

# ACCELERATED DISTRIBUTION DEMONSTRATION SYSTEM

## REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR:9011070104 DOC.DATE: 90/11/02 NOTARIZED: NO DOCKET #  
 FACIL:50-315 Donald C. Cook Nuclear Power Plant, Unit 1, Indiana & 05000315  
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SUBJECT: LER 90-011-00:on 901004,steam generator pressure channel  
 check surveillance missed.W/901102 ltr.

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EXTERNAL:	EG&G BRYCE,J.H	3	3	L ST LOBBY WARD	1	1
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November 2, 1990

United States Nuclear Regulatory Commission  
Document Control Desk  
Rockville, Maryland 20852


Operating Licenses DPR-58  
Docket No. 50-315

Document Control Manager:

In accordance with the criteria established by 10 CFR 50.73  
entitled Licensee Event Reporting System, the following  
report is being submitted:

90-011-00

Sincerely,

  
A.A. Blind  
Plant Manager

AAB:clj

Attachment

cc: D.H. Williams, Jr.  
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9011070104 901102  
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EXPIRES: 4/30/92

## LICENSEE EVENT REPORT (LER)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1) D. C. Cook Nuclear Plant, Unit 1										DOCKET NUMBER (2) 0 5 0 0 0 3 1 1 5										PAGE (3) 1 OF 0 4				
TITLE (4) Missed Steam Generator Pressure Channel Check Surveillance due to Personnel Error																								
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)														
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES			DOCKET NUMBER(S)												
1	0	0	4	9	0	9	0	1	1	0	0	1	1	0	2	9	0	0	5	0	0	0	0	0
OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)																						
1		20.402(b)				20.405(c)				50.73(a)(2)(iv)				73.71(b)										
POWER LEVEL (10)		20.405(a)(1)(i)				50.38(c)(1)				50.73(a)(2)(v)				73.71(c)										
0 8 6		20.405(a)(1)(ii)				50.38(c)(2)				50.73(a)(2)(vii)				OTHER (Specify in Abstract below and in Text, NRC Form 366A)										
		20.405(a)(1)(iii)				X 50.73(a)(2)(i)				50.73(a)(2)(viii)(A)														
		20.405(a)(1)(iv)				50.73(a)(2)(ii)				50.73(a)(2)(viii)(B)														
		20.405(a)(1)(v)				50.73(a)(2)(iii)				50.73(a)(2)(x)														
LICENSEE CONTACT FOR THIS LER (12)																								
NAME												TELEPHONE NUMBER												
J. R. Sampson, Operations Superintendent												6 1 6 4 6 5 1 - 5 9 0 1												
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																								
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC														
SUPPLEMENTAL REPORT EXPECTED (14)												EXPECTED SUBMISSION DATE (15)				MONTH	DAY	YEAR						
YES (If yes, complete EXPECTED SUBMISSION DATE)												X NO												

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single space typewritten lines) (16)

On October 4, 1990, an Operator conducting the shiftly surveillance incorrectly logged the steam generator levels in the steam generator pressure channel section of the surveillance for the loop 3 and loop 4 channels. The review of the surveillance by the Operator and the Unit Supervisor did not identify the error. As a result of the failure of the Operator and the Unit Supervisor to recognize the logging error, the Technical Specification requirement to perform a channel check of the redundant steam generator pressure channels was not completed.

The cause of this event was personnel error. The involved Operator and Unit Supervisor failed to confirm the proper surveillance readings during their review to ensure the Technical Specification surveillance requirements were satisfied.

When the error was discovered, a channel check of the redundant steam generator pressure channels was completed. This channel check confirmed that the redundant channels of the affected S/G pressures were within required limits during the time of the surveillance.

LICENSEE EVENT REPORT (LER)  
TEXT CONTINUATION

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
D. C. Cook Nuclear Plant, Unit 1	0 5 0 0 0 3 1 5	9 0	— 0 1 1	— 0 0	0 2	OF	0 4

TEXT (If more space is required, use additional NRC Form 366A's) (17)

Condition Prior to Occurrence

Unit 1 in Mode 1 at 86% power.

Unit 2 in Mode 5 at 00% power.

