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ACCESSION NBR:8908090474 DOC.DATE: 89/08/04 NOTARIZED: NO DOCKET #
 FACIL:50-315 Donald C. Cook Nuclear Power Plant, Unit 1, Indiana & 05000315
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 RECIP.NAME RECIPIENT AFFILIATION

SUBJECT: LER 89-009-00:on 890620,required post-maint testing not
 performed due to personnel error prior to entry into mode.
 W/8 ltr.

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August 4, 1989

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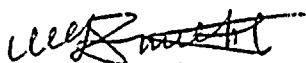
Operating License DPR-58
Docket No. 50-315

Document Control Manager:

In accordance with the criteria established by 10 CFR 50.73
entitled Licensee Event Reporting System, the following
report is being submitted:

89-009-00

Sincerely,


W. G. Smith, Jr.
Plant Manager

WGS:clw

Attachment

cc: D.H. Williams, Jr.
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LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) D. C. COOK NUCLEAR PLANT - UNIT 1										DOCKET NUMBER (2) 0 5 0 0 0 3 1 5					PAGE (3) 1 OF 0 3	
TITLE (4) Required Post-Maintenance Testing not performed due to personnel error prior to entry into a mode for which the equipment was required to be operable.																
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)						
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES				DOCKET NUMBER(S)			
0 6	2 0	8 9	8 9	0 0 9	0 0	0 8	0 4	8 9					0 5 0 0 0			
OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)														
1		20.402(b)				20.405(c)				50.73(a)(2)(iv)				73.71(b)		
POWER LEVEL (10)		20.405(a)(1)(i)				50.36(c)(1)				50.73(a)(2)(v)				73.71(c)		
0 3 2		20.405(a)(1)(ii)				50.36(c)(2)				50.73(a)(2)(vi)				OTHER (Specify in Abstract below and in Text, NRC Form 366A)		
		20.405(a)(1)(iii)				50.73(a)(2)(i)				50.73(a)(2)(vii)(A)						
		20.405(a)(1)(iv)				50.73(a)(2)(ii)				50.73(a)(2)(viii)(B)						
		20.405(a)(1)(v)				50.73(a)(2)(iii)				50.73(a)(2)(ix)						
LICENSEE CONTACT FOR THIS LER (12)																
NAME J. B. Droste - Maintenance Superintendent										TELEPHONE NUMBER						
										AREA CODE 6 1 6 4 6 5 - 5 9 0 1						
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC						
SUPPLEMENTAL REPORT EXPECTED (14)										EXPECTED SUBMISSION DATE (15)			MONTH	DAY	YEAR	
<input checked="" type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)										<input type="checkbox"/> NO			0 8	1 8	8 9	
ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)																

This an an interim report pending completion of analysis of event and determination of preventive action.

On July 5, with Unit 1 operating at 32 percent reactor power it was discovered during a review of job order packages that post-maintenance testing documentation for a job order on valve No. 1-CS-442-2, Unit 1 reactor coolant pump No. 2 seal water injection containment isolation check valve had not been signed off as completed. A record search was conducted and no evidence could be found to indicate that the post maintenance testing had been performed. This placed the unit in a condition outside of the operability requirements of Technical Specification 3.6.1.2 Limiting Condition for Operation and a shutdown was initiated per the requirements of T/S 3.0.3 at 1545 hours on July 5, 1989. An Unusual Event was declared at 1605 hours and the NRC was notified via the ENS at 1613 hours. The RCS was placed in Hot Standby at 2132 hours, entered Hot Shutdown at 0305 on July 6, 1989 and Cold Shutdown at 0909 hours the same day.

Upon discovery of this condition the unit was placed in cold shutdown and the valve was tested. The test indicated that the valve was functioning satisfactorily; therefore, containment integrity had not been compromised and no other safety concerns were noted.
Additional details will be included in a supplemental report.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1) D. C. Cook Nuclear Plant - Unit 1	DOCKET NUMBER (2) 0 5 0 0 0 3 1 5 8 9	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		— 0	0 9	— 0 0	0 2	OF	0 3

TEXT (If more space is required, use additional NRC Form 366A's) (17)

This is an interim report pending completion of analysis of event and determination of preventive action.

CONDITIONS PRIOR TO OCCURANCE

Unit One - 32 Percent Thermal Power
Unit Two - 100 Percent Thermal Power

DESCRIPTION OF EVENT

On July 5, 1989 with Unit 1 operating at 32 percent reactor power it was discovered during a review of job order packages that post-maintenance testing documentation for a job order on valve No. 1-CS-442-2, Unit 1 reactor coolant pump No. 2 seal water injection containment isolation check valve (EIIS/ISV-CB) had not been signed off as completed. A record search was conducted and no evidence could be found to indicate that the post maintenance testing had been performed. This placed the unit in a condition outside of the operability requirements of Technical specification 3.6.1.2 Limiting Condition for Operation and a shutdown was initiated per the requirements of T/S 3.0.3 at 1545 hours on July 5, 1989. An Unusual Event was declared at 1605 hours and the NRC was notified via the ENS at 1613 hours. The RCS was placed in Hot Standby at 2132 hours, entered Hot Shutdown at 0305 on July 6, 1989 and Cold Shutdown at 0909 hours the same day.

On May 2, 1989 following a maintenance inspection and bonnet gasket replacement on valve 1-CS-442-2 the repair job order and its associated paperwork were submitted for closeout with the understanding that the required post maintenance testing (Type C leakage rate testing) would be completed under a separate testing job order prior to entry into a mode for which the equipment was required to be operable. Unit 1 entered Hot Shutdown on June 20, 1989 at 0330 hours with the Type C testing not having been performed.

Upon discovery of this condition the unit was placed in cold shutdown and the valve was tested. The test indicated that the valve was functioning satisfactorily; therefore, containment integrity had not been compromised and no other safety concerns were noted.

Additional details will be included in supplemental report.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/88

FACILITY NAME (1) D. C. Cook Nuclear Plant - Unit 1	DOCKET NUMBER (2) 0 5 0 0 0 3 1 5 8 9	LER NUMBER (8)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
			- 0 0 9	- 0 0	0 3	OF	0 3

TEXT (If more space is required, use additional NRC Form 366A's) (17)

CAUSE OF EVENT

The cause of this event was personnel error due to the fact that the required testing was not verified complete prior to declaring the valve operable.

Additional details will be included in supplemental report.

ANALYSIS OF EVENT

This event is considered reportable per the requirements of 10 CFR 50.73 (a) (2) (i) (B) as Unit 1 entered hot shutdown from cold shutdown with a containment isolation valve not proven operable.

CORRECTIVE ACTIONS

To be addressed in supplemental report, scheduled to be submitted August 18, 1989.