Indiana Michigan
Power Company
P.O. Box 16631
Columbus, OH 43216

AEP
INDIANA
MICHIGAN
POWER

AEP:NRC:1108

Donald C. Cook Nuclear Plants Units 1 and 2
Docket Nos. 50-315 and 50-316
License Nos. DPR-58 and DPR-74
NRC GENERIC LETTER 89-21: REQUEST FOR
INFORMATION CONCERNING STATUS OF IMPLEMENTATION
OF UNRESOLVED SAFETY ISSUE (USI) REQUIREMENTS

U. S. Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, D. C. 20555

Attn: T. E. Murley

November 29, 1989

Dear Dr. Murley:

This letter and its attachment are in response to NRC Generic Letter No. 89-21 entitled "Request for Information Concerning Status of Implementation of Unresolved Safety Issue (USI) Requirements," dated October 19, 1989. The status of unresolved safety issues for Cook Nuclear Plant is given in the attached table. Several issues require discussion, which is given below.

1) USI A-17: System Interactions

Generic Letter (GL) 89-18 dated September 6, 1989 was sent to all power reactor licensees and constitutes the resolution of USI A-17. The GL did not require any response/action on the part of the licensees. However, GL 89-18 is currently under review within AEPSC. It is acknowledged that the Individual Plant Examinations (IPE) will include some consideration of system interaction during plant walkdowns. Our submittal AEP:NRC:1082 dated October 24, 1989, "Individual Plant Examination Program Plan, Response to GL No. 88-20, Supplement No. 1," documents the IPE program for the Donald C. Cook Nuclear Plant. During the proposed walkdown in support of the IPE program, issues such as flooding from the internal sources will be considered.

8912070397 7pp.

2) USI A-26: Reactor Vessel Pressure Transient Protection

Generic Letter 88-11 dated July 12, 1988 was sent to all power reactor licensees for review and response. We responded to the GL in our submittal AEP:NRC:0894K dated December 5, 1988. Plant-specific evaluation and the associated technical specification change request for Cook Nuclear Plant Unit 2 were submitted to the NRC for approval in our submittal AEP:NRC:0894L dated October 25, 1989. For Unit 1, the latest surveillance capsule is currently under evaluation. We anticipate submitting the Unit 1 capsule analysis report by June 30, 1990 and the associated technical specification change request by October 31, 1990.

3) USI A-36, Control of Heavy Loads Near Spent Fuel

Generic Letter 81-07 and NUREG-0612 were issued to all power reactor licensees for implementation of methods for controlling heavy loads at nuclear power plants. All utilities were requested to evaluate their plants against the guidance of NUREG-0612 and to provide their responses in two phases. Our submittals AEP:NRC:0514 and 0514 Series A through W constitute our two-phase response and document the handling procedures for heavy loads at the Cook Nuclear Plant.

The NRC Safety Evaluation Report (SER) for Phase 1 was issued on September 20, 1983, and by issuing GL 85-11, "Completion of Phase II of Control of Heavy Loads at Nuclear Power Plants - NUREG-0612," dated June 28, 1985, the NRC concluded that a detailed review of Phase II was not required and Phase II was considered completed. Further, while not a requirement, the NRC encouraged the implementation of any actions identified in Phase II, as appropriate.

4) USI A-40, Seismic Design Criteria

USI A-40, involved evaluation of NUREG-CR-3805, CR-5347, and CR-3509, which concluded that further action on the part of the utilities was not required. However, further evaluations of the seismic structural stability of certain items such as above ground tanks are required. As a result, evaluation of these items has been included in the USI A-46 program. American Electric Power is a member of the Seismic Qualification Utility Group (SQUG), which was formed to systematically implement USI A-46 requirements. Therefore, USI A-40 is considered closed for Cook Nuclear Plant.

5) USI A-46: Seismic Qualification of Equipment in Operating Plants

The NRC issued GL 87-02 dated February 19, 1987, "Seismic Qualification of Electrical and Mechanical Equipment in Operating Plants," to all power plant licensees. The Seismic Qualification Utility Group (SQUG) is developing the criteria and procedures to implement the requirements of GL 87-02. The SQUG methods and approach have been approved by the NRC. Our submittal AEP:NRC:1040, dated October 1, 1988, documents the seismic qualification reevaluation program as it is applicable to Cook Nuclear Plant.

