

## ACCELERATED DISTRIBUTION DEMONSTRATION SYSTEM

## REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR:9008300159 DOC.DATE: 90/08/23 NOTARIZED: NO DOCKET #  
 FACIL:50-315 Donald C. Cook Nuclear Power Plant, Unit 1, Indiana & 05000315  
 AUTH.NAME AUTHOR AFFILIATION  
 BEILMAN,T.P. Indiana Michigan Power Co. (formerly Indiana & Michigan Ele  
 BLIND,A.A. Indiana Michigan Power Co. (formerly Indiana & Michigan Ele  
 RECIP.NAME RECIPIENT AFFILIATION

SUBJECT: LER 90-005-00:on 900724,Tech Spec calibr interval exceeded  
 due to incorrect entry into computerized schedular program.  
 W/9 ltr.

DISTRIBUTION CODE: IE22T COPIES RECEIVED:LTR 1 ENCL 1 SIZE: 4  
 TITLE: 50.73/50.9 Licensee Event Report (LER), Incident Rpt, etc.

## NOTES:

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	PD3-1 LA	1 1	PD3-1 PD	1 1
	COLBURN,T.	1 1		
INTERNAL:	ACNW	2 2	AEOD/DOA	1 1
	AEOD/DSP/TPAB	1 1	AEOD/ROAB/DSP	2 2
	NRR/DET/ECMB 9H	1 1	NRR/DET/EMEB9H3	1 1
	NRR/DLPQ/LHFB11	1 1	NRR/DLPQ/LPEB10	1 1
	NRR/DOEA/OEAB11	1 1	NRR/DREP/PRPB11	2 2
	NRR/DST/SELB 8D	1 1	NRR/DST/SICB 7E	1 1
	NRR/DST/SPLB8D1	1 1	NRR/DST/SRXB 8E	1 1
	REG FILE 02	1 1	RES/DSIR/EIB	1 1
	RGN3 FILE 01	1 1		
EXTERNAL:	EG&G BRYCE,J.H	3 3	L ST LOBBY WARD	1 1
	LPDR	1 1	NRG PDR	1 1
	NSIC MAYS,G	1 1	NSIC MURPHY,G.A	1 1
	NUDOCS FULL TXT	1 1		

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Indiana Michigan  
Power Company  
Cook Nuclear Plant  
P.O. Box 458  
Bridgman, MI 49106  
616 465 5901



August 23, 1990

United States Nuclear Regulatory Commission  
Document Control Desk  
Rockville, Maryland 20852

Operating Licenses DPR-58  
Docket No. 50-315

Document Control Manager:

In accordance with the criteria established by 10 CFR 50.73  
entitled Licensee Event Reporting System, the following  
report is being submitted:

90-005-00

Sincerely,

  
A.A. Blind  
Plant Manager

AAB:clj

Attachment

cc: D.H. Williams, Jr.  
A.B. Davis, Region III  
M.P. Alexich  
P.A. Barrett  
J.E. Borggren  
R.F. Kroeger  
B. Walters - Ft. Wayne  
NRC Resident Inspector  
J.G. Gitter, NRC  
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G. Charnoff, Esq.  
Dottie Sherman, ANI Library  
D. Hahn  
INPO  
S.J. Brewer/B.P. Lauzau

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PDR ADOCK 05000315  
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## LICENSEE EVENT REPORT (LER)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST 500 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P 530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503

FACILITY NAME (1)

D. C. COOK NUCLEAR PLANT - UNIT 1

DOCKET NUMBER (2)

0 5 0 0 0 3 1 5 1 OF 0 3

PAGE (3)

TITLE (4) TECHNICAL SPECIFICATION CALIBRATION INTERVAL EXCEEDED DUE TO INCORRECT ENTRY INTO  
COMPUTERIZED SCHEDULER PROGRAM

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
0 7	2 4	9 0	9 0	0 0 5	0 0	0 8	2 3	9 0		0 5 0 0 0
OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)								
1										
POWER LEVEL (10)	1 0 0	20.402(b)		20.405(c)		50.73(a)(2)(iv)		73.71(b)		
		20.405(a)(1)(i)		50.36(c)(1)		50.73(a)(2)(v)		73.71(c)		
		20.405(a)(1)(ii)		50.36(c)(2)		50.73(a)(2)(vi)		OTHER (Specify in Abstract below and in Text, NRC Form 366A)		
		20.405(a)(1)(iii)		50.73(a)(2)(i)		50.73(a)(2)(viii)(A)				
		20.405(a)(1)(iv)		50.73(a)(2)(ii)		50.73(a)(2)(viii)(B)				
20.405(a)(1)(v)		50.73(a)(2)(iii)		50.73(a)(2)(ix)						

LICENSEE CONTACT FOR THIS LER (12)

NAME

T. P. BEILMAN  
MAINTENANCE DEPARTMENT SUPERINTENDENT

TELEPHONE NUMBER

AREA CODE

6 1 6 4 6 5 5 9 0 1

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFAC- TURER	REPORTABLE TO NPROS	CAUSE	SYSTEM	COMPONENT	MANUFAC- TURER	REPORTABLE TO NPROS

SUPPLEMENTAL REPORT EXPECTED (14)

EXPECTED  
SUBMISSION  
DATE (15)

MONTH DAY YEAR

YES (If yes complete EXPECTED SUBMISSION DATE)

X NO

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

During a routine QA audit completed on July 24, 1990, it was discovered that the Technical Specification T/S time limit of 18 months, plus 25 percent grace period had been exceeded by 28 days between July 7, 1986 and June 20, 1988 for calibration of the Auxiliary Building Ventilation System Unit Vent Sampler Flow Rate Measuring Device (VFS-1521).

