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 FACIL: 50-315 Donald C. Cook Nuclear Power Plant, Unit 1, Indiana & 05000315
 50-316 Donald C. Cook Nuclear Power Plant, Unit 2, Indiana & 05000316
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 RECIP. NAME RECIPIENT AFFILIATION
 MURLEY, T.E. Document Control Branch (Document Control Desk)

SUBJECT: Submits Reg Guide 1.97 status update, per 901005 ltr.

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AEP:NRG:0773AJ

Donald C. Cook Nuclear Plant Units 1 and 2
Docket Nos. 50-315 and 50-316
License Nos. DPR-58 and DPR-74
REGULATORY GUIDE 1.97 STATUS UPDATE

U.S. Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, D.C. 20555

Attn: T. E. Murley

November 9, 1990

Dear Dr. Murley:

The purpose of this letter is to update the status of each variable listed in our Regulatory Guide 1.97 submittal AEP:NRG:0773AB dated October 5, 1988. We have completed all of the upgrade work shown in the table portion of our AEP:NRG:0773AB submittal with the following exceptions:

- 1) Items B-04 and B-05: The reactor coolant system hot and cold leg water temperature work on Unit 1 is expected to be completed during the present refueling outage. This work was not completed on schedule due to the need for a subsequent request for an Appendix R exemption. This Appendix R exemption was granted on February 9, 1990. The work on Unit 2 has been completed.
- 2) Item B-9: The coolant inventory (reactor vessel water level) work described in AEP:NRG:0773AB involves upgrading the reactor coolant system hot leg temperature inputs. This work was not completed as scheduled because of the above (item B-04) mentioned delays. The work on Unit 1 is expected to be completed during the present Unit 1 refueling outage. The work on Unit 2 has been completed.
- 3) Item E-5b: The portable radiation monitoring instruments have been calibrated and are available for use in the range of 10^{-3} to 10^{+4} R/hr for gamma radiation. Appropriate correction factors for beta/low energy gamma dose rates will be determined by December 1, 1990.

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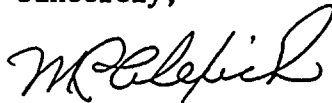
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- 4) Item E-5c: The plant and environs radioactivity variable (portable gamma spectroscopy system) is shown in AEP:NRC:0773AB as being completed. However, the procedure for using this equipment and its final calibration is not expected to be completed until April 1, 1991.
- 5) Item B-14: As discussed with T. Colburn of your staff on September 20, 1990, 32 containment isolation valve position indication switches on each unit lacked the documentation required to verify their seismic adequacy. These position indication switches will be seismically tested and the results of this testing are expected within one year. At that time we should be able to verify their seismic adequacy. If the seismic testing shows these switches to be inadequate, new seismically qualified position indicating switches will be purchased and installed. These position indication switches are located on small sized sample lines and have no impact on the ability of the containment isolation valves to perform their intended safety function.

This document has been prepared following Corporate procedures that incorporate a reasonable set of controls to ensure its adequacy and completeness prior to signature by the undersigned.

Sincerely,



M. P. Alexich
Vice President

ldp

cc: D. H. Williams, Jr.
A. A. Blind - Bridgman
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