It is expected that the final Safety Evaluation Reports for the generic implementation procedures (GIPs) and relay evaluation and screening procedures will be issued by the NRC by April-May 1990. Assuming all other commitments between SQUG and the NRC will be on schedule, we are planning to perform the plant walkdown under the guidelines of the GIP during the 1992 refueling outages for both units.

This document has been prepared following Corporate procedures that incorporate a reasonable set of controls to ensure its accuracy and completeness prior to signature by the undersigned.

Sincerely,

M. P. Alexich
Vice President

MPA/eh

Attachment

cc: D. H. Williams, Jr.
A. A. Blind - Bridgman
R. C. Callen
G. Charnoff
NFEM Section Chief
A. B. Davis - Region III
NRC Resident Inspector - Bridgman

ENCLOSURE 1

UNRESOLVED SAFETY ISSUES FOR WHICH A FINAL TECHNICAL RESOLUTION HAS BEEN ACHIEVED

USI/MPA NUMBER	TITLE	REF. DOCUMENT	APPLICABILITY	STATUS/DATE*	REMARKS
A-1	Water Hammer	SECY 84-119 NUREG-0927, Rev. 1 NUREG-0993, Rev. 1 NUREG-0737 Item I.A.2.3 SRP revisions	All	NA AEP:NRC:0398 and 0678 Series of letters	Based on enclosure 2 to GL 89-21
A-2/ MPA D-10	Asymmetric Blowdown Loads on Reactor Primary Coolant Systems	NUREG-0609 GL 84-04, GDC-4	PWR	C AEP:NRC:0137D Dated: Sept. 10, 1984	NRC SER dated Nov. 22, 1985 Amendment No. 76 to Operating License No. DPR-74.
A-3	Westinghouse Steam Generator Tube Integrity	NUREG-0844 SECY 86-97 SECY 88-272 GL 85-02 (No requirements)	W-PWR	C AEP:NRC:0936 dt. June 21, 1985	Cook Unit 2 steam generators were replaced; for Unit 1 additional information is being provided to NRC under IE Bulletin 88-02
A-4	CE Steam Generator Tube Integrity	NUREG-0844, SECY 86-97 SECY 88-272 GL 85-02 (No requirements)	CE-PWR	-NA-	None
A-5	B&W Steam Generator Tube Integrity	NUREG-0844, SECY 86-97 SECY 88-272 GL 85-02 (No Requirements)	B&W-PWR	-NA-	None
E A-6	Mark I Containment Short-Term Program	NUREG-0408	Mark I-BWR	-NA-	None

* C - COMPLETE
NC - NO CHANGES NECESSARY
NA - NOT APPLICABLE
I - INCOMPLETE
E - EVALUATING ACTIONS REQUIRED

USI/MPA NUMBER	TITLE	REF. DOCUMENT	APPLICABILITY	STATUS/DATE*	REMARKS
A-7/ D-01	Mark I Long-Term Program	NUREG-0661 NUREG-0661 Suppl. 1 GL 79-57	Mark I-BWR	-NA-	None
A-8	Mark II Containment Pool Dynamic Loads	NUREG-0808 NUREG-0487, Suppl. 1/2 NUREG-0802 SRP 6.2.1.1C GDC 16	Mark II-BWR	-NA-	None
A-9	Anticipated Transients Without Scram	NUREG-0460, Vol. 4 10 CFR 50.62	All	I AEP:NRC:0838 Series of letters	Waiting for NRC review and SER on several items
A-10/ MPA B-25	BWR Feedwater Nozzle Cracking	NUREG-0619 Letter from DG Eisenhut dated 11/13/80 GL 81-11	BWR	-NA-	None
A-11	Reactor Vessel Material Toughness	NUREG-0744, Rev. 1 10 CFR 50.60/ 82-26	All	-NC-	
A-12	Fracture Toughness of Steam Generator and Reactor Coolant Pump Supports	NUREG-0577, Rev. 1 SRP Revision 5.3.4	PWR	-NC-	
A-17	Systems Interactions	Ltr: DeYoung to licensees - 9/77 NUREG-1174, NUREG-1229, NUREG/CR-3922, NUREG/CR-4261, NUREG/CR-4470, GL 89-18 (No requirements)	All	-NC-	See note in cover letter, Item 1
A-24/ MPA B-1	Qualification of Class 1E Safety-Related Equipment	NUREG-0588, Rev. 1 SRP 3.11 10 CFR 50.49 GL 82-09, GL 84-24 GL 85-15	All	-C-	Our letters: AEP:NRC:0356 Series AEP:NRC:0578 Series AEP:NRC:0775 Series

