

ACCELERATED DISTRIBUTION DEMONSTRATION SYSTEM

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR:9006280322 DOC.DATE: 90/06/25 NOTARIZED: NO DOCKET #
 FACIL:50-315 Donald C. Cook Nuclear Power Plant, Unit 1, Indiana & 05000315
 50-316 Donald C. Cook Nuclear Power Plant, Unit 2, Indiana & 05000316
 AUTH.NAME AUTHOR AFFILIATION
 ALEXICH,M.P. Indiana Michigan Power Co. (formerly Indiana & Michigan Ele
 RECIP.NAME RECIPIENT AFFILIATION
 Document Control Branch (Document Control Desk)

SUBJECT: Forwards response to Generic Ltr 90-04 re closeout of generic safety issues.

DISTRIBUTION CODE: A018D COPIES RECEIVED:LTR 1 ENCL 1 SIZE: 8
 TITLE: Generic Ltr 90-04 re Status of Licensee Implementation of Generic Saf

NOTES:

	RECIPIENT		COPIES			RECIPIENT		COPIES	
	ID CODE/NAME		LTTR	ENCL		ID CODE/NAME		LTTR	ENCL
	PD3-1 LA		1	1		PD3-1 PD		1	1
	GIITTER,J.		2	2					
INTERNAL:	CROCKER,L 9H3		1	1		ECCLESTON,K10D4		1	1
	NUDOCS-ABSTRACT		1	1		REG FILE 01		1	1
	SCHOLL,R 13E4		1	1					
EXTERNAL:	LPDR		1	1		NRC PDR		1	1

NOTE TO ALL "RIDS" RECIPIENTS:

PLEASE HELP US TO REDUCE WASTE! CONTACT THE DOCUMENT CONTROL DESK,
 ROOM P1-37 (EXT. 20079) TO ELIMINATE YOUR NAME FROM DISTRIBUTION
 LISTS FOR DOCUMENTS YOU DON'T NEED!

TOTAL NUMBER OF COPIES REQUIRED: LTTR 11 ENCL 11



AEP:NRC:1108C

Donald C. Cook Nuclear Plant Units 1 and 2
Docket Nos. 50-315 and 50-316
License Nos. DPR-58 and DPR-74
GENERIC SAFETY ISSUES
RESPONSE TO GENERIC LETTER 90-04

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D. C. 20555

Attn: T. E. Murley

June 25, 1990

Dear Dr. Murley:

This letter is in response to Generic Letter (GL) 90-04 regarding close-out of generic safety issues. We support your concern and desire to close long-standing generic safety issues.

Attachment 1 to this letter is a copy of Enclosure 1 to GL 90-04 with our responses identified in the "STATUS" column. We have used the "COMMENTS" column to identify the safety evaluation report (SER) or license amendment that, to our understanding, indicates that the issue is closed. Comments with "NRC ACTION" annotated means that, to our knowledge, we have not received a response to our latest submittals on that safety issue. The AEP:NRC: submittal numbers and dates reflected in the "STATUS" column are provided for reference and to the best of our knowledge reflect the complete list of submittals on a given topic.

We have responded to each issue noted in the enclosure to GL 90-04 except for the most recent generic letter (GL 90-03), which is an iteration of Item 2.2 of GL 83-28. Our response, which will slightly modify our original position, will be submitted in the near future.

9006280322 900625
PDR ADOCK 05000315
P PDC

A018
11

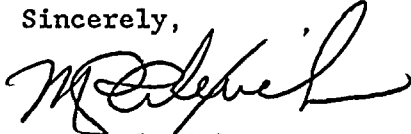
8

Dr. T. E. Murley

-2-

This document has been prepared following Corporate procedures that incorporate a reasonable set of controls to ensure its accuracy and completeness prior to signature by the undersigned.

Sincerely,

A handwritten signature in dark ink, appearing to read 'M. P. Alexich', written in a cursive style.

M. P. Alexich
Vice President

cc: D. H. Williams, Jr.
A. A. Blind - Bridgman
J. P. Padgett
NFEM Section Chief
A. B. Davis - Region III
NRC Resident Inspector - Bridgman

FACILITY NAME: COOK NUCLEAR PLANT U1 AND U2
 DOCKET NO.: 50-315 and 50-316
 LICENSEE: DPR-58 and DPR-74

STATUS OF LICENSEE IMPLEMENTATION OF GENERIC SAFETY ISSUES

RESOLVED WITH IMPOSITION OF REQUIREMENTS OR CORRECTIVE ACTIONS

(Information is identical for Units 1 and 2 except as noted with Amendment No.)

