0
10	0	26	0
11	0	27	185
12	0	28	262
13	0	29	325
14	0	30	414
15	0	31	453
16	0		

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH: January 1991

DOCKET NO: 50-315
UNIT NAME: D.C. COOK UNIT 1
DATE: February 7, 1991
COMPLETED BY: E.C. Schimmel
TELEPHONE: (616) 465-5901

NO.	DATE	TYPE[1]	DURATION HOURS	REASON[2]	METHOD OF SHUTTING DOWN REACTOR[3]	LICENSEE EVENT REPORT NO.	SYSTEM CODE[4]	COMPONENT CODE[5]	CAUSE AND CORRECTIVE ACTION TO PREVENT RECURRENCE
290 (cont.)	901020	S	623.7	C	N/A	N/A	ZZ	ZZZZZZ	The unit was removed from service at 0031 hours on 901020 for cycle 11-12 refueling and maintenance outage. All refueling activities and maintenance work have been completed. The reactor achieved initial criticality for cycle 12 on 910115 and the unit was paralleled to the system at 2345 hours on 910126. Reactor power was increased to 32% for turbine warming prior to overspeed testing. The total outage duration was 2376.2 hours.
291	910127	S	1.5	B	N/A	N/A	ZZ	ZZZZZZ	The unit was removed from service at 1742 hours on 910127 for main turbine overspeed trip testing. Testing was completed and the unit was returned to service at 1913 hours on 910127.

1
F: Forced
S: Scheduled

2
Reason:
A: Equipment Failure (Explain)
B: Maintenance or Test
C: Refueling
D: Regulatory Restriction
E: Operator Training and License Examination
F: Administrative
G: Operational Error (Explain)
H: Other (Explain)

3
Method:
1: Manual
2: Manual Scram
3: Automatic Scram
4: Other (Explain)

4
Exhibit G - Instructions for preparation of data entry sheets for Licensee Event Report (LER) File (NUREG 0161)

5
Exhibit I: Same Source

DOCKET NO: 50-315
UNIT NAME: D.C. Cook Unit 1
COMPLETED BY: E.C. Schimmel
TELEPHONE: (616) 465-5901
DATE: February 7, 1991
PAGE: 1 of 3

MONTHLY OPERATING ACTIVITIES - January 1991

HIGHLIGHTS

The unit entered the reporting period with the RCS in mode 5. RCS heatup was started on January 4. The RCS temperature was held at approximately 300 degrees Fahrenheit for 48 hours for steam generator crevice flushing. The RCS reached no-load rated temperature and pressure on January 9. The low-power physics testing program was started on January 9, with the reactor being critical for the first time in cycle 12 at 0826 hours on January 15. The low-power physics testing was completed on January 16.

An RCS cooldown to cold shutdown was required due to a weld leak in an instrument line root valve for main steam flow transmitter MFC-120. Mode 5 was reached on January 18. Following completion of the weld repairs, the RCS was again heated up with rated temperature and pressure conditions being reached on January 19.

The reactor was taken critical on January 23 and following turbine roll and completion of required turbine balancing the generator was paralleled to the system grid for the first time in cycle 12 at 2345 hours on January 26, 1991. Reactor power was increased to 32% RTP for turbine rotor warming prior to overspeed testing .

On January 27, reactor power was reduced to 7% and the generator removed from parallel at 1742 hours to perform main turbine overspeed testing. The generator was re-paralleled to the system grid at 1913 hours on January 27 and reactor power increase was commenced in accordance with the power escalation testing program. The unit exited the reporting period at approximately 53% RTP with power being increased to 68%.

Gross electrical generation for the month of January was 43350 MWH.

DETAILS

01/04/91	1041	Commenced RCS heatup to mode 4.
	1054	The RCS is in mode 4.
01/07/91	2209	Commenced RCS heatup to mode 3.
	2310	The RCS is in mode 3.

01/14/91	2355	Commenced reactor startup.
01/15/91	0537	The reactor is in mode 2.
	0826	The reactor is critical.
01/16/91	0933	Commenced reactor shutdown.
	0946	The RCS is in mode 3.
	1900	Commenced RCS cooldown to mode 5 to allow repair of a weld leak in an instrument line root valve for main stem flow transmitter MFC-120.
	2324	The RCS is in mode 4.
01/18/91	0130	The RCS is in mode 5.
	1459	With repairs complete, commenced an RCS heatup to mode 3.
	1526	The RCS is in mode 4.
01/19/91	0228	The RCS is in mode 3.
01/23/91	2209	Commenced reactor startup.
	2345	The reactor is critical.
01/24/91	0425	Commenced reactor power increase to 7.5% RTP for turbine roll.
	1223	The reactor entered mode 1.
	1317	The reactor is at 7.5% RTP.
	1320	Rolled main turbine.
	1621	Manually tripped main turbine due to high vibrations.
01/26/91	0250	Commenced reactor power decrease to 2% RTP while balancing the main turbine.
	0405	The reactor is in mode 2.
	0515	The reactor is at 2% RTP.

	1554	Commenced reactor power increase to 7% RTP.
	1603	The reactor is in mode 1.
	1626	The reactor is at 7% RTP.
	2345	The main generator is paralleled to the system grid. Commenced unit power increase to 32% RTP.
01/27/91	0400	The unit is at 32% RTP.
	1520	Commenced unit power reduction to 7% RTP to perform main turbine overspeed trip testing.
	1735	The unit is at 7% RTP.
	1742	The main generator output breakers are opened. Commenced turbine overspeed testing.
	1913	Turbine overspeed testing is complete. The main generator is paralleled to the system power grid. Commenced unit power increase to 35% RTP.
	2209	The unit is at 35% RTP.
01/29/91	0810	Commenced unit power increase to 48% RTP.
	2038	The unit is at 48% RTP.
01/31/91	1350	Commenced unit power increase to 68% RTP

DOCKET NO: 50-315
UNIT NAME: D.C. Cook Unit 1
COMPLETED BY: E.C. Schimmel
TELEPHONE: (616) 465-5901
DATE: February 7, 1991
PAGE: 1 of 1

MAJOR SAFETY-RELATED MAINTENANCE - January 1991

1-M-1 The interlocks on the 612'containment airlock doors were not functioning, due to a broken bracket on the outer airlock door. The bracket was re-welded and the interlocks restored to operability.

1-M-2 An elbow on the instrument line between #12 Steam Generator and instrument valve 1-MFC-120-V1 was leaking steam. A leaking elbow in the line was re-welded to repair the leak.