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 MURLEY, T.E. Document Control Branch (Document Control Desk)

SUBJECT: Submits info to closeout corrective actions for SPDS, in
 response to NRC 900720 ltr.

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Figure 6



AEP:NRG:0773AI

Donald C. Cook Nuclear Plant Units 1 and 2
Docket Nos. 50-315 and 50-316
License Nos. DPR-58 and DPR-74
COOK NUCLEAR PLANT SAFETY PARAMETER DISPLAY
SYSTEM (SPDS) CORRECTIVE ACTION CLOSE OUT

U. S. Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, D. C. 20555

Attn: T. E. Murley

August 17, 1990

Dear Dr. Murley:

This letter responds to your July 20, 1990, letter and provides confirmation that corrective action in response to the findings resulting from the on-site audit of the SPDS has been taken and the items are closed. The specific findings and our confirmation of close-out are given below.

SPDS Is Not Continuous

- o The SPDS monitor does not provide any visual or audible cues to alert operators to a change in status for the five plant-specific safety functions on displays other than the two, top level iconic displays.

Response

The modification to limit the keyboard functions available on one of the control room TSC computer terminals and provide a dedicated display of the SPDS iconic was completed on July 2, 1990, for both Units 1 and 2. The software modification prevents a user from accessing any displays other than the iconic displays from the SPDS-dedicated terminal. A long range solution is also being investigated that would permit a user to access other displays on the SPDS dedicated terminal while still being aware if any adverse changes to the ICONIC Displays' process variables. If this alternative solution is found to be practical it will be implemented.

SPDS Is Not Designed to Incorporate Accepted Human Factors Principles

- o Numerical values for the high and low level alarm setpoints

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on each safety function "spoke" of the narrow- and wide-range iconic displays were not identified. Some alarm setpoints were reactor trip values and others were either anticipatory values or system capacity values. In addition, the Unit 2 SPDS steam generator low level reactor trip setpoint, 21%, was incorrectly set at the Unit 1 value of 17%. In another case, the Unit 1 SPDS high Tavg alarm setpoint on the SPDS had not been changed to reflect the amended Technical Specification value.

Response

The apparent disparity in the selection of reactor trip values, system capacity values, or anticipatory values has been rectified.

The narrow-range steam generator level low limit on the Unit 2 narrow-range iconic display was changed to the correct reactor trip setpoint. The RCS average temperature high limit on the Unit 1 narrow-range iconic display has been changed to the correct value corresponding to the amended Technical Specification limiting condition for operation requirement.

Operator Training With SPDS Was Not Satisfactory

- o Control room did not know, in all instances, what the numerical values were for the high and low level alarm setpoints for each safety function on the SPDS narrow- and wide-range iconic displays.

Response

Numerical values for the alarm limits on the iconic display have been incorporated into the Plant Operators technical data reference. The general philosophy on how these values were selected was incorporated into the operator requalification training beginning in the third quarter of 1990.

This document has been prepared following Corporate procedures that incorporate a reasonable set of controls to ensure its accuracy and completeness prior to signature by the undersigned.

Sincerely,



M. P. Alexich
Vice President

Dr. T. E. Murley

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AEP:NRC:0773AI

eh

cc: D. H. Williams, Jr.
A. A. Blind - Bridgman
J. R. Padgett
G. Charnoff
NFEM Section Chief
A. B. Davis - Region III
NRC Resident Inspector - Bridgman



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