<u>GSI/(MPA No.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS*</u>	<u>COMMENTS</u>
40 (B065)	Safety Concerns Associated With Pipe Breaks in the BWR Scram System	All BWRs	NA	
41 (B058)	BWR Scram Discharge Volume Systems	All BWRs	NA	
43 (B107)	Reliability of Air Systems	All Plants	C (AEP:NRC:1075-2/24/89) C (AEP:NRC:1075A-3/23/89)	NRC ACTION
51 (L913)	Improving the Reliability of Open-Cycle Service Water Systems	All Plants	C (AEP:NRC:0553-5/28/81) C (AEP:NRC:1104-1/25/90)	NRC ACTION
67.3.3 (A017)	Improved Accident Monitoring	All Plants	C (AEP:NRC:0773-4/15/83) C (AEP:NRC:0773J-2/28/85) C (AEP:NRC:0773O-10/15/85) C (AEP:NRC:0773P-10/1/85) C (AEP:NRC:0773S-6/29/87) C (AEP:NRC:0773AB-10/15/88) C (AEP:NRC:0773AC-6/20/88) C (AEP:NRC:0773AG-10/16/89)	NRC ACTION

*Please follow attached guidance for completing this column.

**The 16 items listed for GSI 75 all relate to actions derived from the generic implications of Salem ATWS events. Item numbers correspond to Generic Letter 83-28 action item numbers.

<u>GSI/(MPA No.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS*</u>	<u>COMMENTS</u>
75** (B076)	Item 1.1 - Post-Trip Review (Program Description and Procedure)	All Plants	C (AEP:NRC:0838B-3/30/84)	NRC ACTION
75 (B085)	Item 1.2 - Post-Trip Review Data and Information Capability	All Plants	C (AEP:NRC:0838S-12/17/86)	NRC ACTION
75 (B077)	Item 2.1 - Equipment Classification and Vendor Interface (Reactor Trip System Components)	All Plants	C (AEP:NRC:0838H-4/10/85)	SERs 3/27/87 4/2/87
75 (B086)	Item 2.2.1 - Equipment Classification for Safety-Related Components	All Plants	C (AEP:NRC:0838B-3/30/84) C (AEP:NRC:0838W-12/19/86)	SER 8/13/87
75 (L003)	Item 2.2.2 - Vendor Interface for Safety-Related Components	All Plants	C (AEP:NRC:0838U-1/5/86) C (AEP:NRC:0838AB-6/16/87) [AEP:NRC:0838AH has been initiated to answer GL 90-03 on the same subject]	NRC ACTION
75 (B078)	Items 3.1.1 & 3.1.2 - Post-Maintenance Testing (Reactor Trip System Components)	All Plants	C (AEP:NRC:0838O-8/20/85)	SER 6/27/85
75 (B079)	Item 3.1.3 - Post-Maintenance Testing - Changes to Test Requirements (Reactor Trip System Components)	All Plants	C (AEP:NRC:0838O-8/20/85)	SER 5/20/86
75 (B087)	Items 3.2.1 & 3.2.2 - Post-Maintenance Testing (All Other Safety-Related Components)	All Plants	C (AEP:NRC:0838B-3/30/84) C (AEP:NRC:0838K-4/12/87)	SER 4/27/87

<u>GSI/(MPA No.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS*</u>	<u>COMMENTS</u>
75 (B088)	Item 3.2.3 - Post-Maintenance Testing - Changes to Test Requirements (All Other Safety-Related Components)	All Plants	C (AEP:NRC:0838H-4/10/85) C (AEP:NRC:0838P-12/9/85)	SER 5/20/86
75 (B080)	Item 4.1 - Reactor Trip System Reliability (Vendor-Related Modifications)	All Plants	C (AEP:NRC:0838O-8/20/85)	SER 6/27/85
75 (B081)	Items 4.2.1 & 4.2.2 - Reactor Trip System Reliability - Maintenance and Testing (Preventative Maintenance and Surveillance Program for Reactor Trip Breakers)	All PWRs	C (AEP:NRC:0838A-11/4/83) C (AEP:NRC:0838G-3/29/85) C (AEP:NRC:0838Q-4/22/86) C (AEP:NRC:0838X-5/5/87)	SER 5/23/85 NRC ACTION [ITEMS 4.2.3 & 4.2.4]
75 (B082)	Item 4.3 - Reactor Trip System Reliability - Design Modifications (Automatic Actuation of Shunt Trip Attachment for Westinghouse and B&W Plants)	All W and B&W Plants	C (AEP:NRC:0838E-12/21/84)	NRC ACTION
75 (B090)	Item 4.3 - Reactor Trip System Reliability - Tech Spec Changes (Automatic Actuation of Shunt Trip Attachment for Westinghouse and B&W Plants)	All W and B&W Plants	C (AEP:NRC:0895B-5/10/85) C (AEP:NRC:0895E-6/20/85)	T/S AMEND. U1 #86 U2 #72
75 (B091)	Item 4.4 - Reactor Trip System Reliability (Improvements in Maintenance and Test Procedures for B&W Plants)	All B&W Plants	NA	

