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 FACIL:50-315 Donald C. Cook Nuclear Power Plant, Unit 1, Indiana & 05000315
 AUTH.NAME AUTHOR AFFILIATION
 DROSTE,J.B. Indiana Michigan Power Co. (formerly Indiana & Michigan Ele
 BLIND,A.A. Indiana Michigan Power Co. (formerly Indiana & Michigan Ele
 RECIP.NAME RECIPIENT AFFILIATION

SUBJECT: LER 90-015-00:on 901102,containment Type B & C leakage
 exceeded LCO due to degradation of isolation valve seats:

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	RGN3 FILE 01	1 1		
EXTERNAL:	EG&G BRYCE,J.H	3 3	L ST LOBBY WARD	1 1
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November 21, 1990

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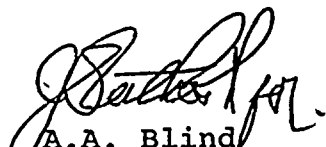
Operating Licenses DPR-58
Docket No. 50-315

Document Control Manager:

In accordance with the criteria established by 10 CFR 50.73
entitled Licensee Event Reporting System, the following
report is being submitted:

90-015-00

Sincerely,


A.A. Blind
Plant Manager

AAB:clj

Attachment

cc: D.H. Williams, Jr.
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LICENSEE EVENT REPORT (LER)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-630), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1) D. C. COOK NUCLEAR PLANT, - UNIT 1										DOCKET NUMBER (2) 0 1 5 0 0 0 3 1 5										PAGE (3) 1 OF 0 1																															
TITLE (4) Containment Type B and C Leakage Exceeds L.C.O. Value Due to Degradation of Isolation Valve Seating Services																																																			
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)																																									
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES						DOCKET NUMBER(S)																																				
1	1	0	2	9	0	9	0	0	1	5	0	0	1	1	2	1	9	0	0 5 0																																
OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)																																																	
5		20.402(b)										20.405(c)										50.73(a)(2)(iv)										73.71(b)																			
POWER LEVEL (10)		0 0 0										20.405(a)(1)(i)										50.36(c)(1)										50.73(a)(2)(v)										73.71(c)									
		20.405(a)(1)(ii)										50.36(c)(2)										50.73(a)(2)(vi)										OTHER (Specify in Abstract below and in Text, NRC Form 366A)																			
		20.405(a)(1)(iii)										X 50.73(a)(2)(i)										50.73(a)(2)(vii)(A)																													
		20.405(a)(1)(iv)										50.73(a)(2)(ii)										50.73(a)(2)(viii)(B)																													
		20.405(a)(1)(v)										50.73(a)(2)(iii)										50.73(a)(2)(ix)																													
LICENSEE CONTACT FOR THIS LER (12)																																																			
NAME J.B. Droste - Plant Engineering Superintendent																TELEPHONE NUMBER AREA CODE 6 1 6 4 6 5 1 5 9 0 1																																			
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																																																			
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC																													
SUPPLEMENTAL REPORT EXPECTED (14)																EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR																															
X YES (If yes, complete EXPECTED SUBMISSION DATE)																NO		0	2	1	5	9	1																												
ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)																																																			

On November 2, 1990, with the Reactor Coolant System in Mode 5 (Cold Shutdown), the accumulated leakage found while performing the Type B and C Leak Rate Tests on Containment penetrations exceeded the L.C.O. value (0.60 Ia) of Technical Specification 3.6.1.2.b., using the maximum pathway methodology.

Three valves have exhibited excessive leakage rates and are the cause for exceeding the Technical Specification Limit: 1-CS-442-1 and 1-CS-442-3 (EIIIS:ISV/BD) are Containment Isolation Valves for the Seal Water Injection Lines to Reactor Coolant Pumps Nos. 11 and 13; The weld pressurization system valve enclosure manway for 1-ICM-305 and 1-ICM-306 (EIIIS-ISV/BD).

Those Containment Isolation Valves that exhibit leak rates in excess of the acceptance criteria are being repaired and retested to ensure the leak rates are within allowable limits. The B and C Leak Rate Testing is 91 percent complete and will not be completed until the end of the Refueling Outage.

This is an interim report. The final report will be submitted by February 15, 1990, following completion of the B and C Leak Rate Tests.