Prior to becoming overdue, the scheduler identified the need for recalibration, but did not adequately identify it as being required to satisfy a T/S requirement. Although the exact reason for delay of the calibration could not be determined, it is believed to have been allowed due to review of the scheduler-supplied information. Once calibrated, scheduler updating would have noted the missed due date, but not the relationship to T/S compliance.

The calibration was completed on June 20, 1988. No adverse conditions had resulted due to the extended interval as the instrument was found within acceptable tolerances. During review of the scheduler program for similar problems, the equivalent instrument for the Unit 2 vent system, VFS-2521, was also found to be inadequately identified to T/S requirements, but the calibrations had been done on time. The scheduler was reprogrammed.

LICENSEE EVENT REPORT (LER)  
TEXT CONTINUATION

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST 500 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)

DOCKET NUMBER (2)

LER NUMBER (6)

PAGE (3)

D. C. COOK NUCLEAR PLANT - UNIT ONE

0 5 0 0 0 3 1 5

YEAR	SEQUENTIAL NUMBER	REVISION NUMBER
9 0	- 0 0 5	- 0 0

0 2 OF 0 3

TEXT (If more space is required, use additional NRC Form 366A's) (17)

Conditions Prior to Occurrence

Unit One in Mode 1 operating at 100 percent reactor thermal power.

Description of Event

During a routine QA audit completed on July 24, 1990 at approximately 1500 hours, it was discovered that the grace period requirement of twenty-five percent of the 18-month surveillance interval for Technical Specification 4.3.3.10.2, Table 4.3-9 Item 3e was exceeded between July 7, 1986 and July 20, 1988 for calibration of the Auxiliary Building Ventilation System Unit Vent Sampler Flow Rate Measuring Device, VFS-1521 (EIIS/IL-FI).

There were no inoperable systems, components or structures which contributed to this event.

Cause of Event

Plant surveillance scheduling is accomplished via a computer (EIIS/CMP) software package called Nuclear Test Scheduler (NTS) and is located on the corporate mainframe. The computer scheduler initiates a report on individual items becoming due at a time sufficient to allow planning and performance of the surveillance prior to exceeding the allowable time interval, less the grace period. At the time this event occurred, NTS tracked both Technical Specification (T/S) requirements as well as non-T/S (Preventive Maintenance) items.

Prior to the event, VRS-1521 was initially added to the NTS non-T/S report as it was not required by T/S at that time. T/S was later changed to add VFS-1521, but it did not get transferred to the T/S report. When it was due for calibration, a report was made and it appeared to be non-T/S. The significance of a subsequent delay in calibration was not noted and VFS-1521 was allowed to exceed its allowable time limit. Once calibrated, the computer scheduler noted that the due date had been exceeded, but the failure to satisfy the T/S requirement was not noted again due to VFS-1521 appearing to be a non-T/S item. The reason for delay in calibration could not be determined due to the time elapsed since the event.

Analysis of Event

This event is being submitted under 10CFR.73(a)(2)(i) as a condition prohibited by the plant's T/S 4.0.2a. The calibration was completed 28 days after the grace period had expired. The results of the calibration had shown no adverse affects to the instrument due to the extended time interval and that the instrument remained within specifications during this time frame. Based on the above, it has been concluded that the event had no impact on public health and safety.



LICENSEE EVENT REPORT (LER)  
TEXT CONTINUATION

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FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)	PAGE (3)														
D. C. COOK NUCLEAR PLANT - UNIT ONE		<table border="1"><tr><th data-bbox="1093 264 1169 298">YEAR</th><th data-bbox="1172 264 1205 298"></th><th data-bbox="1209 264 1328 298">SEQUENTIAL NUMBER</th><th data-bbox="1331 264 1364 298"></th><th data-bbox="1367 264 1433 298">REVISION NUMBER</th></tr><tr><td data-bbox="1093 302 1169 336">90</td><td data-bbox="1172 302 1205 336">-</td><td data-bbox="1209 302 1328 336">005</td><td data-bbox="1331 302 1364 336">-</td><td data-bbox="1367 302 1433 336">00</td></tr></table>	YEAR		SEQUENTIAL NUMBER		REVISION NUMBER	90	-	005	-	00	<table border="1"><tr><td data-bbox="1437 264 1513 298">0</td><td data-bbox="1516 264 1549 298">3</td><td data-bbox="1552 264 1585 298">OF</td><td data-bbox="1589 264 1625 298">03</td></tr></table>	0	3	OF	03
YEAR		SEQUENTIAL NUMBER		REVISION NUMBER													
90	-	005	-	00													
0	3	OF	03														
	05000315																

TEXT (If more space is required, use additional NRC Form 366A's) (17)

Corrective Action

The calibration was completed on June 20, 1988.

On May 6, 1989, during a Preventive Maintenance upgrade program review, VFS-1521 was transferred to the NTS T/S report. The associated Unit 2 instrument, VFS-2521 was also found on the non-T/S report and was transferred, but calibration history shows the T/S requirement to have been met. A recent review performed as a result of this QA audit found no additional discrepancies and have determined that the proper controls are in place to prevent recurrence.

Failed Component Identification

None.

Previous Similar Events

None.