USI/MPA NUMBER	TITLE	REF. DOCUMENT	APPLICABILITY	STATUS/DATE*	REMARKS
A-26/ MPA B-04	Reactor Vessel Pressure Transient Protection	DOR Letters to Licensees 8/76 NUREG-0224 NUREG-0371 SRP 5.2 GL 88-11	PWR	I AEP:NRC:0894K dt. - Dec. 5, 1988 AEP:NRC:0894L dt. - Oct. 25, 1989	See cover letter, Item 2
A-31	Residual Heat Removal Shutdown Requirements	NUREG-0606 RG 1.113, RG 1.139 SRP 5.4.7	All OLs After 01/79.	-NA-	
A-36/ C-10, C-15	Control of Heavy Loads Near Spent Fuel	NUREG-0612 SRP 9.1.5 GL 81-07, GL 83-42, GL 85-11 Letter from DG Eisenhut dated 12/22/80	All	C AEP:NRC:0514 Series of letters NRC SER (Phase I) dt. Sept. 20, 1983	See cover letter, Item 3
A-39	Determination of SRV Pool Dynamic Loads and Pressure Transients	NUREG-0802 NUREGs-0763,0783,0802 NUREG-0661 SRP 6.2.1.1.C	BWR	-NA-	None
A-40	Seismic Design Criteria	SRP Revisions, NUREG/CR-4776, NUREG/CR-0054, NUREG/CR-3480, NUREG/CR-1582, NUREG/CR-1161, NUREG-1233, NUREG-4776 NUREG/CR-3805 NUREG/CR-5347 NUREG/CR-3509	All	-NC-	See cover letter, Item 4
A-42/ MPA B-05	Pipe Cracks in Boiling Water Reactors	NUREG-0313, Rev. 1 NUREG-0313, Rev. 2 GL 81-03, GL 88-01	BWR	-NA-	None

USI/MPA NUMBER	TITLE	REF. DOCUMENT	APPLICABILITY	STATUS/DATE*	REMARKS
A-43	Containment Emergency Sump Performance	NUREG-0510, NUREG-0869, Rev. 1 NUREG-0897, R.G.1.82 (Rev. 0), SRP 6.2.2 GL 85-22 No Requirements	All	C	No NRC submittal is required
A-44	Station Blackout	RG 1.155 NUREG-1032 NUREG-1109 10 CFR 50.63	All	C, our letter AEP:NRC:0537D dt. April 14, 1989	Awaiting NRC review
A-45	Shutdown Decay Heat Removal Requirements	SECY 88-260 NUREG-1289 NUREG/CR-5230 SECY 88-260 (No requirements)	All	C, AEP:NRC:1082 dt. Oct. 24, 1989	This has been incorporated into IPE program under GL 88-20
A-46	Seismic Qualification of Equipment in Operating Plants	NUREG-1030 NUREG-1211/ GL 87-02, GL 87-03	All	AEP:NRC:1040 I	See cover letter, Item 5
A-47	Safety Implication of Control Systems	NUREG-1217, NUREG-1218 GL 89-19	All	I	Under review and evaluation by AEPSC
A-48	Hydrogen Control Measures and Effects of Hydrogen Burns on Safety Equipment	10 CFR 50.44 SECY 89-122	All, except PWRs with large dry containments	I AEP:NRC:0476 and AEP:NRC:0500 Series	Remaining open items will be addressed in the IPE Program by AEPSC
A-49	Pressurized Thermal Shock	RGs 1.154, 1.99 SECY 82-465 SECY 83-288 SECY 81-687 10 CFR 50.61/ GL 88-11	PWR	C, AEP:NRC:0561A dt. January 22, 1986	NRC SER dated March 27, 1987