<u>GSI/(MPA No.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS*</u>	<u>COMMENTS</u>
75 (B092)	Item 4.5.1 - Reactor Trip System Reliability - Diverse Trip Features (System Functional Testing)	All Plants	C (AEP:NRC:0838A-11/4/83) C (AEP:NRC:0838D-10/24/85) C (AEP:NRC:0838Q-8/20/85) C (AEP:NRC:0838V-11/7/86) C (AEP:NRC:0838Z-6/25/87) C (AEP:NRC:0838AA-10/28/87) C (AEP:NRC:0838AD-12/18/87) C (AEP:NRC:0838AE-3/22/88) C (AEP:NRC:0838AF-5/2/88)	SER 6/27/85 NRC IEIR 50-316/316; 89032, 89032
[THESE SUBMITTALS ALSO CLOSE OUR RESPONSIBILITY UNDER 10CFR50.62, ISSUED ON THE SAME SUBJECT]				
75 (B093)	Items 4.5.2 & 4.5.3 - Reactor Trip System Reliability - Test Alternatives and Intervals (System Functional Testing)	All Plants	C (AEP:NRC:0838A-11/4/83) C (AEP:NRC:0838H-4/10/85)	SERs 3/3/87 8/16/89
86 (B084)	Long Range Plan for Dealing with Stress Corrosion Cracking in BWR Piping	All BWRs	NA	
93 (B098)	Steam Binding of Auxiliary Feedwater Pumps	All PWRs	C (AEP:NRC:1047-5/31/88)	SERs 2/17/88 7/6/88
99 (L817)	RCS/RHR Suction Line Valve Interlock on PWRs	All PWRs	C (AEP:NRC:1033-11/20/87) C (AEP:NRC:1033A-10/21/87) C (AEP:NRC:1033B-1/9/89)	SER 5/21/90
124	Auxiliary Feedwater System Reliability	ANO-1&2, Rancho Seco, Prairie Island 1&2, Crystal River-3 Ft. Calhoun	NA	

<u>GSI/(MPA No.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS*</u>	<u>COMMENTS</u>
A-13 (B017)	Snubber Operability Assurance - Hydraulic Snubbers	All Plants	AEP Ltrs. 3/22 & 8/7/77 C (AEP:NRC:0504-11/11/81) C (AEP:NRC:0931-8/19/86)	T/S AMEND U1#20-8/5/77 U1#104 U2#91 4/13/87
A-13 (B022)	Snubber Operability Assurance - Mechanical Snubbers	All Plants	NA-No Mechanical Snubbers in Safety-Related Systems	
A-16 (D012)	Steam Effects on BWR Core Spray Distribution	Oyster Creek & NMP-1	NA	
A-35 (B023)	Adequacy of Offsite Power Systems	All Plants	AEP Ltr. 6/22/77 C (AEP:NRC:0268-12/17/79) C (AEP:NRC:0313-2/22/80) C (AEP:NRC:0268B-5/28/80) C (AEP:NRC:0591-1/22/82) C (AEP:NRC:1063-11/29/88)	T/S AMENDS U1#39-7/25/80 U2-7/10/80 SER 9/10/80 6/1/81 T/S AMENDS U1#76 U2#57 11/22/83
B-10	Behavior of BWR Mark III Containments	All BWR Mark III Plants	NA	
B-36	Develop Design, Testing and Maintenance Criteria for Atmosphere Cleanup System Air Filtration and Adsorption Units for Engineered Safety Features Systems and for Normal Ventilation Systems	All Plants with OL Applications After 4/1/80	NA	



10-11-68

<u>GSI/(MPA No.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS*</u>	<u>COMMENTS</u>
B-63 (B045)	Isolation of Low Pressure Systems Connected to the Reactor Coolant System Pressure Boundary	All Plants	C (AEP:NRC:0560-2/23/80)	T/S UN- NUMBERED ORDER DATED 4/20/81