Description of Event

On October 4, 1990, during the 1800 to 0600 shift, the Operator conducting the shiftly surveillance per 1-OHP 4023.STP.030 Data Sheet No. 3 (Shift Surveillance Checks, Mode 1,2,3,4) incorrectly logged the steam generator levels (EIIS/SB-LI) for loop 3 and loop 4 in the steam generator pressure (EIIS/SB-PI) channel section of the data sheet.

At about 0330 on October 4, 1990, the Unit Supervisor completed a review of the surveillance and signed the surveillance. The Unit Supervisor review did not identify that the steam generator levels had been recorded in the steam generator pressure section of the STP.030 Data Sheet No. 3.

During the following 0600 to 1800 shift on October 4, 1990, the Operator recording the shiftly surveillance noted that the steam generator level channels had been recorded for the steam generator pressure channels. The Operator reported the finding to the Unit Supervisor and completed a satisfactory steam generator pressure channel check on the redundant loop 3 and loop 4 channels.

The incorrect logging of the steam generator level channels in the steam generator pressure channel section of the data sheet led to the failure of the operators to satisfy the Technical Specification 3.3.2.1 (Engineering Safety Feature Actuation System Instrumentation) surveillance requirement to perform a channel check between redundant steam generator pressure channels each shift (12 hours).

Cause of Event

The cause of this event was personnel error. The Operator recording data on the shiftly surveillance data sheet failed to correctly record and review the steam generator pressure channel data.

In addition, the Unit Supervisor completing a review of the surveillance failed to identify the operator error and ensure that the Technical Specification surveillance requirements were satisfied.

LICENSEE EVENT REPORT (LER)  
TEXT CONTINUATION

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FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)				PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER				
D. C. Cook Nuclear Plant, Unit 1	0   5   0   0   0   3   1   5	9   0	—   0   1   1	—   0   0	0   3	OF	0   4	

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Analysis of Event

Technical Specification 3.3.2.1 (Engineering Safety Feature Actuation System Instrumentation) requires a steam generator pressure channel check each shift (12 hours). The failure to perform a steam generator pressure channel check within 12 hours is a violation of Technical Specification 3.3.2.1 and reportable under 10 CFR 50.73 (a)(2)(1)(B).

A review of the previous STP.030 surveillance was completed. The review noted that redundant loop 3 and loop 4 steam generator pressure channel checks were satisfactory. The surveillance was signed off by the Unit Supervisor at 2244 on October 3, 1990.

A review of the next STP.030 surveillance data recorded following the logging error was completed. The review noted that the redundant loop 3 and loop 4 steam generator pressure channel checks were satisfactory. The surveillance was signed off by the Unit Supervisor at 1514 on October 4, 1990.

When the missed steam generator pressure channel check was brought to the attention of the involved operators, a history of the redundant loop 3 and loop 4 steam generator pressure channels was displayed and reviewed on the OTSC (Operational Technical Support Center) computer. The review noted that the redundant loop 3 and loop 4 steam generator pressure channels were less than the maximum differential notification limit for redundant channels during the period of time between the STP.030 data recorded at 2244 on October 3, 1990, and at 1514 on October 4, 1990.

Considering that the loop 3 and loop 4 steam generator pressure channel checks were satisfactory on the STP.030 surveillance before and after the pressure channel recording error, and that the OTSC computer history showed that the maximum differential between the redundant pressure channels was less than the notification limit, it is concluded that the differential between the redundant loop 3 and loop 4 steam generator pressure channels was satisfactory during the time the pressure channels were not recorded and reviewed.

Based on the above analysis, the missed steam generator pressure channel check surveillance is not considered to have created a significant safety concern, nor did the missed surveillance create a significant hazard to the health and safety of the general public.



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D. C. Cook Nuclear Plant, Unit 1	0 5 0 0 0 3 1 5	9 0	— 0 1 1	— 0 0	0 4	OF	0 4

TEXT (If more space is required, use additional NRC Form 366A's) (17)

Corrective Action

1. When the steam generator pressure channel surveillance recording error was identified, a channel check between the redundant loop 3 and loop 4 pressure channels was satisfactorily completed.
2. The involved Operator and Unit Supervisor were counselled by their Shift Supervisor.

Failed Component Identification

None

Previous Similar Events

A review of previous License Event Reports was completed to identify similar events where the incorrect recording of surveillance data led to a missed surveillance. A similar event was not identified.



